UNITED STATES OF AMERICA U.S. NUCLEAR REGULATORY COMMISSION

BRIEFING ON LICENSE RENEWAL FOR RESEARCH AND TEST REACTORS

MARCH 27, 2012

9:00 A.M.

TRANSCRIPT OF PROCEEDINGS

Public Meeting

Before the U.S. Nuclear Regulatory Commission:

Gregory B. Jaczko, Chairman

Kristine L. Svinicki, Commissioner

George Apostolakis, Commissioner

William D. Magwood, IV, Commissioner

William C. Ostendorff, Commissioner

APPEARANCES

NRC Staff:

Bill Borchardt Executive Director for Operation

Tim McGinty
Director, Division of Policy and Rulemaking
NRR

Jessie Quichocho Branch Chief, RTR Licensing Branch NRR

John Adams Senior Level Advisor for RTR NRR

External Panel:

Leo Bobek Chair, National Organization of Test, Research, and Training Reactors, Inc. (TRTR) Reactor Supervisor, Nuclear Radiation Laboratory University of Massachusetts Lowell

Facility with LR Review Completed:

Sastry Sreepada Director, Walthousen Reactor Critical Facility Rensselaer Polytechnic Institute

Facilities with LR Review In-Process:

Ralph Butler Director, University of Missouri Research Reactor (MURR)

Stephen Miller Armed Forces Radiobiology Research Institute (AFFRI)

PROCEEDINGS

CHAIRMAN JACZKO: Well, good morning everyone. We're
meeting today to talk about the license renewal status of research and test and
training reactors. This is certainly an important issue as we work to deal with
what has been a significant backlog of renewals for the RTR licenses. I'm
certainly pleased to hear that we've made considerable progress in this area
since, I think, the last meeting we had in August of 2009, so almost three years
ago. As I look at today's meeting agenda, I think this will be an important
opportunity to strengthen the continued partnership we've had with the research
and test reactor community in looking at the issues of license renewal.
I think, though, there will still be challenges on both sides, as we
work through this backlog, but I'm certainly very interested in hearing about the
progress that we've made in the last two years. I think this has certainly been a
very good work on the part of the staff to get us in a place where we can more
faithfully process these applications, and I think there's been a lot of work on the
part of the RTR community, as well, to improve the applications and improve the
submittals, and improve the timeliness. So I look forward to hearing about the
progress, and what the challenges are, and offer my colleagues, any comments
they'd like to make? Okay, Bill, you want to start?
BILL BORCHARDT: Good morning, thank you. NRC staff
recognizes that research test reactors are a very important and valuable national
asset. These facilities have relatively low risk, and they have a timely renewal
provision. Those two factors, in my mind, have allowed us to utilize some
resources, and make budget decisions over the years, that have, at times, staffed
the renewal process with fewer resources than we might have preferred to do,

and it seems that a number of events have happened over the years: Three Mile 2 Island, the 9/11 event, now Fukushima, at unique time periods which made the

3 problem even a little bit worse.

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Having said that, well that created a backlog in renewals, but having said that, we have maintained the appropriate level of regulatory oversight of these facilities throughout that time period, and are quite comfortable with the decisions that had been made. Today in the staff briefing you're going to hear some very good news that the staff's worked very hard, and made significant improvements in that backlog, making significant process improvements as well, which will ensure that the backlog continues to work down, and that the new renewals coming in will be worked on a timely process. So with that introduction, we'll turn it over to Tim.

TIM MCGINTY: Thanks Bill. I'd like to start today's presentation by providing background information on the organizational structure for research and test reactors, and a description of the license renewal backlog and its causes. Following me, Mr. Jessie Quichocho will provide you information on current status, and our activities to complete the license renewal backlog, and following Jessie Mr. John Adams will provide information on the current status and activities underway to address streamlining of the research and test reactors license renewal process. Next slide please.

There are currently 30 research reactors and one test reactor licensed by NRC to operate. Two branches in the Office of Nuclear Reactor Regulations Division of Policy and Rulemaking share responsibility for the licensing and oversight regulatory functions. The research and test reactor licensing branch is responsible for licensing actions, including the renewal of 1 operating licenses. Of these 31 facilities, license renewal applications for 19

2 facilities comprised the license renewal backlog at the beginning of fiscal year

3 2009.

So regarding the causes for the license renewal backlog, the foremost cause, as Bill mentioned, was the deferral of license renewal work due to emergent issues. The majority of our RTRs were originally licensed for 20 years in the late 1950s and early 1960s. The majority of the RTR licenses expired in the period of time that coincided with the years following the accident at Three Mile Island. Work to review those renewal applications was deferred to focus on post-Three Mile Island priorities. The option to defer license renewal was acceptable, given the minimal operational impact on the research reactor licensees due to the timely renewal provision of 10 CFR 2.109 alpha that Bill mentioned. That provision allows for continued operation of an RTR during the review of the renewal application, as long as the applicant has provided an acceptable application at least 30 days before the expiration of the license.

The resulting backlog was eventually eliminated, but did result a concentration of renewed license expiration dates. Jumping ahead 20 years, the majority of the research and test reactor licenses were once again expiring in the early to mid 2000s. This time the license expirations coincided with the post-9/11 security initiatives, and license renewal work was again deferred creating the current backlog.

Considering the circumstances, the deferral option was again deemed acceptable considering the timely renewal provision. Other factors have also contributed to the license renewal backlog. Early in fiscal year 2005 staffing levels were recognized as inadequate to address the current and expected

1 emerging license renewal backlog. The staff communicated these workload and

2 human capital issues to the Commission in SECY-05-0062. The Commission

3 responded by providing resources requested by the staff, however, the staff was

not able to complete the license renewals at the rate estimated in the SECY

5 paper. There were two reasons for this.

First, the rate at which the staff was able to fill these new positions was much slower than estimated. Second, the conversion of research reactor high enriched uranium fuel to low enriched uranium fuel became a higher priority, demanding the attention of staff that would otherwise be reviewing license renewal applications. The resources necessary to complete the HEU to LEU conversions was not considered in the original staffing estimate. Next slide please.

The staff also did not accurately estimate the availability of the licensee's resources in the development of backlog elimination estimates and schedules. This miscalculation contributed to significant delays to the estimated backlog completion schedule. Many RTRs have limited staff and resources available for licensing. Licensee's staff at small research reactor facility can be as small as one part-time employee. In most cases, the staff that performs licensing functions do so in addition to their many other organizational responsibilities.

Licensing infrastructure and guidance changes has also negatively impacted the rate of completing license renewal application reviews. The last time that many of the licenses in the current backlog came due for renewal was in the 1980s. In the period between the first and second license renewal applications, a number of changes occurred. Most notably was the development

- 1 of NUREG 1537, Guidelines for Preparing or Reviewing Applications for the
- 2 Licensing of Non-power Reactors. NUREG 1537, issued in 1996, describes the
- 3 necessary content for research and test reactor safety analysis reports.
- 4 Guidance and staff review process preceding the issuance of NUREG 1537 were
- 5 informal, which resulted in varying levels of completeness and consistency.
- 6 Many of the safety analysis reports submitted with the most recent renewal
- 7 applications did not include the necessary information. This resulted in a need
- 8 for staff to issue numerous requests for additional information to licensees,
- 9 further burdening their limited resources and requiring additional reviews by NRC
- 10 staff, and thus resulting in additional delays. Next slide please.

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So on October 24th, 2008, staff provided options and recommendations to the Commission in SECY-08-0161 for the elimination of the current license renewal backlog. In the subsequent Staff Requirements Memorandum, the Commission directed the staff, in conjunction with the research and test reactor community and the public stakeholders to develop both short-term and long-term approaches to address the backlog. Direction for the short-term approach specified the development of an interim, streamlined process to focus reviews on the most safety significant aspects of the license renewal application, and to apply a graded approach.

In October, 2009, the staff issued interim staff guidance on the streamlined review process, to address the backlog in the short term. Based on the feedback from the RTR community, and the NRC project managers, the use of a graded approach within the interim streamlined review process has resulted in fewer requests for additional information, more effective licensee responses to issues, and has significantly reduced the level of effort to submit and review a

1 license renewal application.

Concurrent with the short-term activities, the staff commenced work to develop a more efficient, streamlined process for the long term. The staff acquired contractor support for the development of a regulatory basis that will determine if rulemaking is an appropriate remedy to streamline the license renewal process. John Adams will be addressing this issue in more detail during his presentation.

At this point, I'm going to ask Mr. Quichocho to present his information on the status and activities underway to address these challenges, and to complete the remaining license renewal reviews in the backlog.

JESSIE QUICHOCHO: Thank you, Mr. McGinty. I will be discussing activities that the staff performed to reduce the backlog as well as current status of license renewal activities. Since 2005, the staff hired at a slow pace due to the limited availability of specialized and highly valued skill sets, and research from test reactor experience. The staff aggressively pursued alternative means of hiring, and incentives to attract experienced and highly qualified candidates. In 2010, we were successful to fully staff the research and test reactor branch to accomplish the work at budgeted levels. With sufficient staff on hand to maintain project management activities for the renewals, and address emergent and higher priority activities, the staff developed a path for success: a streamlined review process for license renewal. The staff developed an interim staff guidance that focused on the most safety significant aspects of the license application, and applied a graded approach for those facilities less than two megawatt thermal.

Following multiple interaction and input from external stakeholders,

and the research and test reactor community, the staff issued the interim staff guidance in 2009. This focused review process has been a key contributor to the significant completion of many research and test reactor license renewal reviews to date. With the streamlined review process in place, the staff initiated Lessons Learned activities after the first few applications were completed. The staff identified flexibilities in the review based on varying degrees of designs, technical issues, and operational characteristics of research and test reactors, and applied them to subsequent applications resulting in a more efficient and effective process.

To address the limited resources, to perform thermohydraulic and neutronic analyses that licensee's face with their license renewal applications, the staff coordinated efforts with the Department of Energy, Office of Nuclear Energy Science and Technology, in identifying technical expertise to assist licensees. To date, five facilities have received Department of Energy assistance. The availability of these resources has proven indispensable in contributing to the reduction of the research and test reactor backlog. The staff and research and test reactor community continue to work together throughout the license renewal process. Next slide please.

The staff reinforced the Commission direction to continuously interact with external stakeholders in all NRC regulatory activities. During the license renewal process, the staff increased face to face meetings and developed, and sustained, a dialogue for those facilities that require additional time to respond to requests for additional information due to limited technical and budget resources. The staff had many discussions with licensees, who seldom used the licensing process during their 20 year license period. By performing

these activities, the project managers developed a more detailed understanding of their assigned facilities, and licensees developed an understanding of the NRC

licensing process and requirements.

The staff recognizes that early communication of regulatory activities specific to the research and test reactor community is important to the development of informed regulation, and improves the licensee's understanding during implementation. The staff implemented initiatives to encourage the research and test reactor community to engage and provide feedback to the development and implementation of our regulatory process. Some of these initiatives include quarterly meetings with the National Organization of Test Research and Training Reactor Executive Committee, substantial participation of the staff and the annual and National Organization of Test Reactor and Training Reactor Conference, and quarterly newsletters called "The Nonpower Reactor Explorer."

The staff has routinely engaged in the research and test reactor community through public meetings on topics such as developing interim staff guidance for a digital instrumentation and control standard review plan, developing a regulatory basis for a streamlined license renewal process, and most recently, the development of staff's interpretation for 50.54(m). These types of engagements are more than information sharing. It sets the tone and standard we would like to achieve to develop good working relationships and a more collaborative working environment with the research and test reactor community.

Perhaps the most significant program accomplishment of the research and test reactor community and the NRC, after undergoing license renewal, is the mutual thorough understanding of the design and operational

1 characteristics of licensed facilities, having a documented safety analysis from

2 which to base future licensing actions, and clarity of the technical specifications

and bases. So you may ask where are we today with research and test reactor

license renewal backlog? Next slide please.

Here is a graphical representation of the number of renewals in the inventory by fiscal year. Since the implementation of the streamlined review process in 2009, and after reaching a fully augmented staff in 2010, the staff reduced the backlog from 19 in 2008 to 9 today. This graph also illustrates the amount of effort the licensee's have contributed in working with the NRC staff to reduce the backlog. Recognizing the challenges faced by the NRC staff and licensees, the staff, in applying the streamlined review process with its current staffing levels and improved licensee interactions, expects to complete all renewals in the backlog by fiscal year 2014.

The staff continues to work on license renewals not part of the backlog. For those we have received, the staff has taken a more interactive role in engaging with the licensees early and prior to the renewal date. Approximately 12 months prior to a license renewal is expired, the staff sends a letter to the facility describing the required contents of renewal -- of a renewal application, including information used by other similar facilities as examples. The staff conducts a follow up site visit a month of so afterward.

