



**RIC 2014  
State-of-the-Art Reactor  
Consequence Analyses (SOARCA)  
Peach Bottom Uncertainty Analysis**

Tina Ghosh, PhD  
RES/DSA  
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**Goals of the Uncertainty Analysis**

- Develop insight into overall sensitivity of SOARCA results and conclusions to uncertainty in inputs
- Identify most influential input parameters for releases and consequences
- Demonstrate uncertainty analysis (UA) methodology for future studies

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**Study Approach**

- An integrated uncertainty analysis of key parameters completed for the Peach Bottom, unmitigated, long-term station blackout scenario
- Monte Carlo simulation carried out with 865 samples of ~40 independent uncertain input parameters
- Results studied include cesium and iodine release, latent cancer fatality (LCF) risk, early fatality risk
- Tools used to analyze results include statistical regression-based methods as well as scatter plots and investigation of individual realizations of interest
- Handful of separate sensitivity analyses also completed

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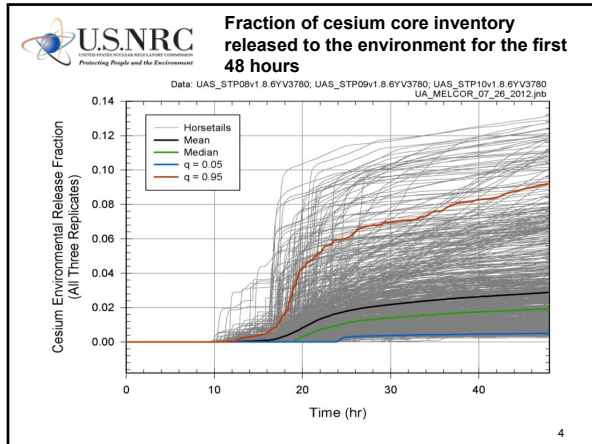
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**Conditional, mean (over weather) individual LCF risk statistics for the 10- and 50-mile radial areas around the plant**

	0-10 miles	0-50 miles
<b>5<sup>th</sup> percentile</b>	$3 \times 10^{-5}$	$2 \times 10^{-5}$
<b>Median</b>	$1 \times 10^{-4}$	$7 \times 10^{-5}$
<b>Mean</b>	$2 \times 10^{-4}$	$1 \times 10^{-4}$
<b>95<sup>th</sup> percentile</b>	$4 \times 10^{-4}$	$3 \times 10^{-4}$
<b>SOARCA UA Base Case</b>	$9 \times 10^{-5}$	$3 \times 10^{-5}$

*Note that the scenario frequency is  $\sim 3 \times 10^{-6}$  per reactor year*

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**Conditional, mean (over weather) individual early-fatality risk statistics for given radial areas**

	0 - 1.3 miles	0 - 2.5 miles	0 - 5 miles	0 - 10 miles
Median and 75 <sup>th</sup> percentile	0.0	0.0	0.0	0.0
Mean	$4.5 \times 10^{-7}$	$8.9 \times 10^{-8}$	$1.4 \times 10^{-8}$	$4.8 \times 10^{-9}$
95 <sup>th</sup> percentile	$1.9 \times 10^{-6}$	$3.5 \times 10^{-8}$	0.0	0.0

*Note that the scenario frequency is  $\sim 3 \times 10^{-6}$  per reactor year*

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### Overall UA Conclusions

- Peach Bottom UA corroborates SOARCA study conclusions
  - Public health consequences from severe nuclear accident scenarios modeled are smaller than previously calculated, and very small in absolute terms
  - Delayed releases calculated provide more time for emergency response actions such as evacuating or sheltering; long-term phase dominates health effect risks because emergency response is faster than progression to release
  - Essentially zero early fatality risk

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### Overall Conclusions (2)

- A major determinant of source term magnitude and health consequences is whether or not Main Steam Line (MSL) rupture occurs (leads to higher consequences)
- Health-effect risks vary sublinearly with source term because people are not allowed to return to their homes until dose is below habitability criterion
- Analysis confirms known importance of some phenomena, and reveals some new phenomenological insights
- The use of multiple techniques to post-process Monte Carlo results provides better explanatory power of which input parameters are most important to uncertainty in results

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### Most Influential Parameters for Individual LCF Risk

- MACCS2 dry deposition velocity
- MELCOR safety relief valve (SRV) stochastic failure rate
- MACCS2 residual cancer risk factor
- MELCOR fuel failure criterion
- MELCOR drywell liner melt-through open flow area
- MACCS2 residual dose and dose-rate effectiveness factor

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