

April 24, 2002

The Honorable Harry Reid, Chairman
Subcommittee on Transportation,
Infrastructure, and Nuclear Safety
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510

Dear Mr. Chairman:

The Fiscal Year (FY) 2002 Energy and Water Development Appropriations Act, House Report 107-258, directed the Nuclear Regulatory Commission (NRC) to continue to provide a monthly report on the status of its licensing and regulatory duties. The initial reporting requirement arose in the FY 1999 Energy and Water Development Appropriations Act, Senate Report 105-206. The FY 2000 Energy and Water Development Appropriations Act, House Report 106-253, expanded the scope of the report requirement to include regulatory reform efforts affecting power reactor operations beyond 10 CFR Part 50, particularly NRC efforts to evaluate NRC security regulations. In FY 2000, we also expanded the monthly report to include the status of all license renewal applications that are under active review and other NRC initiatives in developing implementation guidance for the license renewal rule. In response to increased Congressional interest, in the May 2001 report we began to provide information regarding the status of activities involving power uprate licensing actions. On behalf of the Commission, I am pleased to transmit the thirty-ninth report, which covers the month of February 2002 (Enclosure 1).

The January 2002 report provided information on a number of significant NRC activities including an update of our actions taken following the terrorist attacks of September 11. On February 26, 2002, the NRC issued Orders to all 104 commercial nuclear power plants to implement interim compensatory security measures for the current threat environment. Some of the requirements in the Orders formalize a series of security measures that NRC licensees had taken in response to advisories issued by the NRC following the September 11 terrorist attacks. Additional security enhancements, which have emerged during NRC's on-going comprehensive security review, are also spelled out in the Orders. The requirements will remain in effect until such time as the Commission determines that the level of threat has diminished, or that other security changes are needed following a comprehensive re-evaluation of current safeguards and security programs. We will continue to keep you informed of the status of our activities in this area.

On April 7, 2002, we established the Office of Nuclear Security and Incident Response (NSIR) to consolidate and streamline selected NRC security, safeguards, and incident response responsibilities and resources. The newly created office reports to the Deputy Executive Director for Reactor Programs. The formation of the new office is one result of the Commission's ongoing comprehensive review of its safeguards and physical security program in the aftermath of last September's terrorist attacks.

The Commission is now investigating an incident involving reactor vessel head corrosion at the Davis-Besse Nuclear Power Station in Oak Harbor, Ohio. On February 16, 2002, the licensee for Davis-Besse, FirstEnergy Corporation, began a refueling outage that included inspecting certain nozzles in the head of the reactor pressure vessel. The licensee's inspection focused on the nozzles associated with the mechanism that drives the control rods. Both the inspections and their focus were consistent with the licensee's commitments in response to NRC Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles," which the agency issued on August 3, 2001. During these inspections the licensee identified three penetration nozzles with indications of axial cracking that resulted in leakage. While repairing one of these penetration nozzles, the licensee detected a cavity on the top of the reactor vessel head in the area surrounding the penetration nozzle (the reactor vessel head is fabricated from carbon steel with a stainless steel liner and is approximately 6.5 inches thick). The cavity in the carbon steel head material was measured to be approximately 6 inches deep, 7 inches long, and 5 inches wide. The remaining thickness of the reactor pressure vessel head in this area was reported to be that of stainless steel liner material (approximately 3/8 inches thick). The root cause of this condition is currently under investigation by the licensee. Preliminary assessments suggest that the cause may be corrosion due, in part, to the presence of boric acid used in the reactor coolant system. On March 12, 2002, the NRC dispatched an augmented inspection team to the Davis-Besse facility to collect and analyze factual information and evidence related to this issue. Additionally, a Confirmatory Action Letter was issued to the licensee on March 15, 2002 to confirm FirstEnergy's commitments regarding the actions it will take to evaluate and resolve the damaged reactor vessel head. These actions include obtaining NRC approval prior to restart. On March 18, 2002, the NRC issued a bulletin to PWR licensees requiring information on the structural integrity of the reactor vessel head and the basis for concluding that the reactor vessel head will continue to perform its function as a reactor coolant pressure boundary. We will continue to keep you informed of the status of this issue.

Since our last report, the Commission and the NRC staff also:

- released to the public, on March 4, 2002, Draft Revision 2 of a plan that the NRC would use to review an application to build a potential high-level nuclear waste repository at Yucca Mountain, Nevada, if the Department of Energy submits such an application. The principal purpose of the Yucca Mountain Review Plan would be to ensure the quality and uniformity of the NRC staff's licensing reviews.
- held three public meetings in Nevada between April 8-10, 2002 to discuss safety and regulatory issues regarding a potential application from the Department of Energy to build and operate a high-level radioactive waste repository at Yucca Mountain, Nevada. The meetings included an overview of NRC's responsibilities, a presentation on the NRC's regulations and preparations for evaluating a potential DOE license application, and concluded with a discussion on the NRC role with respect to the transportation of high-level waste. Members of the public were afforded the opportunity to ask questions.
- published in the Federal Register (67 FR 14818) on March 27, 2002 a proposed rule that would amend the licensing, inspection, and annual fees charged to the Commission's applicants and licensees. The proposed rule is intended to ensure NRC compliance with the Omnibus Budget and Reconciliation Act of 1990. The Act mandates that the NRC recover approximately 96 percent of its budget authority in

Fiscal Year 2002, less amounts appropriated from the Nuclear Waste Fund and the General Fund. The amount to be recovered for FY 2002 is approximately \$479.5 million. The comment period on this action closes April 26, 2002.