With early engagement and communication, the licensees are provided the opportunity to interact with the staff during the development of their safety analyses that may help minimize requests for additional information during the review period. The staff is working on four license renewals not part of the backlog. Of those, one is expected to be completed this fiscal year. I will now

ask Mr. John Adams to discuss the long-term plan.

JOHN ADAMS: Good morning. On March 26, 2009, in response to SECY-08-0161, the Commission directed the staff to develop and submit a long-term plan for an enhanced research and test reactor license renewal process for Commission review. The plan was to include the development of a basis for redefining the scope of the process as well as a recommendation regarding the need for rulemaking and guidance development. On June 24th, 2009 the staff provided that plan to the Commission in enclosure two of SECY-09-0095. Since that time, a contract has been awarded for assistance in the development of a regulatory basis. The contractor commenced work on the regulatory basis on April 11th, 2011. Next slide please.

Four major tasks related to the development of the regulatory basis have been completed to date. The contractor has completed their analysis of the current regulatory requirements related to research and test reactor license renewal, conducted two public meetings to solicit input related to the research and test reactor license renewal from public stakeholders and from the research and test reactor community. They've completed an analysis of the potential efficiencies that could result from the segregation of regulations applicable to research and test reactors from regulations that apply only to power reactors, and they've completed the analysis of license renewal methodologies used by other agencies such as the Department of Energy, and the Department of Defense, and by other organizations such as the International Atomic Energy Agency. Next slide please.

The NRC staff is currently reviewing the contractor's results and recommendations from those completed tasks, and has recently commenced an

- 1 early review of a first draft of the regulatory basis. The staff will soon begin
- 2 preparation for a third public meeting to solicit input from public stakeholders and
- 3 the research and test reactor community on the draft regulatory basis. Following
- 4 the public meeting, the staff will commence work on the final regulatory basis.
- 5 Regulatory basis completion is anticipated in the fourth quarter of fiscal year '12.
- 6 Next slide please.

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All proposed rulemakings were assessed using the agency's common prioritization of rulemaking process to assign each proposed rulemaking a priority. Based on the process criteria, the research and test reactor license renewal streamlining rule was ranked as a medium priority. Faced with a significant number of high priority rulemakings in fiscal year '13, the Office of Nuclear Reactor Regulation decided not to work medium priority rules in fiscal year '13. Should the regulatory basis provide the necessary justification to conclude that rulemaking is an appropriate remedy to streamline the research and test reactor license renewal process; the staff will request the necessary resources through the budgeting process in fiscal year '14 to develop the proposed and final rules. Concurrent with the development of the proposed rule, the staff will update existing implementation guidance where appropriate, and develop new implementation guidance where necessary. I would now like to turn the presentation back to Mr. McGinty who will provide a brief summary of key points from today's presentation.

TIM MCGINTY: Okay, a brief summary. First, the backlog of renewal applications resulted from an NRC focus on emergent high priority issues. Second, the staff implemented processes to address the backlog of RTR applications as directed by the Commission, proactively engaged external

- 1 stakeholders throughout the regulatory process, and reduced a significant
- 2 number of license renewal applications in the backlog.
- Finally, the staff is actively working on a long-term solution to
- 4 streamline the research and test reactor license renewal process. This
- 5 concludes our prepared remarks, we look forward to your questions.
- 6 CHAIRMAN JACZKO: Thank you for that presentation, we'll start
- 7 with Commissioner Magwood.
- 8 COMMISSIONER MAGWOOD: Morning, first let me thank the staff
- 9 for the work they've done over the last several years to pursue this. The
- 10 research reactor community is different, as you indicated in your opening
- 11 remarks, than many of our other licensees, and I wanted to explore some of that
- with you. One aspect, and I think that Tim you focused on this, of the research
- reactor community is the fact that staffs are relatively small for the most part, in
- some cases it's one person or maybe even less than one person. Can you talk in
- 15 some -- give us a little bit more specificity, how the staff deals with that situation?
- 16 I think the staff is much more used to having a licensee relationship where the
- 17 NRC asks a question, and the licensee uses whatever resources necessary to
- get the question answered in a very timely fashion. In this case, you may have,
- 19 you know, half a professor who's responsible for the research reactor who's off
- 20 grading papers somewhere, and you're asking these thermohydraulic questions.
- 21 How does the staff deal with that?
- 22 BILL BORCHARDT: Before I ask Tim to answer that, I'm -- just
- provide some background on how the NRC's adapted to that, and a long time
- ago, 20-plus years ago, research and test reactors were treated like a power
- 25 reactor. They were -- the work was accomplished through a matrix kind of

review, and we made the mistake, which we learned a lesson, of treating them just like a power reactor, and the kind of issues that you raised really caused a lot of problems. I think it was back in the mid-90's we made the decision to have a self-contained -- more or less self-contained organization within NRR that would be the point of contact for all RTRs, and since that time, there's been an increased learning from the NRC's staff on how to interest, so I'll lot Tim talk

increased learning from the NRC's staff on how to interact, so I'll let Tim talk about the specific of how we accommodate the resources.

TIM MCGINTY: There are a number of different ways. One of the most significant, of course, is the availability of resources to focus on individual licensees, so this organization was relatively small up until, as we discussed during the presentation, there was on the order of three or four licensing project managers in the organization historically, but with the HEU and LEU work, as well as with the renewal backlog, and now on the horizon utilizing the same staff for the Moly-99 which will be the subject of a future Commission meeting, we've staffed up significantly and that really has enabled us to leverage our ability to personally interact with the RTR community on a much more proactive basis. Then I'd like to point to Jessie because we've talked about this, there's about three or four different avenues that we'd like to call your attention for this particular element.

JESSIE QUICHOCHO: Yes, thank you. There's -- more specifically I discussed dialogue. It's important to create this dialogue with each facility. The importance of that so that we understand, like you said, if they have minimal staffing efforts, or if they're limited budgeted resource; and so creating that dialogue early on, and understanding the facility provides us with, now, an understanding of -- or how do we mitigate -- what strategies do we put in place to

mitigate the impacts to the facility as well as to the NRC.

So what we did was -- some of the things that we have performed, we communicate with the facilities to identify those questions, a request for additional information that are least complex, and focus on certain ones to be accomplished in a given period. That period, defined by the licensee, as you and Bill alluded to was that before in power reactor space, we were so used to the response within 30 days within a short timeframe. Here, in the way we interact with our licensees is, we ask the licensee how long would it take you to provide that information, and have them give us that timeframe.

COMMISSIONER MAGWOOD: When -- and I appreciate that, and as you -- give you a chance to sort of give us a little bit more of a -- maybe I'll ask Bill to do this, but more of a general view of how this should be viewed by the public because in the case of a commercial reactor, for example, the long drawn out process driven by the willingness or the ability of the licensee to respond to questions would probably raise issues with the public. In the case of these facilities, it doesn't, and I wonder if maybe Bill or others could just give a general, more or less, for the record explanations as to why that's not a safety concern to let these -- because some of these reactor processes have been going on for almost a decade.

BILL BORCHARDT: In my mind, I think there's two factors. One is that these facilities have significantly less risk in their operation than power reactors, so from that perspective we're being risk informed. The second is that it doesn't impact our ongoing and continuous oversight inspections and interactions between the staff and the licensee, so that continues regardless of whether or not the license renewal has been completed, so there's ongoing active engagement,

so we're confident in the safe operation of the facilities.

TIM MCGINTY: I would add that it's just become quite apparent to me, personally, that developing the relationship between the licensing staff at the facility and our staff, and the process of having gone through in the past decade of upgrading the applications to be in conformance with the SRP. It's a mutual learning by both the facility as well as the NRC staff. And in this particular case, in an area of limited resource where the resources -- where the risk is not particularly high, that's the opportunity to leverage a mutual understanding. And that's what we try to do with the community.

appreciated John's discussion of the process leaning towards the regulatory basis for potential rulemaking in the future. Can you -- and in your remarks you didn't really give a -- I guess I'd say didn't really -- I didn't hear you making a strong case that this was something we need to do. It was more a response to a Commission direction, but what is the case to go forward with the rulemaking. I realize the staff hasn't gone down this path yet well enough to make a recommendation, but I just wanted to get your personal views on this. You know, we'll have gone through most of these renewals. Obviously, someone will come back around in the future if some of the reactors continue in operation, but what is really the value at this stage of the game in implementing a new rule, and just getting your opinion after looking at this for some time.

JOHN ADAMS: Well, right now, as you know, we're in the regulatory basis development stage. And the purpose of the regulatory basis is to determine if rulemaking is the appropriate remedy. We have a lot of information through the work that has been done; that work is really just coming

ı	together now in its entirety so we can, as i pointed out, we have a very, very
2	rough draft of the first rough draft of the regulatory basis. Right now, that has
3	more areas that require further assessment and evaluation than has the answers
4	at this point. So that work needs to be completed. I believe there is a good
5	chance that this could show that rulemaking isn't necessary. There are some
6	potential options that such as you could rely possibly more heavily on your
7	inspection program that would, in turn, minimize the impact or the burden of
8	license renewals. You could shift more of the burden to the inspection side from
9	that. And there are probably other methods too that I'm not aware of, but by no
10	means have we reached a conclusion that rulemaking will be the option that we
11	need to pursue. I think the direction that we got in SECY-08-0161 to provide a
12	plan we did have to make an early assessment at that point to provide to the
13	Commission whether we believed that rulemaking could be necessary. And I
14	think that that was our response there, that it was likely that we would have to
15	rely on some level of rulemaking. I hope I've answered your question.
16	TIM MCGINTY: I would only add that that's where the importance

TIM MCGINTY: I would only add that that's where the importance of the stakeholder and the public interaction and the public meetings, that really helps reveal whether there's any sticking points where the rulemaking is the most appropriate outcome.

COMMISSIONER MAGWOOD: I appreciate that. One last quick question, just in summary, do you feel you now have the right balance in the interactions with the research reactor community regarding the intensity of the effort?

TIM MCGINTY: There's more to work on. I think the way you phrased it really enables me to say yes. I think the -- we have the correct

1	balance now, but it's always a work in progress.	There's improvements that we

- 2 can make, and we'll continue to strive to do those, but yes.
- 3 COMMISSIONER MAGWOOD: Okay. Thank you very much.
- 4 Thank you, Chairman.

- 5 CHAIRMAN JACZKO: Commissioner Ostendorff.
- COMMISSIONER OSTENDORFF: Thank you, Mr. Chairman. I'll add my thanks to those of others for the progress you've made in reducing the backlog. I know there's still much more work to be done, but I think the trend looks good.
 - I think I'm going to start out with you, Jessie, in I want to react when -- and then ask you a question on the skill set issue you raised that applies to the people. And I know that this agency has been challenged at times in trying to have the resources to deal with NFPA 805, PRA, et cetera. Can you comment on the top challenges you faced, that your area faced in trying to bring on board people to the NRC staff to deal with the RTR license renewal?

16 JESSIE QUICHOCHO: Sure.

- COMMISSIONER OSTENDORFF: And specifically any particular skill sets you found were either a niche or a unique capability that created some challenges.
- JESSIE QUICHOCHO: I think the niche here is that the skill sets with experience in the research and test reactor background whether the individual was licensed at a facility that performed a health physics operations at a facility, and with nuclear engineering background. It's, you know, as you recognize, we have 31 research reactor facilities. Already we're starting out with a small pool of individuals. It definitely provides breadth and -- to the -- and

1	complements our process in what we do here at the agency. And that's pretty
2	much the complement that we're looking for, is the research and test reactor
3	experience, the operations at a facility. It's very small; that pool of individuals is
4	small.
5	COMMISSIONER OSTENDORFF: So, that most of the people that
6	were hired since 2005 have actual RTR operating experience?
7	JESSIE QUICHOCHO: Yes, the for the folks that are working on
8	the backlog, that's correct.
9	COMMISSIONER OSTENDORFF: Okay. Jessie, I'm going to stick
10	you another question. In your slides, you talked about two public meetings that
11	were I think it was your slides, or were those John's? Okay, John's. Okay.
12	John, let's shift to you. Thank you, Jessie. Can you kindly summarize I'm
13	interested in knowing what kind of feedback you got from the public meetings at
14	a high level.
15	JOHN ADAMS: Actually, the public meetings that have been held,
16	we've gotten considerable, actually more than normal feedback. The TRTR
17	community has been very active in this particular effort. They see that it's, I
18	think, very important for their continued operations of their facilities. And they will
19	be the first to tell you that they believe that we need to enhance this process to
20	make it more efficient. And they've been very forthcoming with
21	recommendations and ideas that have been communicated to us through
22	comments in these public meetings.
23	I think that there will be significant participation, and you'll probably

hear about that in the second half of this meeting, with the upcoming meeting

where we actually put a draft regulatory basis on the table in front of them,

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1	because that's where this is all going to come together and they can see the
2	direction that we are leaning towards with the information that we gained through
3	the regulatory basis development and can start comparing that to their actual
4	operations and see how that impacts them. I find that's when we get the best
5	comments, is when our licensees can compare the impact of a proposed
6	direction to their specific operation. That seems to be where we get the best
7	comments and ideas to inform our process. So, I think there has been good
8	participation on this, and we expect that that will only get better at this point.
9	COMMISSIONER OSTENDORFF: Okay, let me understand this
10	thing a little bit. That same slide, you mentioned also about benchmarking
11	against the Department of Energy, the Department Defense, IAEA, license
12	renewal methodologies can you are there any fundamental or significant
13	differences between how the NRC is approaching this issue compared to those
14	other three organizations?
15	JOHN ADAMS: There are some minor differences. The biggest
16	difference that I've ascertained out of the information that we've received from
17	the contractor on this comparison and contrast has been the amount of public
18	involvement. Our process tends to involve the public much more than the DOE
19	methodologies and the DOD methodologies. So, I'm not sure I really don't
20	want to reach conclusions on the work that's ongoing and the regulatory basis,
21	but I suspect that we will find that the methods used there aren't going to fit our
22	needs where we want a significant participation
23	COMMISSIONER OSTENDORFF: Well, let's I'm sorry, just
24	because to be mindful of the time, let's put aside the process. I'm talking about

as far as the substance of the safety approach, anything in the technical area

that's different than those other organizations compared to how we are currentlydoing business here.

