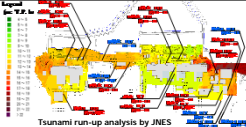


JNES
 Incorporated Administrative Agency
 Japan Nuclear Energy Safety Organization:
 Japanese TSO for Safety Regulation



Tsunami run-up analysis by JNES

Post-Fukushima Safety Research in Japan

Masashi HIRANO
 Japan Nuclear Energy Safety Organization (JNES)

Presented at
 U.S. NRC Regulatory Information Conference,
 Washington, March 2012

JNES 1

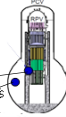
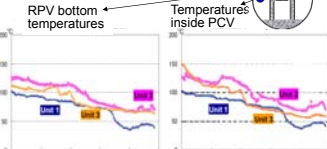
Outline

- **Overview**
 - Current Status of Fukushima Daiichi NPP and Nuclear Power in Japan
- **Safety Research with Short-term Needs**
 - Seismic and Tsunami Hazard Evaluation
 - Upgrading of SA Progression and Source Term Analysis
 - Upgrading of Accident Management Measures
 - Hydrogen Mixing and Detonation Behavior
 - Environmental Consequence Analysis (Level 3 PSA)
- **Summary: Post-Fukushima Safety Research Needs and Priorities**

JNES 2

Overview of Current Status (1/2)

- **Stress test is in progress:**
 - Only 2 units are in operation while the rest of 52 units are in shutdown. (as of Feb. 21, 2012) Three relevant ministers decided the test be a necessary condition for restarting.
 - A total of 16 units have submitted the stress test results to NISA. (as of Feb. 17, 2012)
- **Reform of regulatory framework will be taken place:**
 - On Aug. 15, 2011, the Cabinet made decision to create a new regulatory body as an external agency of the Ministry of the Environment, in April 2012. Relevant part of NISA and the function of NSC will be merged.
- **On Dec. 16, 2011, the Government declared the completion of roadmap towards restoration from accident conditions at Fukushima Daiichi NPP.**
 - Stable core cooling has been established. Leaked / accumulated water is processed and injected back into the RPVs.
 - All the temps measured at RPV bottom and inside PCV became well below 100°C. The radiation dose at the site boundaries due to additional release is assessed to be 0.1mSv/y.

TEPCO Press Release (Nov. 17, 2011)

JNES Overview of Current Status (2/2) 3

Camera-mounted devices

- On Dec. 21, 2011, the government and TEPCO jointly set forth the **mid-and-long-term roadmap towards decommissioning of Fukushima Daiichi Units 1-4**:
 - Phase 1: to start removal of fuel from SFPs within 2 years
 - Phase 2: to start removal of fuel debris within 10 years
 - Phase 3: decommissioning in 30 – 40 years
- This roadmap includes various specific plans for R&D activities such as:
 - Remote decontamination of R/B interior
 - Technologies for **identification of leak areas** in PCV and investigation of PCV interior
 - PCV repair technologies

An example: R&D for **identifying the leak areas in PCV**

Item/Year	Phase 1	Phase 2	Phase 3	2014
	2011	2012	2013	(beginning)
1. Examination of inspection methods and designing of devices	[Progress bar]			
2. Construction and improvement of inspection devices	[Progress bar]			

Reference: http://www.tepco.co.jp/en/press/corp-com/release/betu11_e/images/111221e14.pdf

Conceptual diagram of the investigation of leak areas in the PCV

- It is expected that indispensable information could be obtained from the viewpoint of further event investigation and also SA progression and source term evaluation.



JNES Safety Research with Short-term Needs - Seismic and Tsunami Hazard Evaluation – (1/2) 4

- JNES estimated a **tsunami source model** through **joint inversion analysis** based on references [1] and [2], as well as the characteristics of the tsunami resulted from the Tohoku earthquake. The source model includes the source rupture process.
- Then, JNES conducted a **tsunami propagation analysis** by using the source model.

Source Model: Time history of slides at different sub faults obtained by the inversion analysis

Observed and calculated tsunami waveforms

Wave Height (m)

Fukushima Daiichi NPP

Onagawa NPP

Tokai Daini NPP

No data due to failure of tide gauge

The observed tsunami waveforms were well reproduced not only at Fukushima Daiichi site but also other sites.