- approved a request by Entergy Operations, Inc. to increase the generating capacity of the Waterford Electric Station, Unit 3, by about 1.5 percent, or about 16 megawatts of electricity (MWe). The power uprate at the plant, 20 miles west of New Orleans, Louisiana, will increase the generating capacity of the reactor to about 1,169 MWe. The facility intends to implement the power increase during its current outage.
- approved a request by AmerGen Energy Company, LLC, to increase the generating capacity of the Clinton Power Station by about 20 percent, or about 186 MWe. The power uprate at the plant, near Clinton, Illinois, will increase the generating capacity of the reactor to about 1,116 MWe. The facility intends to implement the power increase in two phases beginning this spring.
- published in the Federal Register (76 FR 16298) on April 5, 2002 a final rule amending 10 CFR Part 20 to change the definition and method of calculating shallow-dose equivalents (SDEs) by specifying that the assigned SDE must be the dose averaged over the 10 square centimeters of skin receiving the highest exposure, rather than 1 square centimeter as stated in the existing regulation. This rulemaking serves to make the skin dose limit less restrictive when small areas of skin are irradiated (i.e., more representative of actual health risks) and to address skin and extremity doses from all source geometries under a single limit. The Commission believes that this change represents a substantial increase in worker protection because reduced monitoring for discrete radioactive particles will result in reduced external dose and will result in fewer industrial hazards in the workplace as a result of reduced use of protective clothing.
- received from the Westinghouse Electric Company an application for design certification of its AP1000 standard plant design. The plant features enhanced safety systems that rely on gravity and pressure differentials to shut down the reactor or mitigate the effects of an accident. It is designed for a 60-year operating life. In submitting its application for design certification, Westinghouse referenced the AP600 standard design, which was certified by NRC in 1999. NRC staff will perform an acceptance review to determine whether the application contains sufficient information to be processed. If it is found acceptable, NRC will publish a notice in the Federal Register announcing its acceptance and docketing of the application.
- conducted a public meeting on February 20 - 21, 2002 with the Nuclear Energy Institute, Materials Reliability Program, Electric Power Research Institute, various utility representatives, and other stakeholders to discuss progress on establishing a control rod drive mechanism action plan and the status of industry progress on a variety of technical issues.
- published in the Federal Register on February 15, 2002 draft wording of a possible amendment to the regulations governing the NRC's fitness-for-duty program. The NRC is seeking public comment on the draft rule outline and on individual sections of the draft regulatory text.

- published in the Federal Register on February 11, 2002 a proposed rule that would add an entry for the Standardized Advanced NUHOMS-24PTI cask system to the list of approved spent fuel storage casks. The proposed rule would allow holders of power reactor operating licenses to store spent fuel in this storage system under a general license.
- awarded a contract on February 1, 2002 to Anbex, Inc., to supply potassium iodide (KI) tablets to requesting States. To date, Maryland, Massachusetts, New York, New Hampshire, Vermont, Connecticut, Arizona, Florida and Alabama have received KI shipments. In March 2002, Delaware and New Jersey submitted requests which have been forwarded to the Federal Emergency Management Agency for a reasonableness review.
- completed a survey to obtain information regarding industry's plans related to power uprate applications over the next 5 years. The survey results indicate that 38 power uprate applications will be submitted in the next 5 years. Of the 38 applications, 23 will request power uprates less than 2 percent and 14 will request power uprates greater than approximately 7 percent. One licensee did not report a magnitude for its planned power uprate. Planned power uprates are expected to result in an increase in generating capacity of approximately 1590 megawatts electric, which is equivalent to more than one large nuclear power plant.

I have enclosed (Enclosure 2) the update to the Tasking Memorandum which delineates the schedules for accomplishing high priority initiatives.

Please do not hesitate to contact me if I may provide additional information.

Sincerely,

/RA/

Richard A. Meserve

Enclosures:

1. Monthly Report
2. Tasking Memorandum

cc: Senator James M. Inhofe

Identical letter to:

The Honorable Harry Reid, Chairman
Subcommittee on Transportation,
Infrastructure, and Nuclear Safety
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510
cc: Senator James M. Inhofe

The Honorable Joe Barton, Chairman
Subcommittee on Energy and Air Quality
Committee on Energy and Commerce
United States House of Representatives
Washington, D.C. 20515
cc: Representative Rick Boucher

The Honorable Sonny Callahan, Chairman
Subcommittee on Energy and Water Development
Committee on Appropriations
United States House of Representatives
Washington, D.C. 20515
cc: Representative Peter J. Visclosky

The Honorable Harry Reid, Chairman
Subcommittee on Energy and Water Development
Committee on Appropriations
United States Senate
Washington, D.C. 20510
cc: Senator Pete V. Domenici