JOHN ADAMS: Well, we use a graded approach, which the Department of Energy, who has several large research and test reactors, they don't rely as heavily on that graded approach because we have to deal with 5 watts to 20 megawatts. So, a graded approach becomes very, very important in our process. So, again, I think that when we come right down to it, I don't believe that we're going to gain any synergy between the processes used for those facilities because they're significantly different.

COMMISSIONER OSTENDORFF: Okay. One area that -- this will be an anecdote, but I'll ask you to react to it, and then Tim, and Jessie, and Bill, as well, if you'd like to. I've heard one concern voiced by operators of research test reactors that falls in this kind of category that, gosh, these guys are asking me -- "these guys" being the NRC -- to go back 30 years in the past, recreate a design basis for, you know, this reactor, those license and I'm going to make up this data of 1975, and the people that were involved at that point in time, the contractor, the research university support, some of that information is just not available. So, why doesn't NRC just look at what's changed, just look at the aging management kind of issues? That's been a -- I'm trying to capture a little bit of a theme I've heard in two different conversations with RTR operators at two different universities. Can you react to that?

JOHN ADAMS: Well, I can say that that's one area that we did task our contractor to look at, is that specifically, so -- but I can't comment in any great detail because I haven't really been --

- 1 assessment -- I mean, obviously, you guys have experience, so let's put aside
- 2 the contractor work. What do you -- you must have some gut feel on that -- you
- 3 know, some reaction to that statement.
- 4 JOHN ADAMS: Well, I can go back to the day when I was the
- 5 reactor manager at Iowa State University. And I have some sympathy that I
- 6 share with those folks that have given you that position. It would seem very
- 7 logical that you could do that. But what's important to realize is with the creation
- 8 of the standard review plan we have, I think that's where the difficulties have
- 9 arisen is we've established a more formal process that ensures that we look at all
- 10 aspects relative to safety. And that is a much more formal process than it has
- 11 been in the past when most of these facilities were issued a license. So, I'll let
- 12 Tim --
- 13 COMMISSIONER OSTENDORFF: Tim, do you want to add to
- 14 that?
- 15 TIM MCGINTY: It's -- we have to be reasonable about this, but
- creating or reestablishing the platform, the safety basis platform for which you
- 17 assess changes from now and into the future is an important endeavor, and it
- can be somewhat frustrating. But that's why, you know, we have, as a staff,
- we've tried to work more with our licensees to establish that co-understanding as
- 20 opposed to just doing it, as Bill was mentioning, with the traditional reactor
- 21 process where there's a lot more resources available. So, there -- as John said,
- there is some logic to the views that the community has, but from the position of
- a regulator, having an established basis from which to assess is paramount.
- 24 COMMISSIONER OSTENDORFF: Okay. Thank you,
- 25 Mr. Chairman.

CHAIRMAN JACZKO: Commissioner Svinicki.

COMMISSIONER SVINICKI: Well, I'll begin by commenting on the progress that has been made. I think Chairman Jaczko mentioned the meeting I think three years ago, but there might have been one in 2008 as well. And I recall when the Commission first directed some focus on this, there was a really substantial backlog. We talked about 19, but I think there were more, of course, at that time. And on a denominator of, you know, 30-some facilities, it was a substantial backlog. So, I think to sit here in 2012 and have really worked it down and to have a clear path forward on the remainder is a really commendable amount of progress. And I don't know, given some of the things we've touched on here in terms of the needing to remediate or rehabilitate some of the existing safety analyses within staffing and resources on the university side, all of these things that are just the factors that contributed to having the backlog. And then, of course, there's the kind of accident of timing that many of them were licensed, you know, at the same time. So, we've got this clump.

I'm actually trying to -- so I'm happy about the progress that has been made. I'm going to ask you a couple of questions about continuing to push forward on elimination of the backlog, but I'm also trying, as we talk about the long-range plan, John, that you talked about, trying to think about the next wave, you know, 20 years from now, and how can the NRC position itself to not have to go -- well, first of all, we hope that there won't be another large external event that has just been an accident of history, I guess, that's contributed to the demands on the NRC's resources every time we're up against this 20-year wave, but how can we have the regulatory framework in the best shape. And I think that when we think about rulemaking, we need to think not just about near-term

benefits. We might be feeling good that the backlog is gone and we will leave	1	benefits.	We might be feeling	good that the	backlog is gone	and we will leave
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- 2 our successors 20 years from now the next wave, but we also have some
- 3 obligation to enshrine whatever it is that we learned in this wave and move
- 4 forward, and perhaps have our successors 20 years from now best positioned to
- 5 deal with the next wave of renewals, however many there might be at that time.

6 I will ask just on status, in the presentation today, I think that what

7 was presented was the RTR license renewal backlog would be eliminated by the

8 -- oh, this is a projection, of course -- by the end of fiscal year '14. In December,

just this past December status report, the staff was projecting that in March of

2013, so, the backlog could be eliminated -- I think if I have those dates right.

So, it seems like something has caused us in just about four months or so to

push that out. Tim, do you want to talk at a high level about what -- what you

13 attribute that to?

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TIM MCGINTY: That is -- it's certainly not an issue of availability of staff resources. The backlog -- for terms of definition, the backlog -- and I think you touched on it already -- as new license renewals continue to come in, they are part of what's not done yet and part of a backlog, if you will. And so, we've frozen a point of time. Instead, in 2009, there were 19 RTRs in the backlog. What has -- it's -- and I don't know actually the actual facility that has been deferred out of 13 into 14. But I believe it's relatively small delays associated with working with the facility on a time scale that they can respond to the necessary -- to the questions that we --

COMMISSIONER SVINICKI: That's kind of what Jessie had talked about, working with them, too. And so, it sounds like you're saying it's really only, as you recall, at one facility that has kind of pushed that out. And again, it's

1 because we're working and collaborating on when they can provide responses.

JESSIE QUICHOCHO: Absolutely. The 2013 timeline was based on the staff's projection on schedule. The 2014 timeline is based on what we've seen in how to create the dialogue and interacting with the licensees to provide

us their timeline. And so, that's the 2014.

COMMISSIONER SVINICKI: Okay, I appreciate that clarification.

That's very helpful. Also, at the meeting, if I'm recalling three years ago or so, we talked about we were heavily contractor-dependent in terms of conducting the reviews. We've talked about staffing up the branch on our side of the house. But one of the things that the RTR community panel had talked about at that meeting was if contractor personnel changed, they got a whole new set of RAIs as if someone was beginning their review all over again, and they indicated sometimes the RAIs didn't even show an awareness of the technology or the differences between power reactors. Has that situation changed from three years ago? And if so, how?

JESSIE QUICHOCHO: Yes, that has changed for the better. The what we've done is we've basically educated the contractors on numerous
occasions to get them to understand the philosophy behind licensing research
and test reactors. We've held numerous training sessions since then. In fact,
we've just completed another session this year. And we also hired a different set
of contractors that are more familiar with the research and test reactor
community. So, there is still some work. We are taking other actions to minimize
the impact to the facilities.

TIM MCGINTY: And I think the use of the graded approach has allowed us as part of screening the contractors' work has allowed us to more

efficiently reduce the number of unnecessary questions. But, again, this is an area that we also will need to continue to improve.

assistance that was mentioned, I wasn't that familiar with that, but it sounded like, from your presentations that that was in some measure a contributor to being able to work off the five in one year, I think, in 10, and the five in 11 out of the backlog. Could you talk -- is that an ongoing assistance program, and does that continue to be available to some of these applicants who might need some technical assistance?

JESSIE QUICHOCHO: Yes. The assistance is still available to applicants. We do highlight the DOE assistance to licensees. There have been a few that have taken advantage of it. In addition to that as well, there are a few that reach out to those facilities that just recently were issued a renewed license to assist them of similar design. So, does that answer your question?

COMMISSIONER SVINICKI: Yes, yes, thank you. I appreciate it. Sounds, again, like that was a positive contributing factor, but I wouldn't want us to have too much of a reliance on that. I know that DOE's budget is under, you know, some stress, so it may be something that, again, it's been very beneficial it sounds like in working down the backlog, so I just wanted to ask a little bit more about that.

This was a question that Commissioner Ostendorff had raised, but it is this general theme that, at the time of originally licensing these RTRs, the standards were different in terms of having the documented, the safety bases, you know, the safety analysis reports. So, from what I have observed on this issue is that those seeking renewal have had to undergo at times a very, very

substantial reconstitution of our safety basis. On the other hand, if the NRC is
needing to make contemporary findings, we have to make findings now about the
safety of continued operation. It seems to me that, as Commissioner Ostendorff
said, those seeking renewal may say, "Well, why can't you just look at the years
of operation we've had," and I'm sure we're informed by that, but at some point, if
we have to make findings today, we have to have some sort of something that is,
you know, a contemporary basis upon which to make those findings. But I also
understand, at times, that requires a very substantial undertaking by some of the
licensees.

Now, one of you had made mention of the NUREG that came out in 1996. Do you think, again, looking 20 years into the future, if enough of during this round of renewal that reconstitution and documentation has had to be done, and I know that conformance with the NUREG is not compulsory, but I took from some of the presentations that a number of the facilities are now at least more in alignment with a more contemporary safety analysis report. Will that stand us in good stead? Will that help in the future so that is this the kind of reconstitution we'll only have to do once?

TIM MCGINTY: The short answer is yes. I think you initially raised in your opening remarks that we have a responsibility 20 years from now to leave a legacy that best enables the community and the staff to make timely and safe decisions and minimize the impact on the licensees. And that is exactly it.

COMMISSIONER SVINICKI: Jessie, did you want to add something?

JESSIE QUICHOCHO: Yes. Early on, we recognized that -- we recognized that documentation was lacking. The goal is that all facilities will

- 1 have a documented safety analysis and that future licensing actions, whether it's
- 2 a license amendment, a simple license amendment, or a complex renewal, the
- 3 licensees and the NRC are more in a better -- in a more better position to
- 4 perform those more efficiently and effectively. I just do want to make one more
- 5 comment on that. And that is that I see the future of renewals, in my opinion,
- 6 could be early engagement with the facilities. And based on the documentation
- 7 that they have today, with all of the renewals being completed, I feel it would be
- 8 more efficient than it is today.
- 9 COMMISSIONER SVINICKI: Okay, thank you. Thank you. And
- on that positive note, thank you, Mr. Chairman.
- 11 CHAIRMAN JACZKO: Commissioner Apostolakis.
- 12 COMMISSIONER APOSTOLAKIS: Thank you, Mr. Chairman. Is
- the graded approach to the licensing the same as focused approach? I think
- they are the same. Or the focus is different from graded?
- JOHN ADAMS: Well, the graded approach takes a look at the
- hazards based on the specific facility. And with our research and test reactors,
- the driving cause for increased hazard is typically power level, and because of
- 18 increased source term and things of that nature. So, for the most part, as the
- power levels go up, we engage more in depth to ensure that we've addressed all
- of the safety-significant aspects of that facility. With a lower source term,
- 21 typically, we don't have to look at the -- well, actually, for the smaller facility; they
- 22 don't have the systems that the larger facility has. So, they're not there, so we
- 23 don't have to dwell on the KE removal as much, because the small facilities
- 24 actually can air cool quite nicely. So, they don't need engineered systems to do
- 25 that. So, I'm not sure what the focus to -- if you're referring to the focused review