[1]: Tsunami source model (Ver. 4.2)(Fuji and Satake, 2011), http://fisee.kenken.go.jp/stattmju/ut/tonoku/acric2011/tsunami_ja.html
 [2]: Tsunami simulation for the Tohoku District-Off the Pacific Ocean Earthquake (Tohoku University Model) <http://www.tsunami.civil.tohoku.ac.jp/hokusaiz/>



JNES Safety Research with Short-term Needs - Seismic and Tsunami Hazard Evaluation – (2/2) 5

- In this area, urgent research is required, based on the knowledge and experience gained from the present tsunami, for development of:
 - Methods for tsunami source model, and
 - Guides for design basis tsunami height by using probabilistic hazards assessment methodology.
- Rupture process is one of the keys:
 - The inversion analysis by JNES showed that the slide expanded from original rupture area to somewhat deeper areas, and moved to shallow areas along the trench, where the max. slide displacement was more than 70m.
 - The timing and time duration of rupture at each sub fault have large effects on the tsunami waveforms.
- Regarding safety assessment such as tsunami PSA, further R&D may be needed:
 - Development of methods and models to evaluate dynamic tsunami wave force that impacts structural integrity / water tightness.

Observed and calculated tsunami waveforms

Sum of the slides in sub fault in JNES source model (Inversion analysis)

JNES 2011 Symposium, Dec. 22, 2011



JNES Safety Research with Short-term Needs - Upgrading of SA Progression and Source Term Analysis -

6

- Fukushima accident analysis with MELCOR in JNES:
 - Overall trends of the plant data during the accident were reproduced reasonably well by MELCOR as presented at the IAEA Ministerial Conference in June 2011.
 - Onsite monitoring data, that may indicate source term, are also reproduced relatively well.
 - However, fuel melt progression depends largely on the amount of water injected and other factors. Reliable prediction of the debris distribution within RPV and PCV is difficult [1].

Unit 3
Case with large amount of water injected 50hr 120hr
Case with small amount of water injected 50hr 120hr

Core
Lower plenum
Large mass of debris

- TEPCO's analysis with MAAP [2]:
 - 100%, 57% and 63% of total fuel debris exists on the pedestal region in PCV in Unit 1, 2 and 3, respectively.
 - Max. penetration depth into concrete is 0.65m.

[1]: <http://www.nisa.meti.go.jp/shingikai/700/12/001/1-6-1.pdf>
[2]: <http://www.nisa.meti.go.jp/shingikai/700/12/001/1-5.pdf>

JAEA Safety Research with Short-term Needs - Upgrading of Accident Management Measures -

7

- JAEA is a R&D organization under MEXT. The mission of the Nuclear Safety Research Center in JAEA is to contribute to enhancement of safety by supporting safety regulation.

Thermal-Hydraulic Safety Research

- Clarify accident causes and detailed consequences under beyond-DBEs
- Confirm effectiveness of safety features against prevention & mitigation of SA
- Prepare own prediction methods to demonstrate prototypical accident phenomena with high accuracy
 - Development of new test facility
 - Development and V&V of BE methods

Fuel Safety Research

- Evaluate behavior and marginal performance of fuel under beyond-DBE conditions
 - Irradiation tests at NSRR with high enthalpies
 - Oxidation tests of the cladding under various severe loss-of-coolant conditions
 - Characterization of damage fuel rod components
 - Development of computer codes

- RIA
 - Fuel rod degradation process
 - Energy release on fuel failure
 - Influence on core coolability
- LOCA
 - Oxidation behavior
 - Physical and mechanical properties of oxidized cladding

Computer code development

JNES Safety Research with Short-term Needs - Hydrogen Mixing and Detonation Behavior -

8

- JNES is conducting analyses of hydrogen mixing and detonation in R/Bs:
 - MELCOR for hydrogen source evaluation, FLUENT (CFD code) for hydrogen transport and mixing, and AUTODYN for structural analysis of detonation.
 - The objectives are to better understand the phenomena that took place in Unit 1 and 3, and to assess and improve the methods and tools.

- Some results from the analysis of detonation at Unit 3:
 - The amount of H₂ leaked into R/B is estimated to be app. 1 ton.
 - If it is assumed that the leakage took place at S/C, namely, lower part of R/B, overall detonation behavior is well reproduced.

R/B: Reactor Building