The Honorable W.J. "Billy" Tauzin, Chairman
Committee on Energy and Commerce
United States House of Representatives
Washington, D.C. 20515
cc: Representative John D. Dingell

The Honorable James M. Jeffords, Chairman
Committee on Environment and Public Works
United States Senate
Washington, D.C. 20510
cc: Senator Bob Smith

The Honorable Pete V. Domenici
United States Senate
Washington, D.C. 20510

MONTHLY STATUS REPORT ON THE
LICENSING ACTIVITIES AND REGULATORY DUTIES OF THE
UNITED STATES NUCLEAR REGULATORY COMMISSION

FEBRUARY 2002

Enclosure 1

TABLE OF CONTENTS¹

I.	Implementing Risk-Informed Regulations	2
II.	Revised Reactor Oversight Process	2
III.	Status of Issues in the Reactor Generic Issue Program	3
IV.	Licensing Actions and Other Licensing Tasks	3
V.	Status of License Renewal Activities	9
VI.	Status of Review of Private Fuel Storage, Limited Liability Corporation's Application for a License to Operate an Independent Spent Fuel Storage Installation on the Reservation of the Skull Valley Band of Goshute Indians	10
VII.	Enforcement Process and Summary of Reactor Enforcement by Region	11
VIII.	Power Reactor Security Regulations	12
IX.	Power Upgrades	13

¹Note: The period of performance covered by this report includes activities occurring between the first and last day of February 2002. The transmittal letter to Congress accompanying this report may provide more recent information in order to keep Congress fully and currently informed of NRC's licensing and regulatory activities.

XIII. Implementing Risk-Informed Regulations

The staff continues to make progress on tasks involving use of probabilistic risk information in many areas. Although various activities are in progress, in February 2002 we did not reach a milestone of significance that warrants separate reporting. The milestone schedule for significant risk-informed activities is included in the Chairman's Tasking Memorandum (Enclosure 2).

XIV. Revised Reactor Oversight Process

The NRC continues to implement the Reactor Oversight Process (ROP) at all nuclear power plants. The NRC has continued meeting with interested stakeholders on a periodic basis to collect feedback on the efficacy of the process and considers this feedback in making refinements to the ROP. Recent activities include:

- a. The ROP efficiency focus group developed a list of areas for possible efficiency gains in the oversight of reactors and criteria for evaluating the suggestions. On February 7, 2002, NRR staff held a video conference with regional division management to review the results of the ROP efficiency focus group suggestions, provide any necessary clarification, reach agreement on the evaluation criteria, and assign a weighting factor to each criterion based on its relative importance. The next task for the efficiency focus group is to systematically determine the extent to which each suggestion meets the evaluation criteria in order to identify 3-5 suggestions that yield the highest score for possible implementation.
- b. End-of-cycle (EOC) reviews for the assessment period that ended on December 31, 2001, were completed on February 13, 2002, for each operating nuclear power plant. The Regions performed the reviews in accordance with the guidance provided in Inspection Manual Chapter (IMC) 0305, "Operating Reactor Assessment Program." NRR staff, along with other NRC staff representatives, participated in the EOC reviews of all plants. An EOC Summary meeting was conducted on February 20, 2002, to summarize the results of all regional EOC reviews for the Director of NRR and other members of the Executive Team for those plants that met the criteria specified in IMC 0305. The results of the EOC reviews were issued via annual assessment letters for all plants on March 4, 2002.
- c. NRR staff conducted another of a continuing series of public meetings on February 28, 2002, with the NRC/industry working group on the ROP. The key issues discussed included: NEI 99-02, "Regulatory Assessment Performance Indicator Guideline," Revision 2; safety system unavailability performance indicator (PI) policy issues; status update of NRC ROP web page changes; recent planned changes to Inspection Manual Chapters and inspection procedures; time line meeting status of the safety system unavailability (SSU) PI pilot program; radiation protection significance determination process (SDP); draft steam generator tube degradation SDP; and frequently asked questions. In addition, the working group discussed the recent decision to not adopt changes to the scram PI. The proposed changes had been pilot tested in 2001. The working group discussed the feasibility and desirability of reconsidering previous options and any alternative approaches that could be pursued. Based on the discussions, the

working group did not identify any potential replacement candidates that could potentially address the concerns by the industry. The working group agreed to address this issue further as an agenda item at the May 2002 meeting. The next industry and staff ROP public meeting is scheduled for March 21, 2002.

III. Status of Issues in the Reactor Generic Issue Program

GSI Number: 172

TITLE: Multiple System Responses Program

STATUS: This issue was identified to address 21 potential safety concerns that were raised by the Advisory Committee on Reactor Safeguards (ACRS) during the resolution of Unresolved Safety Issues A-17, "Systems Interactions in Nuclear Power Plants;" A-46, "Seismic Qualification of Equipment in Operating Plants;" and A-47, "Safety Implications of Control Systems." In resolving the issue, the staff developed guidance for the review of the safety concerns of GSI-172 in the Individual Plant Examination (IPE) and the Individual Plant Examination of External Events (IPEEE) programs. In the review of licensee submittals in response to the IPE and IPEEE, no significant contributor to core damage frequency (CDF) was identified. Therefore, the staff concluded that no new or revised licensee requirements were warranted. Thus, the issue was closed on January 22, 2002, and will no longer be tracked.