- in the ISG, that does take -- does employ the graded approach, if I understand you right.
- 3 COMMISSIONER APOSTOLAKIS: Well, they are two different
- 4 things. Graded means you take into account the power level, which is two
- 5 megawatts, I believe.
- 6 JOHN ADAMS: Well, the focused review on the ISG used the two-
- 7 megawatt criteria as being eligible for the focused review. You had to be less
- 8 than two megawatts.
- 9 COMMISSIONER APOSTOLAKIS: Graded means I'm looking at
- the power level, and then if it's below two megawatt, I apply the focused review.
- 11 Correct?
- 12 JOHN ADAMS: That's correct.
- 13 COMMISSIONER APOSTOLAKIS: Okay. And in the focused
- review now, you look at the reactor design and accident analysis and so on, but
- 15 not -- you don't look into the system analysis itself in the detail. That's my
- 16 understanding.
- 17 JESSIE QUICHOCHO: Correct. We -- what we do in the focused
- 18 review is we look at those components, equipment and systems programs that
- 19 are safety significant to the facility. And so, there's -- we look at the technical
- 20 specifications, the operations of the facility, the accident analysis, things of those
- 21 -- in those areas where we focus on the facility at two megawatts and below, or
- less than two megawatts. And the reasoning for that is that at two megawatts
- and greater, there's a higher risk. As John mentioned, there's an increased
- 24 source term. For facilities of two megawatts or greater, their emergency planning
- 25 zone now expands beyond the facility of the controlled area. So, there are --

1	COMMISSIONER APOSTOLAKIS: Just take one system. Let's
2	say that system appears in some version in a smaller reactor, less than two
3	megawatts in the larger. What would you do different? I mean, you still have to
4	remove heat, right. So, how what would be different in the focused review?
5	How much more would you do in the complete review?
6	JESSIE QUICHOCHO: I understand. The question if I
7	understand your question correctly, it's what areas do we look at in the focus
8	review versus
9	COMMISSIONER APOSTOLAKIS: Yeah, what do you think about
10	I mean, if I had to do a complete review for a 10-megawatt reactor, but then the
11	same system appears again, say, in a smaller reactor, what would they leave
12	out? Or what is it that I would not pay as much attention to, because we rarely
13	leave things out?
14	JESSIE QUICHOCHO: The development of the interim staff
15	guidance was to narrow the scope of the review, and that was based on
16	determining what chapters there is 18 chapters, and it was part of an
17	application. And so, we focused on, I believe, four chapters that were, you know,
18	contributed to the safe operation of the facility. And that's the the focus review
19	is we chose not to look at other chapters and looked at those chapters that were
20	pertinent to safety
21	COMMISSIONER APOSTOLAKIS: I assume you look but not in
22	detail.
23	JESSIE QUICHOCHO: Correct.
24	COMMISSIONER APOSTOLAKIS: Okay. Now, in this review, are
25	vou looking at aging effects?

1	JESSIE QUICHOCHO: We do we look at, for example, we look
2	at operational history. We look at if there was a we look at inspection reports
3	in the last so many years. If there was something that was documented
4	inspection report that, say, there's a tank leakage, we would ask questions on
5	that, what have they implemented to mitigate against any future happening of
6	that. Some facilities changed their tank liner, have changed their tank liner out
7	as a result of that, not necessarily as a result of their license renewal, but has
8	taken steps to address that. Some facilities have installed pool level devices so
9	that they can monitor should there have any leakage.

COMMISSIONER APOSTOLAKIS: But, as you know, in the license renewal process for power reactors, the whole process is aging oriented. And they don't just look at past incidents. I mean, they go into details of analysis and so on. And here, it appears -- I mean, you have reactors that are very old, right. You said some of them are from the '50s. It would seem to me that the aging would play a major role. Why isn't that so? I mean, you seem to imply that it's part of what we do, no big deal. Is that -- is that a correct impression? It is a big deal? Okay.

JESSIE QUICHOCHO: No. I don't want to leave you with that impression. We do look at, when we -- when the applications do come in for renewal, and we do look at the operation for the next 20 years, absolutely. And so, we do look at if the facility's going to be operating for the next 20 years, how do we address some of the operational issues that they have gone through.

COMMISSIONER APOSTOLAKIS: But, again, if I look at the power reactors, typically what happens there is the licensee is asked to establish more programs to manage aging. Do you ever do that?

1	JESSIE QUICHOCHO: Well, there is no specific program for
2	managing aging, for research and test reactors.
3	COMMISSIONER APOSTOLAKIS: That's what I'm getting a little
4	confused. Why? If I have a reactor that's 50 years old, that's not an issue?
5	Maybe you can give us an answer
6	TIM MCGINTY: Maybe we can give an answer after. Maybe you
7	could also ask some of the presenters that will follow us what their views are
8	COMMISSIONER APOSTOLAKIS: I may do that. Thank you.
9	Now, another topic. With I mean, I understand the review, the license renewa
10	review is really deterministic, right. You're relying on very conservative
11	assumptions and all that which is what we used to do well, we still do with
12	power reactors. And then, we did PRAs and we discovered that certain
13	sequences were a surprise to us like station blackout, anticipate the transit
14	without scram and so on.
15	Now, I fully appreciate these are low source term reactors, but,
16	again, these reactors are in the middle of cities. The MIT reactor is in
17	Cambridge. You know, people walk outside and drive and so on. And you don't
18	have to kill people to create a problem. I'm wondering, has anybody thought
19	about doing maybe a mini PRA so that we'll not we will be assured that there
20	will be no surprises, that the intent would not be to see, you know, whether you
21	have health effects, because I'm pretty sure you don't. But, again, you know,
22	there may be something that the deterministic methods have missed and like a
23	station blackout. And would that be helpful, or I don't know. You realize I had
24	to ask a question like that.

25 [laughter]

1	JOHN ADAMS: Actually, we could do an interim or a study of I
2	would recommend if we were going to pursue that, we would take one of the
3	larger research and test reactors that we have, and you could do some PRA
4	work. Historically, we have never felt that we're going to gain a lot of information
5	that's going to significantly direct our regulatory framework in a different direction.
6	So we didn't think we'd ever get any return on our investment there. But you're
7	right. We could do a test case and see what we do learn. In fact, given that
8	information, it could possibly enable us to further risk inform our regulatory
9	framework. We could be enlightened by information that would lead us to believe
10	that where we are focusing our inspection program currently isn't the areas of
11	highest risk, and it could inform our inspection program.
12	So, there is the possibility to learn usable information from a pilot

risk assessment program is certainly there. But to require our licensees to implement a program specific for their facility, as you probably are well aware of, the cost of conducting probabilistic risk assessments can be significant. And I think our licensees would need some type of assistance if we found a direction we wanted --

COMMISSIONER APOSTOLAKIS: No, I appreciate that. And I didn't mean that we should demand PRAs from the licensees, but I am thinking in terms of power reactors, it seems to me we will feel much better about our license renewal approach if we eliminate the possibility of a surprise that, you know, you might look into systems analysis and find, my God, I hadn't thought about that. So, it would really be internal, and it doesn't even have to be a complete full scope and all that. Thank you, Mr. Chairman.

CHAIRMAN JACZKO: Well, I just had a couple of questions.

- 1 Touching on what Commissioner Svinicki asked a little bit about, which I think is
- 2 a very good point is that the licensing basic reconstitution seems to have been
- 3 the big part of a lot of the challenge as we go forward, and I tend to agree with
- 4 her assessment. And I think, Tim, you hinted at that as well, that that is
- 5 something that should make the next round better, assuming we properly
- 6 maintain it. So, that -- if there's any question I have on that, it's that question.
- 7 Do we have the right programs, whether it's guidance on licensing, or whatever,
- 8 or regulatory, to ensure that the licensing basis are maintained for the next 20
- 9 years, so in 20 years, when we're back here again, we're not trying to

record of the facility licensing basis that sets us up for that.

- 10 reconstitute a licensing basis again. Is that your sense or do we need work
- 11 there? Anybody...

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TIM MCGINTY: Yeah, I think we have established a program that has more rigor and durability that is focused -- it's related to -- we've got more of a focus on process, putting documentation into ADAMS, for example. That's occurred since the 1980s -- a general focus in training our staff in how to conduct review for documenting the basis for our decisions, through our qualification programs, a number of different facets that lead to a essentially a more durable

CHAIRMAN JACZKO: Well, that's good to hear. And I think that should hopefully help in this area. The -- which leads me to another thought or question. In 20 years from now, we'll be looking at reactors that are significantly older, and again, appreciating that they have very different source terms, they have a very different radiation field than power reactors that we're more accustomed to, well, certainly, I would say, I'm more accustomed to dealing with. Have we asked ourselves the question, I mean, is there a finite life for these

- 1 reactors? Are we -- is the next 20 years even feasible, another 20-year renewal,
- 2 or will we at some point reach the stage at which we're looking at
- 3 decommissioning for really most of these reactors? I mean, you know, right now,
- 4 mostly for the larger reactors we're looking at, you know, there's questions about
- 5 60- to 80-year operation. I don't think we have definitive answers yet on whether
- 6 that's feasible or whether that's doable or what the cost would be to do
- 7 something like that. Do we have a similar sense with these reactors? Are they
- 8 on the same kinds of timeframes, or given the source term, the different radiation
- 9 field, that you don't have the same kind of effects on materials that could shorten
- 10 life spans?

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TIM MCGINTY: And also, given our tracking operating experience and our inspection and oversight program, which continually verifies the status of the facility, I hesitate to compare the timing of commercial power operating reactors to RTRs. We haven't systematically asked ourselves that question, though, and assessed it in detail. And there's always a significant cost in terms of effort and on the licensees associated with such an effort. And so, we haven't formally energized that.

JOHN ADAMS: One consideration with research and test reactors, many of them are small enough and the complexity of systems is not great, that you can actually replace any component in the facility with a new one, which is not very feasible with power reactors. So, they are very maintainable.

CHAIRMAN JACZKO: Anybody else? Okay, well good. Well, thank you for those -- I think I, as well as my colleagues, look forward to, you know, the completion of the regulatory basis and what your recommendations are as far as possible regulatory changes as we go forward. But I think we've

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- 2 Thanks. We'll take a quick break.
- 3 [break]
- 4 CHAIRMAN JACZKO: Well, we'll get started. We'll start with Leo
- 5 Bobek who is the chair of the National Organization of Test Research and
- 6 Training Reactors.
- 7 LEO BOBEK: Good morning Commissioners, and thank you for
- 8 the opportunity to provide this briefing to you. My name is Leo Bobek, I'm the
- 9 2012 Chairman of the TRTR --
- 10 FEMALE SPEAKER: The microphone is not on.
- 11 LEO BOBEK: Again, I am Chairman of the National Organization
- of Test Research and Training Reactors, and my other full-time job is director of
- the University of Massachusetts Lowell Research Reactor. Before providing you
- the take or TRTR perspective on relicensing issues, I wanted to take the
- 15 opportunity and provide a perspective on TRTR and research reactors in general.
- 16 I believe both are relevant to this discussion.
- 17 The TRTR membership includes NRC-licensed facilities, DOE-
- 18 licensed facilities. We have members from Canada and, at various times, other
- 19 nations including countries in South America, Australia, European and Asia
- 20 Pacific nations. So we have quite a broad membership and diverse membership.
- 21 We do refer to ourselves as research and test reactors. The
- regulations refer to us as non-power reactors, or NPRs. I'm sorry, I'm not sure
- which slide we're on. I should be saying "next slide." There we are. Research
- 24 and test reactors are termed non-power reactors under 10 CFR 50.2 definitions.
- 25 The only other definition associated with research and test reactors is for testing

- 1 facility, which describes a reactor that is 10 megawatts or greater and has
- 2 various other differentiating factors such as fueled experiments or liquid fuel
- 3 loading. Other than that, you're a research reactor.
- 4 As far as RTR's overview, you've heard some of this from the staff,
- 5 there are 42 RTRs licensed under NRC. Thirty-one of those are currently
- 6 operating. Twenty-five of those facilities operate at university campuses. Most
- 7 of these facilities are class 104 licenses for research and development. And
- 8 most were built, as you were told, between 1960 and 1980, which has been
- 9 significant for the relicensing issues. The licensed thermal power levels range
- 10 from a few watts up to 20 megawatts of power. Next slide, please.

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11 It's important to emphasize that RTRs are not small nuclear power

reactors, and I think that point has been made several times during the

presentation. RTR usage requires a much different design from power reactors.

14 RTRs are designed to produce high thermal neutron outputs at a much lower

thermal neutron power level. And the reason for that is because these are

facilities that are used for neutron research. And we'll talk about that in a minute.

The RTR core volumes are physically much smaller than a power reactor. Most of the research reactors have core volumes that are less than a cubic foot. Power reactors are tens of cubic feet. And so, RTRs require much less fuel, a few kilograms of uranium versus tens of kilograms for power reactors. RTRs have far lower fission product inventories, a few orders of magnitude for most of these facilities. And RTRs lack the thermal hydraulic energy to disperse fission products in the event of an accident. And that's not to say that these facilities are not safety concerns at all. The reactors do provide a significant source term, depending on the power level. However, it is fractional compared to

1 power reactors. And I keep on referring to power reactors in comparison to non-

2 power reactors, and that is because we are licensed under the same regulations

3 as power reactors. But, again, we are much different. Next slide.

