

Vogtle 3 & 4 COL

Southern Nuclear Operating Company VEGP Units 3 & 4
Combined Licenses and Limited Work Authorization

Mandatory Hearing

Safety – Panel 4

September 27, 2011

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Selected Topics - Chapters 9, 12, 13, 14

- Emergency Planning (addressed in ESP)
- Technical Support Center (TSC)
- Cyber Security
- Spent Fuel Pool Redesign
- ALARA
- Minimization of Contamination
- First (three) plant(s) testing



Emergency Planning

- Complete and integrated emergency plan with site specific ITAAC approved as part of the Early Site Permit (ESP) with seven Permit Conditions
 - Six permit conditions (three for each unit) related to emergency actions levels (EALs) – converted to single license condition for each unit
 - One permit condition related to approval of the Technical Support Center (TSC) – closed in the Combined License (COL) Final Safety Evaluation Report (FSER)



Emergency Planning (continued)

- Emergency Action Levels
 - Because certain design details are not complete, some EAL setpoints have not yet been established
 - ◆ A COL license condition was proposed that a complete set of EALS be submitted for NRC review at least 180 days before fuel load
 - ◆ The EALs will be in accordance with Nuclear Energy Institute (NEI) 07-01, Rev. 0, with no deviations
 - The EALs will have agreement of State and local officials



Technical Support Center

- ◆ The ESP referenced AP1000 DCD Rev. 15.
 - Rev. 15 did not include the same material regarding the location of the TSC as AP1000 DCD Rev. 19
- NRC Staff deferred regulatory action on the location of the TSC to the COL phase
 - Location of the TSC addressed in Departure 18-1
 - Radiological and non-radiological habitability issues addressed
 - ◆ITAAC adjusted to address habitability issues



Technical Support Center (continued)

- Designed to handle an accident that affects one or all four of the units at Vogtle
- Designed to accommodate the emergency plan provisions for all four units to be located at the VEGP site



Cyber Security

- Cyber Security Plan (CSP) provides programmatic details for meeting the new requirements of 10 CFR 73.54
 - Based on RG 5.71 Rev. 0 template, with deviations
 - Deviations are provided to:
 - Clarify RG 5.71 intent,
 - Provide flexibility, and
 - Address alternative approaches to meeting Rule requirements
 - CSP summarizes how each deviation meets RG intent or does not reduce plant protection
- Cyber Security Program is an operational program, as per SECY-05-0197, and will be implemented by License Condition



Spent Fuel Pool Redesign

- DCD addresses spent fuel pool design and evaluations
 - ◆ Section 9.1.2 Spent Fuel Storage
 - Design Descriptions
 - Criticality and Seismic Evaluations
 - ◆ This information incorporated by reference into COLA at FSAR Section 9.1.
- COLA FSAR addresses site-specific considerations
 - COL 3.7-2 addresses procedures for verification of spent fuel pool rack-to-wall gap dimensions following a seismic event
 - SUP 9.1-3 addresses safe load paths for heavy loads near the spent fuel pool
 - COL 9.1-7 addresses the metamic coupon monitoring to check for swelling and boron depletion



Raw Water System

- Site specific RWS addressed in COLA FSAR 9.2.11
- No safety related functions
- RWS sources include river water and well water
- Provides makeup to circulating water and other systems
- Provides makeup to cooling tower basins
- Provides dilution for radwaste discharge

ALARA and Minimization of Contamination



- ◆ COLA Incorporates DCD Section 12.4 by Reference and Supplements DCD Section 12.4
 - Addresses Radiation Exposure to Construction Workers
 - Construction worker doses were conservatively estimated
 - ◆ COLA FSAR Table 12.4-201 shows estimated maximum annual dose for each pathway as well as the total dose
- ♦ COLA Incorporates DCD Section 12.5 by Reference
 - Supplements DCD Section 12.5 with Appendix 12AA, "Radiation Protection Program Description"
 - adopts NEI 08-08A for a description of the operational and programmatic elements and controls that minimize contamination



- Pre-operational Testing
 - In-Containment Refueling Water Storage Tank (IRWST) Heatup Test
 - first plant only test
 - ◆ DCD Subsection 14.2.9.1.3 Item (h)
 - Pressurizer Surge Line Stratification Evaluation
 - first plant only test
 - ◆ DCD Subsection 14.2.9.1.7 Item (d)
 - Reactor Vessel Internals Vibration Testing
 - first plant only test
 - DCD Subsection 14.2.9.1.9



- Pre-operational Testing (cont'd)
 - Core Makeup Tank Heated Recirculation Tests
 - first three plants only tests
 - ◆ DCD Subsection 14.2.9.1.3 Item (k)
 - ◆ DCD Subsection 14.2.9.1.3 Item (w)
 - Automatic Depressurization System Blowdown Test
 - first three plants only test
 - ◆ DCD Subsection 14.2.9.1.3 Item (s)



- Initial Criticality and Low-Power Testing
 - Natural Circulation
 - first plant only test
 - DCD Subsection 14.2.10.3.6
 - Passive Residual Heat Removal Heat Exchanger
 - first plant only test
 - DCD Subsection 14.2.10.3.7



- Power Ascension Testing
 - Rod Cluster Control Assembly Out of Bank Measurements
 - first plant only test
 - DCD Subsection 14.2.10.4.6
 - Load Follow Demonstration
 - first plant only test
 - DCD Subsection 14.2.10.4.22



Abbreviations and Acronyms

ACRS Advisory Committee on Reactor Safeguards

CFR Code of Federal Regulations

COL Combined License

CSP Cyber Security Plan

ESP Early Site Permit

FSER Final Safety Evaluation Report

ITAAC Inspections, Tests, Analyses and Acceptance

Criteria

RG Regulatory Guide

STD COL Standard Combined License information

STD SUP Standard supplemental information

TSC Technical Support Center

UNITED STATES OF AMERICA NUCLEAR REGULATORY COMMISSION

BEFORE THE COMMISSION

In the Matter of ()))
Southern Nuclear Operating Company	
(Vogtle Electric Generating Plant,) Units 3 and 4)	September 20, 2011

CERTIFICATE OF SERVICE

I hereby certify that copies of EXHIBIT SNC000009 for the Vogtle Units 3 & 4 COL Mandatory Hearing in the above-captioned proceeding have been served by electronic mail as shown below, this 20th day of September, 2011, and/or by e-submittal.

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