IV. Licensing Actions and Other Licensing Tasks

Licensing actions are defined as requests for: license amendments; exemptions from regulations; relief from inspection or surveillance requirements; topical reports submitted on a plant-specific basis; notices of enforcement discretion; or other licensee requests requiring NRC review and approval before they can be implemented by the licensee. The FY 2002 NRC Performance Plan incorporates three output measures related to licensing actions. These are: the number of licensing action completions per year; the age of the licensing action inventory; and the size of licensing action inventory.

The FY 2002 NRC Performance Plan incorporates one output measure related to other licensing tasks. Other licensing tasks are defined as: licensee responses to NRC requests for information through generic letters or bulletins; NRC responses to 2.206 petitions; NRC review of licensee topical reports; NRR responses to regional requests for assistance; NRC review of licensee 10 CFR 50.59 analyses and FSAR updates; or other licensee requests not requiring NRC review and approval before they can be implemented by the licensee.

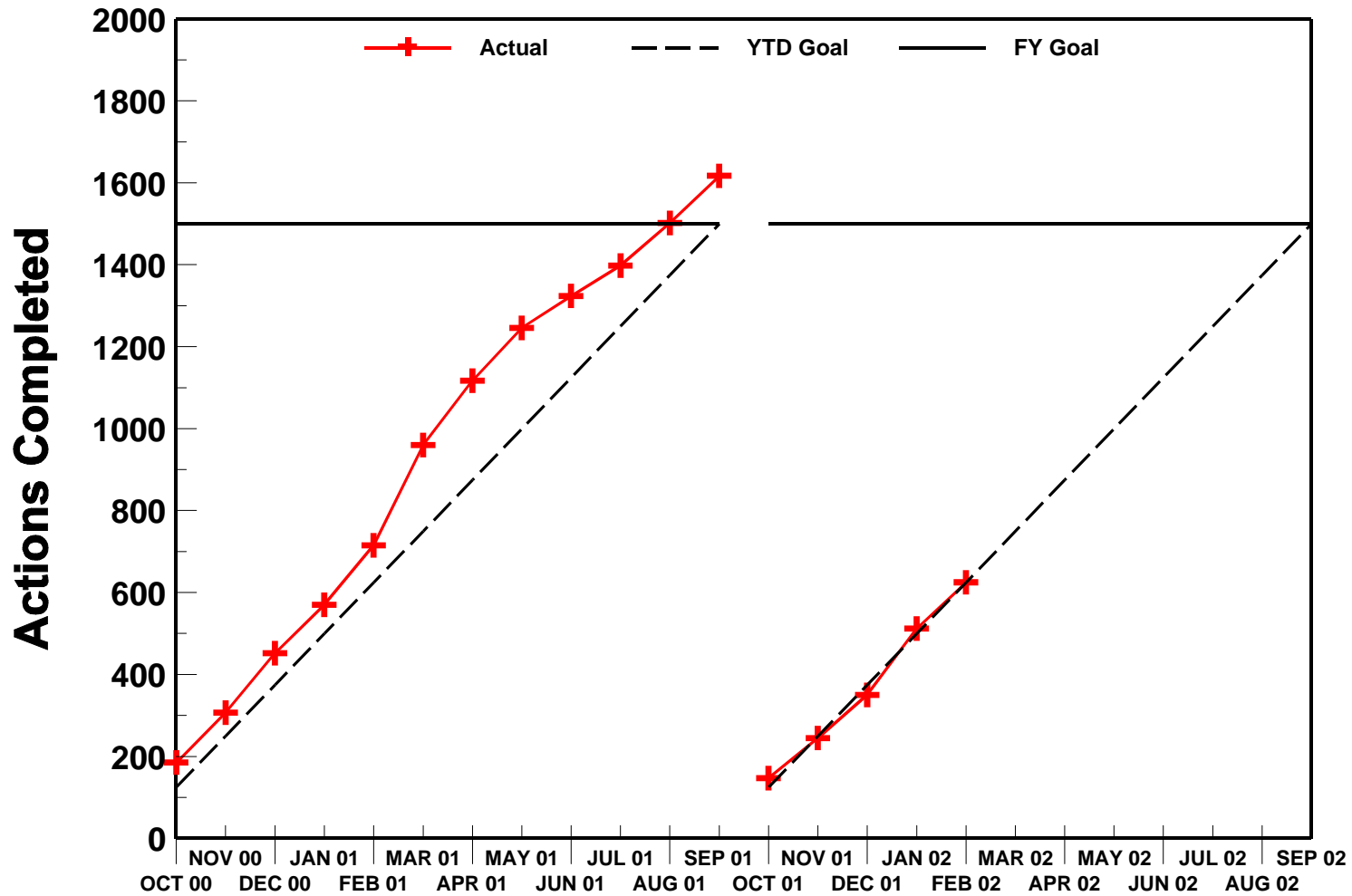
In January 2002, the goal for the size of the licensing action inventory was restored to the Performance Plan and the goal for the percent of licensing action inventory less than or equal to 1 year old was increased from 95% to 96%. The actual FY 2000 and FY 2001 results, the FY 2002 goals and the actual FY 2002 results, as of February 28, 2002, for the four NRC Performance Plan output measures for licensing actions and other licensing tasks are shown in the following table.