As mentioned in the previous slides, RTRs are designed to produce neutrons for research and education purposes -- studies in the fundamental nature of matter, neutron physics, probing the molecular structure of materials using neutron scattering, imaging items that cannot be imaged with X-rays using neutron radiography, and the production of radioisotopes for experimental and research purposes. Next slide.

Research and test reactors are used for non-destructive analysis, neutron activation analysis, which is used to determine the elemental composition of materials. And the production of neutrons and the use of neutrons can be used for changing the characteristics of materials, and both structural and chemical for beneficial applications. Next slide.

So, RTRs offer a unique tool for instructing nuclear engineers, technicians, and scientists in radiation physics and nuclear engineering. The research reactors can be used for everything from subcritical multiplication of neutrons to measuring reactivity, higher-power level reactors; you can measure temperature and xenon effects. Again, these are all teaching and research applications at these facilities. But it's important to recognize that the margin of safety on these facilities is very high such that they can be used by students. And engineering students make mistakes. And these facilities are very forgiving. They're designed such that students can work on them. If we have a scram of a reactor, it's not as a significant event as it is with a power reactor. In fact, a lot of times that's a teaching moment for the students. You can teach them how to

- 1 evaluate a scram and then use procedures to restart the reactor and recover
- 2 from the event. But, again, it's not the type of event like what you would have
- 3 with a power reactor. Next slide, please.
- 4 So, again, RTRs are a unique educational tool. We're also used for
- 5 educating the public on nuclear technology. We offer tours of our facilities. In
- 6 fact, some of our facilities may provide tours to hundreds of people in any given
- 7 year. And one of the advantages of a non-power reactor, research reactor, is
- 8 that you can actually look down into the tank of water and see the reactor
- 9 operating. You've all seen the pictures or experienced it for yourself of the blue
- 10 Cherenkov Effect. And that's very useful for when you're teaching the public
- 11 that, first of all, nuclear power can be safe. It is highly regulated. There are
- many safety systems involved. You can look down into the tank of water, see the
- reactor operating and not become radioactive and go off to tell about it. So,
- 14 again, very unique and very useful teaching tools.
- So, now we get to the position of TRTR on the license renewal
- process. And, first of all, TRTR recognizes the unique challenges that have been
- 17 posed on the NRC during RTR relicensing in the last decade or so. Some of that
- has already been addressed; for example, staffing issues, the situation with 9/11.
- 19 And so, we certainly understand why a backlog occurred. And TRTR certainly
- 20 appreciates all of the efforts that have been made by the Commission and the
- 21 staff to alleviate the relicensing backlog. And we also appreciate the updates to
- 22 guidance and the opportunity that TRTR has been given to participate via the
- 23 public meeting process to provide input to that guidance.
- However, in polling the research reactor community, we find that
- 25 the relicensing process has become very complex compared to the way it was 20

1 years ago. And we understand that is because of the process. The process has

2 changed and is more formal. However, I put the question to you whether or not

3 there is a quantifiable improvement in safety due to this process since 20 years

ago. Most of these facilities have not changed. And the amount of effort

5 required in order to relicense them has been significant.

Some of the suggestions that have been provided to me by others in the TRTR community is that did the NRC consider a generic thermal hydraulic analysis for models of trigger and plate-type fuel reactors. Again, the facilities that are one megawatt or less, it's a very significant effort on their parts to do this type of analysis, and this has already been addressed, there is some efforts to get funding from DOE to do this. But for a small facility to do something like this, it's very difficult.

Similarly, for the maximum hypothetical accident analysis for non-power reactors, this is very similar to the beyond-design-basis event. For research and test reactors, it usually involves the release of fission products or hypothetical release of fission products from the fuel. And then the analysis has to be done to assure that that fission product release does not reach the public dose levels that are provided in 10 CFR Part 20, which, again, can be a very significant analysis for non-power reactors that are small in nature and small staffs.

As far as some additional generic suggestions for streamlining relicensing, one comment was to develop a systematic way outside of the RAI process to correct typographical and editing errors -- there seems to be a significant number of questions associated with that -- to develop a generic decommissioning cost analysis associated with previous experiences of facilities

- 1 that have decommissioned and indexed at the power level and inflation, and then
- 2 to endorse more of the usage of ANSI and ANS standards of regulatory
- 3 guidance. And I see my time is up. And, again, I want to thank you on behalf of
- 4 TRTR for this opportunity.
- 5 CHAIRMAN JACZKO: Thank you. We'll now hear from Sastry
- 6 Sreepada, who's the director of the --
- 7 SASTRY SREEPADA: Walthousen Reactor.
- 8 CHAIRMAN JACZKO: Walthousen Reactor Critical Facility at RPI.
- 9 SASTRY SREEPADA: Yes. Thank you, Mr. Chairman, for inviting
- me to come here and tell you our experiences in relicensing. And, as you said,
- 11 I'm Sastry Sreepada. I'm the director of the Reactor Critical Facility for about a
- 12 year and a half. Just after Commissioner Ostendorff's visit, I became the director
- of the facility. So, at that time, we were in a transition.
- Now, we talked about the big differences between power reactors
- 15 and research reactors. Now, compared to most research reactors, we are
- another order of magnitude below that. We are licensed for 100 watts. We
- operate to a limit of 15 watts. Mostly we run the place at 10 watts, and we run for
- about five or six hours a week. So I want you to keep that when I make the
- 19 safety considerations in this. Next slide, please.
- Now, to appreciate it, let's look at the facility itself. The facility is a
- 21 pool-type, light-water, light-water reflector reactor, we use water only for
- 22 moderation and shielding, and we don't need to remove any heat because it's
- 23 only 10 watts. It's not there for cooling. The reactor core itself is sitting in a
- 24 2,000-gallon water tank, 7-feet diameter, by 7-feet high. And most important, the
- burnup is so negligible, it's about two kilowatt hours per year for the core. So, in

- 1 the next 20 years, probably, we'll have 40 kilowatt hours. As we said, it tests the
- 2 specification limit. But we operate much, much less than that. And right now, we
- 3 use a low enriched fuel. Probably we are the only one with commercial rod type
- 4 fuel. We use the spent fuel. And that's obtained from the Department of Energy.
- 5 The core is approximately three feet high. And due to the extremely low burn-up,
- 6 we can access the reactor core, even touch the fuel minutes after we shut it
- 7 down. So, the students can load the fuel, take the fuel out in the middle of an
- 8 experiment, move the rods to learn more about what the impacts are.

So, because of this, the facility is mostly suited for critical benchmark tests, student training in designing experiments, student training in operations. And you do license our operators to run the facility, too. And it is potentially really good to train on the fundamental knowledge base for even power reactor operators for a continuous education, because that's one thing which they don't have an ability to train themselves. Next slide, please.

Now, like any of the research reactors, this facility was built a long time ago. It was built by an American locomotive company in 1956 to support the research for the Army reactors. And in 1964, Rensselaer Polytechnic assumed the operation of this. In '65, we had the first license in the name of the Rensselaer Polytechnic Institute. We had highly enriched uranium, we had in plate form.

And then, July 1969, we applied for the first renewal. We got that.

And the second renewal, now we -- this is how far I tracked with the system -second renewal, we submitted in 1979. There was one RAI, and it was licensed
in 1983. Those were the good days with a lot of people working in the nuclear
field all around the country. So it's not a question of lack of understanding on

- 1 both sides, but it took four years even then. And we amended the highly-
- 2 enriched uranium to low-enriched uranium. We submitted that amendment in
- 3 October '86. We had one RAI. And it was approved in April in 1987 for the low-
- 4 enrichment core. And recent renewal, it was submitted in November 2002, and it
- 5 was licensed last year.

And let's talk a little bit about the perspective on the safety and scope in licensing. We said we operate the core below 100 watts, and mostly 10 watts, less than two kilowatt hours per year, which is a few micrograms of burn-up, [unintelligible] if we had run the whole year. And in its lifetime, a few

milligrams of uranium-235 is fission, 20 years.

So, the reactor has a 2,000-gallon tank, and we don't need any cooling. The reactor room is surrounded by one-foot-thick concrete on three sides, and three feet between the control room and the reactor. The only thing we have is we have a vault where we store the fuel. My first concern when we had the flood was what are the extremes. We can flood the whole building with water. We are still separated out. So we will only have to clean up the place physically rather than have any criticality problems.

So the RTR building is supported on 104 treated wooden pylons each can take 20 tons. I know it was built in '56, but it was built strong in that sense. So, really, if we start the reactor and doubled its license to power like 200 watts, made the worst possible accident, we will release 10 kilojoules of energy, which will raise the cladding temperature by a tenth of a degree Celsius. So, now, we made the submittal and the submittal was 200 pages, 70 pages of safety analysis. Now the big question I always have is if I ask a graduate student, like a Ph.D. student, to go and write me a safety analysis report, he's

totally confused with all these numbers, NUREG 1537.1 reports to so many other areas, so unless somebody's already familiar with that, we tell him what it is. He can't do it and it should be done in three months as far as I'm concerned, for the reactor of this type. So now let's see -- next slide please.

So what worked well? The initial submittals from the last three renewals were prepared under the same director. We were very lucky. We had the same director from that day until the recent one. And the recent submittals we had the excellent contribution by our volunteer, our really very smart part-time employees of the old lab who help us run the place; it's run on that basis mostly. So we gain a lot of experience from them. So they have prepared the whole thing for us.

So graduate students help and basically the bulk of the material from the last low enriched core we updated the safety analysis report to bring it into the new format. So is this format going to be same 20 years from now? I doubt that, because if you have a focused report for a facility like ours, I should be licensed for a three milligrams of fission product handling than all these complications.

So beyond that, if you change the system again, if you analyze it with new systems, you have to have new computer course and I will demand computer course qualified if I am a good engineer. So that's a lot of work. So we have to have simple generic matters to qualify extremely small reactors like ours.

And what didn't work well? We are run on part-time staff.

Substantial turnover of people and I sit here for five directors during licensing process. If we had five directors change during the period of 2002 until now, which is nine years, five operations supervisors, five nuclear safety review board

chairmen, three radiation safety officers; it's not a good thing to have a coherent picture coming out unless we spend more time every time people change.

The first request for the additional information came almost six years after the submittal, so the director left, all the staff left, so we are working on papers. So what didn't work well? Difficulty to be on the same page with the constant change in personnel from our side. Even if we had a face-to-face meeting, we couldn't improve that, because the best thing if we ever had good record of all the telecoms we had and there were records written, how could I follow that? When I came and I spent almost six months reading the whole documents, but I was lucky because I was at the end of the licensing process.

So where did we spend most of the time? In decreasing order, technical specifications? Or the maximum time we spent safety analysis and other sections. What kinds of things we had to do? A lot of editorial changes, definition of terms, nothing related to the safety. But in three years if I really take from the first area I did good with 5 reactors changing so I think if we had a face-to-face meeting once or twice it could have been licensed.

So when we talk about focused approach for small reactors, I'm the smaller of the smallest. And it is necessary to have a continuity of personnel throughout process and some documentation of the discussions would have helped us a lot. And other than that, this is good.

What did we accomplish? We put the safety analysis in the new format. I can give it to students and say now you design the experiment. Tell me what part of this has to be changed because of that? So that's something that is good and the biggest problem we have at our end is attrition of the staff and most probably all the senior people are on the way out. I am a retired employee from

1 Lockheed Martin, so that tells you the story.

So basically if we can increase the new faculty for [unintelligible] we'll be in a better shape. So other than that I think our licensing process is not a bad process, but is it needed to be this complicated this long? That's something we all can ask. Thank you for the opportunity.

CHAIRMAN JACZKO: Thank you. We'll now turn to Ralph Butler who is the director at the University of Missouri research reactor.

RALPH BUTLER: Well, thank you for this opportunity to come before you and share our experience with the license renewal process. As a quick overview of the facility on slide two, we are located on the University of Missouri Columbia campus approximately one mile south of the main campus. We are a pressurized beryllium and graphite reflected reactor in an open pool, light water moderated and cooled. We are at 10 megawatts and, as such, we are the largest of the university-operated facilities with a staff, full-time staff, of about 160 FTE. We fully support the four missions of the University of Missouri of research, education, service, and economic development. Next slide please.

We initially began operation in October of 1966 at five megawatts and in the mid-70s we upgraded to 10 megawatts. The facility was originally designed for 10, but operated at five for the first few years. An important note is that we started operating seven days a week, or six and a half days a week, greater than 150 hours a week, in 1977, and we have continued to do that around the clock. We submitted our license renewal application in August of 2006. Obviously a vast amount of resources went in to developing the revised safety analysis report and recapturing the construction period point. So, today we've probably invested 10 to 12 years worth of effort toward license renewal

with significant resources. I also need to state that we are working toward fuel conversion from high enriched to low enriched. This also puts a strain on the limited resources that I'll mention later on. Next slide.

Just a couple of slides on the timeline. As you can see, some three years elapsed between the time we submitted our application until we received the first RAIs, which were more on the simpler items of decommissioning and financials. And I'll come back to that in a moment. And then some on reference material and environmental report. On the next slide, you'll see we began to get into the technical questions in May of 2010 timeframe with 19 questions, what we considered to be extremely complex, given 120 days to respond, and then in June of 2010, another 167 questions with 45 days to respond.

On the next slide we talk about our comments and our experience and observations. Some of the things we struggled with was during that three year lapse of starting the RAI process, we felt like we had to go back and reconstitute some of the questions we had already provided, such as in the financial and decommissioning. We had to go back and update all of that previous information we submitted in the application. A very large number of questions of varying complexity limited resources to respond in such a short period of time. I stated we had 160 full-time employees, but there might be perhaps five of the knowledge base necessary to respond to these types of technical questions. Those same five individuals also support our fuel conversion effort. At the same we had Argon National Labs supporting us on field conversion effort and I cannot emphasize the importance of that effort for fuel conversion that helped us on the relicensing effort. But I must say that the sheer number of questions to answer and given 45 days, we obviously walked

out of the room overwhelmed, to say the least.

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On the next slide, some of the questions required significant computer code work. This obviously creates a challenge. I know it does for the smaller facilities, because it created a huge challenge for us. We have limited resources who are competent in some of these computer codes. Some of our computer codes that we've been using were older versions, so we had to go out and acquire the latest versions. We had to send an individual off to training to learn some of these new versions, and again, this is where Argon helped us tremendously. We piggybacked off the fuel conversion mark so that we could come up to speed on developing a model of our reactor to use with these computer codes. But it did require significant resources. In some cases we felt some of the questions were already answered and that the reviewer either missed it in the review of the safety analysis report, or asked for greater detail. Anyway, we spent considerable time going back and saying it's on this page. On the next slide, one of our recommendations is to increase the number of site visits. We had two site visits and one site visit when you're

number of site visits. We had two site visits and one site visit when you're handed 167 questions, you spend all day in a room trying to reach some consensus on the interpretation of the question. With that many, at the end of the day there were still quite a bit of vagueness in our mind on how to answer some of the questions. So it would've been nice to have had broken the questions up or had multiple site visits so we could've spent more time trying to understand what was being asked of us.

One of the other suggestions that we have is perhaps a database or access to some of the common design features among reactors wherein that the questions asked of any of the facilities are just generic questions as the NRC

follows its process that even though we might know the answer, we didn't know whether we were answering it sufficiently or over-answering or if we could've had access to other facility's questions and how they responded. Certainly for the smaller facilities it would've been beneficial to say this is what the answer should be or -- excuse me, or this is sufficient enough. So we spent considerable amount of time kind of arguing among ourselves are we answering the question

and are we answering it to the level necessary?

On the next page, next slide, one of the good things you come out of trying to reconstitute some of your safety basis, we found a discrepancy in the work that was done in 1974 going from five megawatts to 10 megawatts and the Bernath coefficient caused us to question one of the safety limits and with a vast amount of work with Argon National Laboratory. And because of our work on fuel conversion we had new modeling and a new set of peaking factors, so with that we were able to show that the safety limit really had no impact in margins and safety had remained the same. But this probably consumed six months of our time in just trying to address this thing, but the positive side is we identified something that was -- the interpretation of the Bernath equation was different and so we were able to solve that issue.

The other suggestion we have is some more realistic timelines on trying to respond to questions. Forty-five days for 167 is kind of scary.

I guess the positive thing is here -- again, we were very fortunate and I can't say enough about the support from Argon and the fuel conversion market. We were just fortunate that these two things were running in parallel, so it supported us greatly. We had a vast amount of support from our senior project manager. We spent a lot of time on the phone going what do you mean here?

- 1 Or seeking his guidance. But in the same line the staff has been very supportive
- 2 and understanding of our requests for additional time. They know that we are a
- 3 staff dedicated to attempting to answer these questions and we're making pretty
- 4 steady solid progress, it's just a time-consuming effort. So we appreciate the
- 5 NRC staff's understanding on that.
- On the next slide, where are we today? I think the end is in sight.
- 7 We have six questions remaining. These are in the code work area, so requiring
- 8 a vast amount of effort, but we hope to have these completed by the April
- 9 timeframe and submitted, so we'll have completed the RAIs. One of the other
- things that happens to us with the revised safety limit, and we have an
- amendment in for that, is that we'll have to go back and rewrite portions of our
- 12 chapter four.
- So, anyway, thank you for your attention and I look forward to your
- 14 questions.

- 15 CHAIRMAN JACZKO: Well, thank you for your presentation. We'll
- now turn finally to Stephen Miller, who is the head of the Radiation Sciences
- 17 Department and the reactor facility director at the Armed Forces Radiobiology
- 18 Research Institute.
- 19 STEPHEN MILLER: Thank you. So, next slide please. So I
- thought I'd start with a little bit of history of the facility. We achieved our first
- 21 criticality impulse in 1962. It was August so we're coming up on our 50th year.
- We were in the middle of relicensing in 1984 and I was involved in that and I
- 23 knew that I was never going to be involved in another one, I'd be somewhere
- else. And, of course, here I am.
 - So, during that 1980 renewal, we did completely rewrite our SAR

and we decided on our own. It was not an NRC request, but we decided that we needed to refresh that and have a good starting point so that we wouldn't be doing this in the future. So, our current renewal package was submitted in July of 2004, under timely renewal and AFRRI to date has logged over 50 years of safe operations. Grand total for our operational history to date is about 40 megawatt days. Okay? We're a biological research facility, cell cultures,

electronics to go up into space, those sorts of things. So we run for an hour at relatively low power levels here and there and move on. So we haven't burned up a lot of fuel. Next slide please.

So, in August 2005, we started our relicensing process. We had NRC along with the contractor at Brookhaven visit AFFRI and we discussed the first round of 95 questions that were put in front of us on the table. All but seven we were able to answer to everybody's satisfaction on the spot. And we were left with seven that were either highly complex or needed a great deal of work. And the license process stopped. There weren't resources, you know, a lot of reasons. In June of 2010 we started up again and we started out with four RAIs that were financial questions. We submitted about a 300-page financial statement for a government-owned facility to show that we'd be solvent when it came time to decommission. Next slide please.

So, in July of 2010 we received our first round under the focused review process of 41 RAIs and working on those RAIs we eliminated all but about three that required some modeling support that we just simply didn't have at our facility. So, the budget year in September in the federal government is over. We couldn't do anything until October, so as soon as October rolled around we were going to request some funding. Along the way we requested some assistance

- 1 that we had talked about from DOE. We are also a university. We do fall under
- 2 that category. And DOE looked at the questions, prepared a package to go to
- 3 General Atomics and said you've got to be kidding. This is way outside of our
- 4 budget range. You're on your own. So anyway, we were waiting for money.
- 5 With the budget year closed out, October is coming upon us. Next slide please.

6 And the entire budget year for FY11 there was no budget. We 7 were under a continuing resolution and we were told that under a continuing 8 resolution we can't start this and we'll just have to wait for Congress to approve a 9 budget. So, we still had those three RAIs that we were pushing farther and 10 farther down the line. We just received our budget on March 8 of this year, so it 11 has taken that long. Completely out of our control. We kept telling the -- our 12 project manager, we get our budget when we get our budget. We can't -- there's 13 nothing we can do about it. So anyhow, so the contracting process to General

Atomics is underway and next slide please.

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Okay, so to date where have we been? In addition to the original 95 RAIs, we've exchanged 143 RAIs back and forth. Some of them were as simple as state for the record that you're not owned by a foreign government and others were extremely complex, outside of the expertise that we have on our staff and needed outside assistance. The good news is that we're almost done. We expect the process will be complete this year, but at what cost? We've estimated our outlay for this process to be in excess of a million dollars and that's not including what NRC has put into the process, which is probably very close to it. That included money to yet again rewrite our SAR and modeling assistance from General Atomics and the staff time and other resources that were necessary within the institute and an additional temporary hire. Next slide please.

So we're hoping to be done by the end of this FY. So that's the good news. We're all very excited about it. So, next slide please.

So I was asked to speak a little bit about our experience with it, so as I stated earlier, I've been involved in two licensing activities for this reactor: two licensing activities and two refueling for a large cobalt facility, as well as a great deal of work on our by-product license. So, I've been around the block once or twice with this. First one being four years and USCR was put out at that time. The current effort began in 2004, and we have on staff to help with this process, three people that are permanent. I have an electronics technician. I have an administrative specialist. And I have an engineer. In addition to that we have three Military people that are here for training, so in addition to that we have a continuous training burden. And that's what we're given to handle the process.

So, the mention was made of the staffing up of NRC. My comment to that is where do those people come from? A lot of them came from the research reactors. In fact, one of my people is working on licensing process.

Not for our reactor, but -- so there are a small number of people in the community and when one person hires, generally somebody else is the loser in the transaction. So, even though NRC was staffing up, that pool of personnel came from somewhere, many of them from the nonpower world. Next slide please.

Without exception, everybody that we were involved with, the NRC, the NRC contractors were helpful, professional. Everybody was reasonable. When it came to deciding timeline for RAIs they were very understanding. My commitment to NRC was that if I was going to be late I would let them know long in advance and not just let it sit wondering where, you know, so we kept that part of the deal. But everybody was easy to deal with. They

1 were professional. The questions were clear. Next slide please.

So I really don't have any complaints. The question really is what are we doing? So, I'm sure everybody's brought this up on numerous talks with - so, the NRC is -- the Atomic Energy Act stipulates that the Commission shall impose minimum regulation to protect the public health and safety. I'll say no more about that. Next slide please.

NUREG 1537 Part 1, Chapter 13.1.1, states that for a trigger reactor, which we do have, the MHA is a fuel element that's broken on the deck after sustained operations and the resulting accident is analyzed, releases, doses to the public, et cetera, and we found that we were within Part 20 limits. At that point, do we stop? That's the question I have. We've shown that there's not a significant safety issue with the operation of the facility and the rest of it are the nuts and bolts of the facility itself. So, that's -- that was one question that I had for everybody to ponder. Next slide please.

So, what was the benefit of public expenditure in excess of a million dollars to do a new safety analysis report? To do thermohydraulic analysis and all those things? Was there a safety benefit in addition to what had already been done? So, next slide please.

So in the '60s when these reactors were built, GA took a few apart, blew a few up, and did whatever they could to make them fall apart and found out that the world was safe. Do the analyzed safety margins from all the mathematical modeling that we've done really add to that? Is -- you crack an egg you know how it breaks, right? Does modeling it enhance that? So, next slide please, and last.

So, my question would be is there a more efficient way to license

- 1 these reactors, particularly at the level that we're running at? Are we getting an
- 2 increase in safety for the money that was spent for the resources that were
- 3 spent? They could've been put toward research. With that, I'd like to thank you
- 4 for the opportunity to present.
- 5 CHAIRMAN JACZKO: Well, thank you, and thank you for your
- 6 presentation. We will start with Commissioner Magwood.
- 7 COMMISSIONER MAGWOOD: Well, thank you all for your
- 8 presentations this morning. Some of you I've known for quite some time. Some
- 9 of you I don't think -- I think I actually have met you. You came to visit a while
- 10 back. And, you know, as you know I've visited many of the facilities over the
- 11 years. I haven't kept a tracking list. Maybe I should start doing that and start
- 12 checking them all off so I don't miss anybody. I have not been to RPI, so I have
- 13 to correct that.

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14 It was interesting. I was watching a -- I won't say which network. I

15 was watching a television news, "news," yes for the record he used the quotation

sign, program that was looking at the issue of research reactors and during this

discussion and interview, a government official sort of jokingly indicated that, you

18 know, that we now have to clean up the mess left by the Eisenhower

administration from putting these reactors all over the place, and everyone kind

of chuckled about that. But I was thinking about this and it seems to me that, you

know, the research reactors that were deployed, not just in the United States, but

really worldwide, have provided tremendous benefit for many, many years. In

fact, when you think about the initial investment at the time, to think that these

reactors have been in operation for some of them, in the case of yours, coming

up on 50 years. And when you think about the numbers of people that were

trained in these reactors, it's really quite remarkable and a real success story.