PERFORMANCE PLAN				
Output Measure	FY 2000 Actual	FY 2001 Actual	FY 2002 Goals	FY 2002 Actual (thru 02/28/2002)
Licensing actions completed/year	1574	1617	\$ 1500	625
Age of licensing action inventory	98.3% # 1 year; 100% # 2 years	96.9%# 1 year; 100% # 2 years	96% # 1 year; 100% # 2 years	94% # 1 year; 100% # 2 years
Size of licensing action inventory	962	877	1000	906
Other licensing tasks completed/year	1100	523	\$ 350	184

The following charts demonstrate NRC's FY 2002 trends for the four licensing action and other licensing task output measure goals.

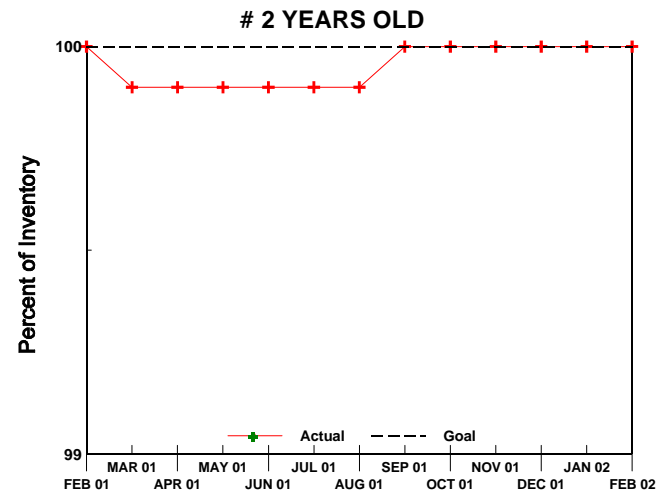
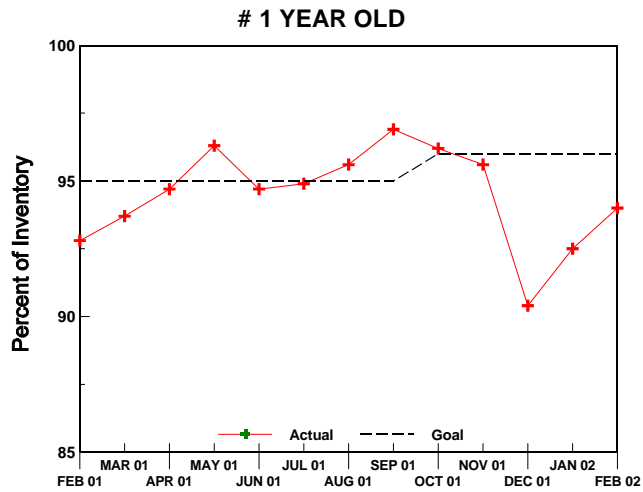
Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Completed Licensing Actions



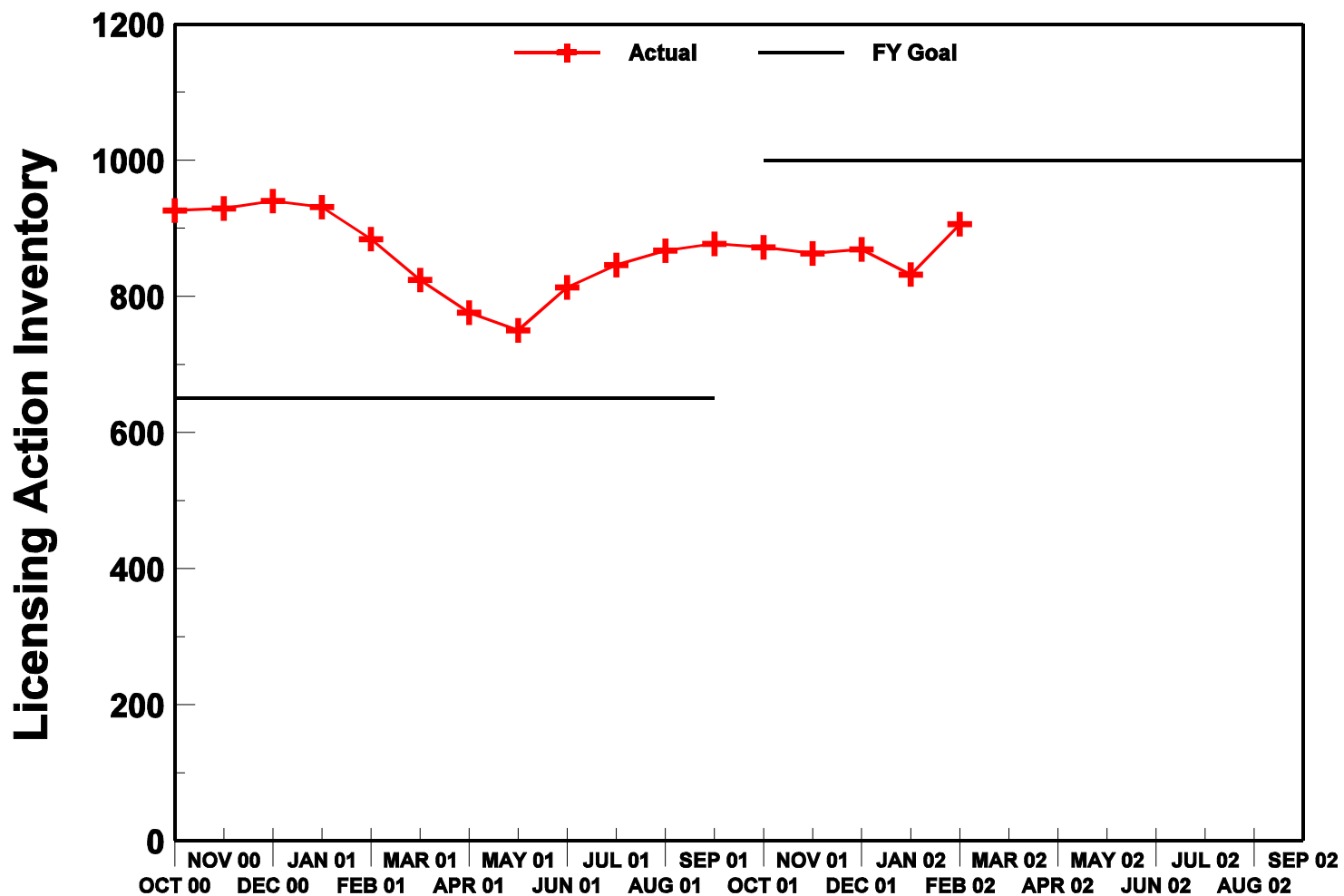
Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Age of Licensing Action Inventory