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2 And so we are now at the point where these facilities had to go 3 through another round of relicensing and it's also worth noting that we went 4 through a period during the, I guess you'd say the '90s for the most part, where a 5 lot of these facilities were shut down. I probably -- probably Leo remembers 6 numbers better than I do, but I seem to remember number 67 reactors at one 7 point, maybe like in the very early '90s or late '80s, now down to I think you said 8 25 university-based research reactors. And it seems like that number stabilized. 9 Before I ask specific questions about the relicensing, I just wanted to just turn to 10 Leo since you were sort of the leader here. Have things stabilized in the 11 community? I know there were so many things. You went through the HEU 12 conversions, the security issues, questions about the future of many programs. 13 Do you feel that the situation is stabilized as far as the future of these facilities? 14 LEO BOBEK: They have and they haven't. As you mentioned, in 15 the 1990s we were losing about two facilities a year, probably from 1980 to 2000 16 we were losing about two facilities a year. When the Department of Energy 17 started funding funds for upgrades to these facilities and for research dollars to 18 use these facilities, we saw a slowing down. However, in the last couple of years 19 we have lost some additional facilities. University of Arizona was the Polytechnic 20 Institute and so these facilities are always at risk and a lot of it is funding based, 21 whether or not there are research programs associated with them and whether 22 there's funding to upgrade them. But there has been a stabilization, to answer 23 your question. 24 COMMISSIONER MAGWOOD: Okay. I'm pleased to hear that. 25 Let me start with you, Ralph. You talked about the large number of questions

- 1 that you received. Can you characterize what -- I don't think you gave a
- 2 breakdown of what sorts of questions. What were they aimed at for the most
- 3 part?
- 4 RALPH BUTLER: Most were aimed at just coming back to the
- 5 safety analysis report and trying to better understand the basis that were in
- 6 those. There -- in some cases there's probably six factors that were in our tech
- 7 specs that just numbers that there was no design basis. Nothing written down
- 8 about where did these numbers come from. I mean, it's kind of like the 10
- 9 megawatt number used for the separation of test and research reactors at NRC.
- 10 There's no documentation on where that number came from. So we spent a lot
- of time trying to rebuild, trying to rethink what they were thinking in the early
- 12 1960's about how did you come up with this number? Most of them were not that
- difficult, it's just time consuming in that you have to make sure you get them in
- the correct format, that you're fully answering the question, researching the
- 15 material to find out, you know, do we have sufficient evidence to support that
- 16 answer?
- 17 COMMISSIONER MAGWOOD: Some of what I can figure all of
- 18 you had to say was that there was some degree of pain as you went through the
- 19 license renewal process, but I also had the impression and this is open to any of
- 20 you -- I also have the impression that now that the process is over, you now have
- 21 the completed safety analysis. That's a good thing. Having it in hand however
- 22 you got there is a benefit that will serve both us as a regulator and you as a
- 23 licensee going forward. Is that a fair statement?
- 24 RALPH BUTLER: I know for us at the MU Research Reactor it
- 25 certainly gives us confidence in the safety basis and our goal is to maintain that

1 safety analysis current and up to date so that if we come back in 20 years all that

2 information is there and our successors don't have to come back and try to figure

3 out what the heck were those guys thinking, you know, 20 years ago. So, I think

4 it's very beneficial. I think for the community it's beneficial, so I'm glad we

underwent the process. I think it was important to do that. It likened

reconstituted some of that safety basis and I think it serves the public well.

COMMISSIONER MAGWOOD: Yeah. Others? Any other?

STEPHEN MILLER: Although I think we learned a lot through the process, the question is, you know, back to the initial safety question. Is there -- was there an increment in safety when we haven't changed the facility. We haven't changed what we do there. We haven't changed the design. And all we did was reaffirm what we thought we already knew with calculations that were done with destructive testing in the '60s, perhaps not in a very scientific way, but you know, they did break a few to find out what happens.

COMMISSIONER MAGWOOD: And I appreciate your comments on that and, you know, it's a little late I guess in terms of most of the facilities, but I still think it's worth remaining about, because I think you raise an interesting policy question about how we approach facilities like this. But it's, you know again, a lot of that is water under the dam at this point. One thought. I was listening to Commissioner Apostolakis' earlier dialogue about whether it be useful to have a, I think he used the term, a mini PRA. I think that was the term -- to look at these facilities and I suspect, especially after hearing what you just said, that you know, the idea probably doesn't sound that attractive to you, but I wonder, though, one thing that I think does raise some issues that might be interesting to talk about and maybe this is more -- this might be something we

- 1 can look at. It might be more of a DOE activity, but it might actually be an
- 2 interesting project for graduate students. Because I think there is a need to
- 3 enhance the level of PRA expertise nationally and there might be some
- 4 opportunity there. So I thought I'd just sort of put that idea on the table for others
- 5 to consider, because I do think that there's a value in the exercise. Whether
- 6 there's value in the product is -- let others debate, but I think the exercise itself
- 7 would actually be something worth considering.

Now you heard Mr. McGinty at the end of his time with me indicate he thinks that the balance is now about right. And I recognized this process has gone on for some number of years and there's been a lot of back and forth with the agency, but I did hear almost all of you, I think, indicate that you felt the staff was -- had been very professional and very fair and very flexible in how they dealt with you. And I wanted to give each of you a chance to react to the -- to his conclusion at the end that the balance is about right now, after recognizing that there's been a lot of things that have happened, but where we are today, do you feel that the balance is right? And maybe all of you just give me a quick answer to that.

STEPHEN MILLER: The -- as I already stated, the NRC has been very, very nice to deal with. Everybody's been professional and so nobody can say that somebody's come down on us with, you know, with a brick, but for these types of processes, I'm not sure that we really are striking a balance at this point between the actual risk to the public for the facilities versus the amount of regulatory oversight. We had talked about aging issues a little bit and for a facility like the one that we have at AFRRI, there is nothing that isn't replaceable. There is nothing that can't be serviced. There is nothing that's going to wear out

1	and irreparably damage the facility. The question might be is it cost effective?
2	Do we need to continue operations if it's going to cost this much money to fix a
3	tank? You know, those decisions will be made, but as far as aging, there's very
4	little risk to the public for the facility and there's nothing that can't be replaced,
5	unlike a power reactor. So, the purpose of a probability risk analysis when you
6	just don't have the source term to reach outside of your walls would seem to be,
7	although very interesting, maybe not from a regulatory standpoint, necessary.
8	COMMISSIONER MAGWOOD: Well, my time is up. If maybe Leo
9	you could just give a quick response to the balance question.
10	LEO BOBEK: The only other point I would make is that, again, as
11	far as a balance for these smaller facilities that have to put in significant
12	resources to doing some sort of thermohydraulic or hypothetical analysis, that it
13	would be beneficial to have a generic analysis that would be available to them
14	that they could use to simplify the process. Now again, as you mentioned, it's
15	sort of water under the dam. We won't have to do this again for about 20 years,
16	but it is something to consider.
17	COMMISSIONER MAGWOOD: All right. Thank you. Thank you
18	very much. Thank you, Chairman.
19	CHAIRMAN JACZKO: Commissioner Ostendorff.
20	COMMISSIONER OSTENDORFF: Thank you, Mr. Chairman.
21	Thank you all for being here today. I think Commissioner Magwood has really hit
22	on a very critical aspect and I want to follow up and tag team on some of his
23	questions and comments. It is helpful for us to hear in your briefings the impact
24	in answering various RAIs has had on time periods, et cetera, and I think our

staff has been pretty candid in the first panel about okay, here's where we are,

here's where we were in the past. I think you have been as well and I commendyou for your candor.

I'm going to try to ask a question that, you know, in a little bit different way than Commissioner Magwood posed, but I think it might help us -- me understand a little bit better what you think of the current practice by the NRC staff. The question I'd ask each of you to respond to is, and I kind of whispered over to Tim a few minutes ago, Tim McGinty, to ask a question. I think the standard review plan that is in use right now is what Ralph, your organization was responding to for these questions and I think Steve had the same experience. And I don't know, Sastry, if you've had the experience with the standard review plan or not.

But let's just put a different hat on. Assume that you were where you are right now with respect to your individual facilities and that you are coming up for initial licensing, not a renewal, but initial licensing at a research test reactor in your current organization. Do you think the standard review plan, the types of things the NRC staff is looking at for initial license are about right going back to the balance question that Commissioner Magwood was asking? Or is the agency far off the mark? And I'm trying to ask this from a standpoint that a different way was used 20, 30, or 40 years ago to look at these things and now something -- now we know a lot more, trying to fill in some gaps of knowledge. So, does that question make sense to you, Leo?

LEO BOBEK: I'm going to pass that on to my colleague since I have not relicensed and I haven't used NUREG 1537.

COMMISSIONER OSTENDORFF: Okay.

25 RALPH BUTLER: I believe it was beneficial. It did feel, you know,

•	that we were doing an initial license, the level of effort and the rigor that went into
2	it was felt like we were applying for an initial license. I'm not sure that how
3	you would change that level of rigor in today's environment to go back to the old
4	days of just a few list of questions because the standard review plan did provide,
5	you know, a methodology of going in and asking questions of pertinence, so I

6 think it's okay, I just -- the level of rigor was a heavy burden, but I think the

process was probably suitable.

COMMISSIONER OSTENDORFF: And one of the motivators from our question was, it's my sense in talking to staff in prior sessions before this one is that there were some significant gaps in being able to document the design basis, the safety analysis of various facilities, so part of their effort was to go through in a comprehensive manner and to capture these issues, recognize that years ago it was not.

RALPH BUTLER: Yeah, and it served that purpose, you know, there was gaps, there was documentation or the couch missing from some of the numbers used back in the '60s, and so it served that purpose well, to go back and have to reconstitute that, so otherwise, you know, we would have continued on without that information. Now, did it really make a difference, as Steve says, in the safe operation of our facility? No, but it was good to have that information, to have it documented, so it's there in case we ever need to answer that question again.

COMMISSIONER OSTENDORFF: Okay, Sastry?

SASTRY SREEPADA: I think one of the -- one of the things I realized was yes, it is complicated, we're loaded down. Now, when there is a flood like Irene, you get questions from the community. They can always point

1 and say, "This is the basis on which we are safe." That	: makes	people
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- 2 comfortable. I know it is a pain when I had to answer the question, but I have a
- 3 basis to say NRC looked at it, we looked at it, this is the worst thing that can
- 4 happen. That normally guiets people, so basically it is not the safety that is
- 5 improved, it is the public relations and the security because we monitor now the
- 6 facility 7/24, so those are the things that are changed, and my future students
- 7 can look at this and say, "My experiment falls into this chapter." That's the thing
- 8 we accomplished, and we were lucky to have the people help us from part-time
- 9 employees from Notre Dame which helped us a lot, basically.
 - COMMISSIONER OSTENDORFF: Thank you. Steve, did you
- 11 want to --

- 12 STEPHEN MILLER: If for an initial license, given that, A, we're not
- 13 going to be cracking a few open to find out what happens in today's day and age,
- the level and types of questions that were asked, I think, would be reasonable,
- 15 but once again, you know, for a relicensing where the operation has been
- sustained for so many years, I think we've already proven the safety of the
- 17 facility.
- 18 COMMISSIONER OSTENDORFF: I appreciate your response,
- and I think it's -- when one's in a nine inning baseball game, if I can use that
- analogy, and the rules somewhat change the fourth, fifth, or sixth inning
- 21 depending upon where you are, there'll always be these questions about well, it
- 22 was not done this way, you know, for the last game we were in, now it's -- we're
- in this game, and it's -- things have changed, but I appreciate your comments,
- 24 Steve from your experience that you thought the questions were fair, but you did
- 25 not necessarily see any safety enhancement of the facility as a result of this

- 1 exercise, and I can reconcile both of those statements. Recognize that we're
- 2 trying, as an agency, to go back and fill in gaps, and provide a more
- 3 comprehensive approach that perhaps, in an ideal world, would have been done
- 4 at initial licensing, it just wasn't done.
- Not a question, but I'll just comment that as the staff goes forward,
- 6 and John had talked about that with the regulatory analysis that's being
- 7 evaluated, and if there is a -- if there is a rulemaking that comes up in the future, I
- 8 think you'll have a great foundation to provide substantive helpful comments to
- 9 the agency if we do preserve rulemaking.

Leo, I want to go back to a comment that Ralph made on his slide eight. This deals with knowledge management and databases, and so forth, and Ralph had a very, I think, constructive comment about lesson learned database that might help with common design features, and seems like also, from a knowledge management standpoint for individual facilities, would be faculty or students, and NRC staff, there's some attractiveness to this. Do you have any comments or thoughts on that suggestion on Ralph's slide for the broader community?

LEO BOBEK: Once again, like the power reactor community, a lot of these reactors are different in their design, and the way they are configured. RPI is much different from Ralph's facility, and my facility is much different from either one of them. However, there are commonalities. This MTR flat plate fuel, this trigger fuel, so there's certain analyses that can be done that are generic to those, and again, safety significance associated with fission product releases, that can also be generic. When you get to the individual components of whether or not you have a certain pump for a 10 megawatt reactor versus a 250 kilowatt

1 reactor is going to differences, so there's only so much you can do.