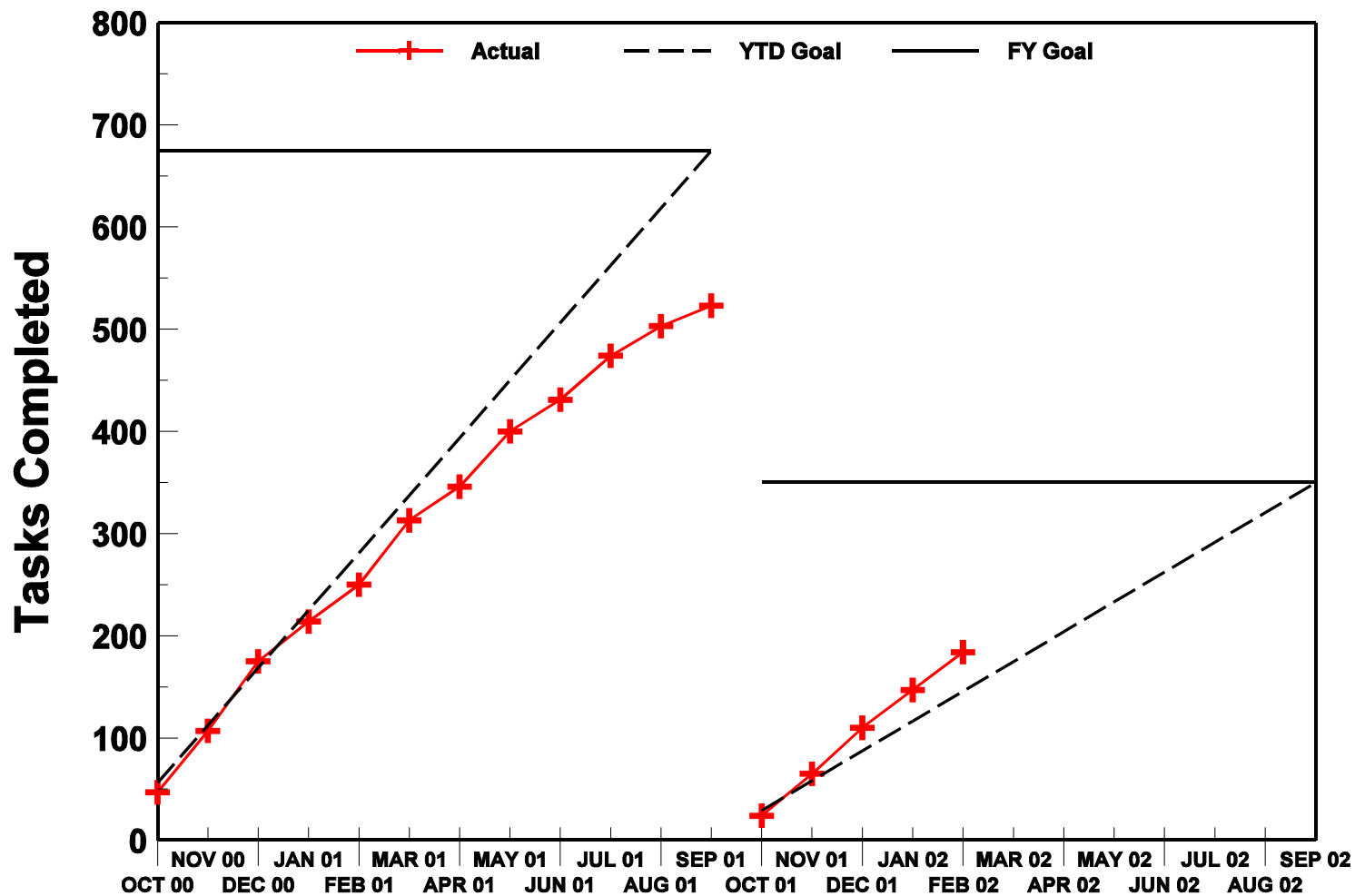
Nuclear Reactor Safety - Reactor Licensing

Performance Plan: Size of Licensing Action Inventory



Nuclear Reactor Safety - Reactor Licensing

Performance Plan Target: Completed Other Licensing Tasks



V. Status of License Renewal Activities

Turkey Point, Units 3 and 4, Renewal Application

The staff issued the completed safety evaluation report in February 2002. The final supplemental environmental impact statement was issued in January 2002. The Commission decision on issuing the renewed license is scheduled for July 2002.

Surry, Units 1 and 2, and North Anna, Units 1 and 2, Combined Renewal Applications

Responses to environmental requests for additional information were received in December 2001 and responses to the safety requests were received in February 2002. The staff is currently preparing the draft supplemental environmental impact statement and the safety evaluation report identifying any open items.

McGuire, Units 1 and 2, and Catawba, Units 1 and 2, Combined Renewal Applications

The McGuire and Catawba renewal applications are currently under review. Responses to environmental requests for additional information were received on January 31, 2002 and February 8, 2002 for McGuire and Catawba, respectively. Responses to the safety requests for information are due by April 2002.

Two petitions were received requesting a hearing on the renewal of the McGuire and Catawba licenses and by Commission Order, an Atomic Safety and Licensing Board (ASLB) was established. In a Memorandum and Order issued January 24, 2002, the Atomic Safety and Licensing Board Panel issued its ruling on standing and the admissibility of contentions finding that both petitioners have standing and concluded that admissible contentions have been proffered. One question relating to terrorism risks was referred to the Commission for its consideration.

On February 4, 2002, the NRC staff and the licensee filed appeals with the Commission regarding the ASLB's admission of contentions concerning the possible use of MOX fuel in the facilities as well as the completeness of the licensee's evaluation of severe accident mitigation alternatives. On February 6, 2002, the Commission accepted certification of the terrorism issues and set a briefing schedule.

Peach Bottom, Units 2 and 3, Renewal Application

The Peach Bottom renewal application is currently under review. Responses to the environmental requests for additional information were received in January 2002. Safety requests for additional information are scheduled to be issued by March 2002.

St. Lucie, Units 1 and 2, Renewal Application

The St. Lucie renewal application is currently under review and the staff is preparing requests for additional information. All environmental requests for additional information are scheduled to be issued by May 2002 and the safety requests by July 2002. The environmental review and

scoping process has begun and a public meeting is scheduled in the vicinity of St. Lucie on April 3, 2002.

Fort Calhoun Renewal Application

On January 11, 2002, the NRC received an application for renewal of the Fort Calhoun operating license. The staff is currently performing the required acceptance review and, if found acceptable, will docket the application, notice an opportunity for hearing, and issue the review schedule.

VI. Status of Review of Private Fuel Storage, Limited Liability Corporation's Application for a License to Operate an Independent Spent Fuel Storage Installation on the Reservation of the Skull Valley Band of Goshute Indians

The Nuclear Regulatory Commission (NRC) staff has completed its safety evaluation of the application by Private Fuel Storage, Limited Liability Company (PFS) for a license to construct and operate an away-from-reactor independent spent fuel storage installation on the Reservation of the Skull Valley Band of Goshute Indians. The NRC staff, with the Department of Interior's Bureau of Land Management and Bureau of Indian Affairs, and the U.S. Surface Transportation Board, has also completed a Final Environmental Impact Statement for the project. However, before a recommendation can be made by the NRC staff to the Commission regarding whether a license should be granted to PFS, litigation in the adjudicatory proceeding before the Atomic Safety and Licensing Board (ASLB) must be concluded and the ASLB must issue its findings.

The litigation in the adjudicatory proceeding on PFS's license application continued during this reporting period: (1) the staff responded to various discovery requests submitted by the State of Utah and another Intervenor; (2) PFS, the State of Utah and the staff filed their testimony on aircraft crash issues; (3) the State of Utah filed an additional contention on the environmental cost-benefit balancing; (4) the Licensing Board denied in part, and granted in part, PFS's motion for summary disposition of Ohngo Gaudadeh Devia's contention regarding environmental justice; (5) the Licensing Board admitted a unified contention on all seismic/geotechnical issues; (6) the Licensing Board admitted a new contention based on the State of Utah's enactment of legislation that would prohibit the Tooele County Sheriff's office from serving as the designated local law enforcement agency for the PFS facility, and (7) the Board established the schedule for hearings on various issues in April and May 2002.

On February 6, 2002, the Commission accepted the ASLB's referral of its decision denying the admission of a contention relating to terrorism and set a briefing schedule.

VII. Enforcement Process and Summary of Reactor Enforcement by Region

Reactor Enforcement by Region

*Numbers of violations are based on enforcement action tracking system (EATS) data that may be subject to minor changes following verification. The number of Severity Level I, II, III listed refers to the number of Severity Level I, II, III violations or problems. The monthly totals generally lag by 30 days due to inspection report and enforcement development.

** Violation totals for Regions II & IV reflect a shift from a 6 week inspection period to a quarterly inspection period.

*** Corrected data due to a reporting error in November 2001.

		Reactor Enforcement Actions*				
		Region I	Region II**	Region III	Region IV**	TOTAL
Severity Level I	Jan 2002	0	0	0	0	0
	FY 2002 YTD	0	0	0	0	0
	FY 01 Total	0	0	0	0	0
	FY 00 Total	0	0	0	0	0
Severity Level II	Jan 2002	0	0	0	0	0
	FY 2002 YTD	0	0	0	0	0
	FY 01 Total	0	1	0	0	1
	FY 00 Total	1	2	0	0	3
Severity Level III	Jan 2002	0	0	0	0	0
	FY 2002 YTD	2	0	0	0	2
	FY 01 Total	1	1	1	1	4
	FY 00 Total	5	0	4	4	13
Severity Level IV	Jan 2002	0	0	0	0	0
	FY 2002 YTD	0	0	2	0	2
	FY 01 Total	1	0	2	1	4
	FY 00 Total	4	1	3	5	13

Reactor Enforcement Actions*						
Non-Cited Severity Level IV & Green	Jan 2002	30	26	22	24	102
	FY 2002 YTD	98***	64	78	60	300***
	FY 01 Total	279	105	201	139	724
	FY 00 Total	313	190	289	258	1050

Escalated Reactor Enforcement Actions Associated with the Revised Reactor Oversight Process						
		Region I	Region II	Region III	Region IV	Total
NOVs related to white, yellow or red findings	Jan 2002 -Red	0	0	0	0	0
	-Yellow	0	0	0	0	0
	-White	1	0	0	0	1
	FY 2002 YTD	1	1	1	1	4
	FY 01 Total	8	4	4	3	19
	FY 00 Total	6	1	0	0	7

Description of Significant Actions taken in January 2002

Exelon Generation Company, LLC (Limerick Unit 2) EA-01-293

On January 11, 2002, a Notice of Violation was issued for a violation associated with a white SDP finding involving a Safety Relief Valve (SRV). The violation cited the failure to establish adequate measures to identify that the SRV was in a degraded condition and was vulnerable to a failure to re-close after lifting.

VIII. Power Reactor Security Regulations

In response to the terrorist attacks on September 11, 2001, the NRC and the nuclear industry have taken a number of actions to ensure the security at nuclear power plants. Immediately following the terrorist attacks on the World Trade Center and the Pentagon, the NRC advised nuclear power plant licensees to go to the highest level of security (i.e., Level 3), and all promptly did so. The Nation's nuclear power plants remain at the highest level of security and the NRC continues to monitor the situation.

For the longer term, the Chairman, with the full support of the Commission, has directed the NRC staff to thoroughly reevaluate the NRC safeguards and security programs. This reevaluation will be a top-to-bottom analysis involving all aspects of the agency's safeguards and security programs. The NRC staff submitted a report to the Commission that outlined the proposed course of action and schedule for conducting the review and identified preliminary policy issues for Commission consideration.

Given the nature of the attacks on September 11, the identification of any necessary adjustments to the safeguards and security measures for civilians must involve other U.S. national security organizations. The NRC is currently interacting with the FBI, other intelligence and law enforcement agencies, the Department of Defense, and the recently established Office of Homeland Security to ensure that all pertinent input from relevant U.S. agencies is considered before any changes are made to the NRC's programs.

IX. Power Uprates

The staff has assigned power uprate license amendment reviews a high priority. The staff considers power uprate applications among the most significant licensing actions and is, therefore, conducting power uprate reviews on accelerated schedules.

Licensees have been applying for and implementing power uprates since the 1970s as a way to increase the power output of their plants. The staff has been conducting power uprate reviews since then and to date, has completed 72 such reviews. Approximately 9800 MWt (3250 MWe) or an equivalent of about three nuclear power plant units has been gained through implementation of power uprates at existing plants. The staff currently has 12 plant-specific applications and two General Electric Nuclear Energy topical reports for power uprates under review.

The staff conducted a survey in January 2002 to obtain information regarding industry's plans related to power uprate applications. The survey requested information for planned power uprates over the next 5 years. Based on this survey, licensees plan to submit 38 additional power uprate applications in the next 5 years. These include 23 measurement uncertainty recapture power uprates (i.e., power uprates less than 2 percent), and 14 extended power uprates (i.e., power uprates greater than about 7 percent). One licensee did not report a magnitude for its planned power uprate. Planned power uprates are expected to result in an increase of about 4777 MWt (1590 MWe) (equivalent to more than one large nuclear power plant unit). Licensees also indicated that they are currently studying the feasibility of power uprates for eight other units. In addition, the staff expects significant interest by pressurized water reactor licensees in large power uprates as a result of ongoing work by pressurized water reactor vendors. The staff will utilize this information for future planning.