2 COMMISSIONER OSTENDORFF: Okay. Any other comments on

- 3 that? Okay, again, thank you all for being here today, and for your presentations.
- 4 Thank you, Mr. Chairman.

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5 CHAIRMAN JACZKO: Commissioner Svinicki?

COMMISSIONER SVINICKI: I want to thank each of you for your presentations as well, and also for the support for your institutions, and the involvement you've had in the public meetings, and I'll begin by platforming off what Commissioner Ostendorff said, I hope that if the recommendation is that NRC move to make some modifications to our regulations in this area, I think that that record could be very beneficially informed by your experiences, so I hope that your institutions will continue to stay involved in this process, and I appreciate your candor as well. I've met some of you privately, and I expect to know less, of course, I know you're all very candid and forthcoming individuals, but I realized, as I listened to your presentations, that I was imprecise in the first round because I think I slipped between relicensing and renewal, and I know that that's a core, really, of a philosophical question at the heart of is this a relicensing or is -- we don't have what we would formally call, you know, a renewal process for RTRs that is of a different scope, so I know in the past it's probably been more ad hoc.

We heard from the NRC panel they're attempting to have a much more disciplined and documented process going forward, but moving into long-range thinking, of course that becomes a question of whether we should have something akin to the license renewal process that we have for power reactors, and again, there's a change or a limiting in scope there from initial licensing. I

the analogy with just aging management, isn't really probably an easier, maybe appropriate fit here in terms of how you would have a true license renewal for RTRs going forward, and of course, as we referenced earlier, it's really an investment in the future because this would be something that be in place for

think, from what I've heard from the NRC staff, though, in the first panel is that

those seeking relicensing or renewal of sometime in the coming decades. So

again, I think your continued insights and involvement in our public workshops

and public comment processes on that will be very helpful.

The other thing that I took from some of the Q and A with my colleagues, and also your presentations, was that I think most of you commented on the professionalism and how it was -- in working with the NRC staff, I have no doubt that they were very pleasant through the process, but I would use the term reasonable as being very different than being professional and easy to work with, and I thought of a question about RAIs that any of you have received. You mentioned, you know, day-long meetings where you would talk about what was intended, and what would likely form an adequate response to particular RAIs. Did any of you ever have an experience, though, where in talking about what was being asked for, and what was being expected, that there was an agreement to limit the scope of an RAI?

What I'm trying to get to is when you receive RAIs, did they look as if someone had done a reasonableness check? Again, three years ago, I had a sense from some of you or your predecessors that we had contractors working on these reviews, and I know in the case of one university, they asked questions about a reactor technology that we don't even have, so you know, did they -- did this contractor look at anything we submitted because, you know, it was almost

1	as if they just had a template that they were sending around, but I'm getting kind
2	of beyond the, you know, appropriateness of RAIs to a sense where, again, I
3	guess, Steve, and this gets to something you mentioned, if you're going to GA to
4	get four three or four RAIs answered, if that cost estimate comes back, you
5	know at \$5 million or something to do some analysis for you, do you feel that the
6	NRC's process allows you to come back and go, hey, you know, this is going to
7	just break the bank, could we agree what do you really need to know to make
8	your safety finding. So I'm trying to get to kind of a pushback on RAIs to say
9	you're asking for an awful lot and I don't doubt that you would find it informative,
10	but what do you really need?

STEPHEN MILLER: Those questions were never asked, and --COMMISSIONER SVINICKI: I guess we discovered another way you differ from the power reactor community if you didn't ask those questions, so

STEPHEN MILLER: No, I mean, we weren't provided an opportunity to say, you know, no, we really don't want to do this because this is going to cost us \$100,000 and what are we getting out of it? The questions were asked, we -- the NRC was open to discussion as to whether or not the questions were relevant to our facility, nobody asked us questions about the steam generator that we don't have. So from that perspective, everything that came through the screening process from the contractors through the NRC to us were relevant to our facilities, absolutely, but we were expected to answer the questions. The thing that was negotiable is whether I have a day -- 45 days, and when I told them two years, they said, "What do you mean two years?" I said, "Well, you know, we've got to appropriate money."

1	COMMISSIONER SVINICKI: Well, and there's been mention
2	made, I think on both panels, of the Atomic Energy Act, you know, information
3	then when I interpret the purpose of that historically it is, as I think Commissioner
4	Magwood was saying, that there is basically science, kind of, in the public
5	interest here, meaning that the original purpose was that people would have
6	research reactors, and that the regulatory footprint on that needs to be balanced
7	against the fact that these research reactors both advance the state of science,
8	and they train the future generations without which you can't have future
9	regulators, and you can't have future operators; so that appropriate balance
10	needs to be struck, and a number of my colleagues have asked about whether
11	we're striking that appropriate balance.
12	I might ask if any of you are aware of any facilities we also talked
13	about the decline in the number of licensed and operating research reactors. Are
14	you aware of universities and institutions that felt that the process of relicensing
15	was so expensive or so daunting that it had at least some strong or dominant role
16	in decisions to discontinue operating? Could you sit here and represent that you
17	think that – is there's some likelihood that occurred, or you know for a fact that
18	occurred?
19	LEO BOBEK: I can answer just for one facility. I know, as far as
20	Worcester Polytechnic Institute, it wasn't the major factor, but it was a
21	contributing factor in them deciding to shut down that facility.
22	COMMISSIONER SVINICKI: Okay, thank you for that. I think the
23	other and I'm not sure I have a question here, but we've talked about what was
24	the safety benefit of some of the you know, if it cost a million dollars to be
25	relicensed I would just say that I think, given the risk profiles being so low, it

- 1 might be very, very difficult for us ever to establish what was the safety
- 2 improvement or benefit. I would say, and I think some of you have substantiated
- 3 this, it's my sense that maybe it isn't so much safety improvement as it is
- 4 defensibility. I will say that as the safety regulator, and I think that Sastry was
- 5 talking about, you know, as being in a community and having a facility, it's the
- 6 same thing from a different vantage point, but there does need to be just some
- 7 level of defensibility of the conclusions that we've drawn, and as painful as the,
- 8 you know, finding the documentation or reconstituting it or getting it in place, I
- 9 think at the end of the day, maybe that didn't improve a safety margin in any way,
- but I do think that it has provided the fundamental, and frankly, essential level of
- 11 defensibility for both our safety conclusions and your operational conclusions
- 12 about the safety of your facility; so I do think it provides that benefit, and Sastry I
- 13 appreciate you mentioning the flooding event, and the needing, you know, to be
- able -- that it is a good place to be to be able to say this an analyzed situation
- 15 and we can confirm that this is the worst possible consequence that would occur
- if we had that level of flooding.
- So, again, I thank all of you, I hope -- I think your experiences are
- very valuable, and please don't undervalue the specifics you've given because I
- will say, as a Commissioner, I hear a lot of generalities about processes, but to
- 20 hear that you had, you know, this many RAIs broken down in this way, it's very,
- 21 very helpful for me to be able to understand the process. So thank you again.
- 22 Thank you, Mr. Chairman.
- 23 CHAIRMAN JACZKO: Commissioner Apostolakis.
- 24 COMMISSIONER APOSTOLAKIS: Thank you, Mr. Chairman. I'll
- come back to a question I asked the staff earlier. The whole issue, when we are

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- 2 mentioned aging once. Mr. Miller, you said that you had 50 years of -- I mean,
- 3 your reactor, has had 50 years of safe and incident-free operations. Did you
- 4 have any aging problems during those 50 years? Did some structure or
- 5 component degrade due to some mechanism and you had to replace it?

see well from 20 feet away through ripply water.

STEPHEN MILLER: Structures? No. In fact, in 2004, I was part of some routine maintenance just to replace ball bearings for our lead shield doors and those sorts of things. We put a team of Navy divers in the pool, and they took a very close look at the walls of the tank, places where you really couldn't

As far as electronics, instrumentation, those sorts of things, absolutely. We are on our third or fourth console now; we've replaced all of our instrumentation multiple times over the years. The question is not so much aging management, it's just whether it's cost-effective, and we've been asked that before. You know, we're in the process, we've been funded for a new console currently, and everybody sat down around the table and said, okay, do we want to continue if this is going to be the bill for the console? But it was never a question of whether the facility is capable of sustaining another 20 or 40 years of operations, it's -- you know, it's a pool with 13 feet of concrete in all directions, so the thought of the facility having structural problems never has come up. It's not in the design.

COMMISSIONER APOSTOLAKIS: Mr. Butler?

RALPH BUTLER: If I may, from a research reactor perspective, we do consider the aging management issue. Every eight years we tear down the reactor to replace the beryllium reflector. That means we disassemble the whole

1	reactor, it's a very modular reactor, it comes apart, it becomes back together.
2	That only takes us a week to do that. During that timeframe, every eight years,
3	we go in and we hire engineering firms to come in and inspect the pressure
4	vessel. Even though we do we have a spare pressure vessel sitting on a shelf,
5	so there's really no reason I mean, this reactor wouldn't operate for another 60
6	years, but we have them come in and look at all the welds and the piping, and so
7	forth on the primary coolant system, and so we have that all documented. We
8	have testing companies come in and do all that. On the pool liner, we've had
9	engineering firms come in and video and inspect the entire pool liner because
10	that is an issue. We don't want to have any leaks in the pool liner, so it is a
1	matter of aging management. How is the facility aging, what's the condition of
12	some of the critical components that provide that barrier of safety? So we do
13	look at some of those things.
14	COMMISSIONER APOSTOLAKIS: Okay, good, thank you. Yeah?
15	SASTRY SREEPADA: I think aging-wise, structurally I don't see
16	many problems.
17	COMMISSIONER APOSTOLAKIS: Your reactor is so small, it's
18	growing up, it's not
19	SASTRY SREEPADA: Yes, but the other issues the other issues,
20	the control drive mechanisms.
21	COMMISSIONER APOSTOLAKIS: Yes.
22	SASTRY SREEPADA: It is going to fail. My power channel failed
23	last year, and we were down for four months to fix it, but these things come out of

COMMISSIONER APOSTOLAKIS: But did the regulations ask you

the surveillance program and regular checks.

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1	to	do	anv	of	that?

- 2 SASTRY SREEPADA: The surveillance program demands that as 3 a part of my relicensing, so it is --
- 4 COMMISSIONER APOSTOLAKIS: So during your relicensing, it's
- 5 not --
- 6 SASTRY SREEPADA: Yeah.
- 7 COMMISSIONER APOSTOLAKIS: -- license extension. Thank
- 8 you.

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addressing how that's done.

- 9 LEO BOBEK: As far as the majority of the non-power reactors, 10 you'll find that aging issues are associated with replacing components that are 11 failing or failed over time, for example chart recorders, and smaller components 12 like that. The smaller facilities don't have the neutron fluxes that create 13 embrittlement of materials, so that's not a concern. One of the major concerns, 14 though, non-power reactors have had over the decades, is leaking tanks. Most 15 of the tanks are lined, either with stainless steel or aluminum or concrete, and 16 they tend to develop small leaks over time, and these are being addressed by 17 everything from replacing the tank liners with stainless steel to putting in epoxy 18 fillers to fill in the leaks. I would say that the staff is addressing some of the aging 19 issues right now, the RTR staff in that a lot of the upgrades that these facilities 20 are doing are digital upgrades, replacing various components with digital systems 21 that were analog previously, and so we are working with the RTR group, and
 - COMMISSIONER APOSTOLAKIS: Thank you. I'm sorry.
- STEPHEN MILLER: Over the past 50 years we have replaced our coolant systems, all of the piping, cooling towers. Everything short of the actual

- 1 structure that the facility sits in has been looked at and replaced with regularity.
- 2 You asked if it was in our license, it is in our technical specifications and our
- 3 surveillance programs. We measure and look at our fuel each element every
- 4 year. So all those things are being addressed, but the actual structure is sound.

5 COMMISSIONER APOSTOLAKIS: Okay. Now, I would just finish

6 by saying that I fully support Commissioner Magwood's suggestion. This would

be, I mean, getting a few students trying to do a PRA, you don't have to follow --

we're not going to review it with -- pick up the ASME and ANS standard and use

it as a way to train students how to do a real PRA with a small reactor, and that

would be very useful, it seems to me, especially if you have courses in PRA. I

don't know whether RPI has one or Missouri, but that would be a good exercise,

a good exercise, without the burden of submitting it to us. Thank you, Mr.

Chairman.

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CHAIRMAN JACZKO: I just had a couple questions. One, the, I think you, Mr. Bobek, you brought up the issue of, concerning the development of a generic thermo hydraulic analysis, and -- which I think is a good idea. I don't think we can do that. It's probably DOE, I would think, would be the right person or the right place to do that, or, you know, maybe you all get together as a community and pool some money or TRTR does it. I don't know if you have thoughts of who would do that.

LEO BOBEK: Well, it would certainly have to be funded at the government level. It would have to be the Department of Energy that would fund something like that, but we would have to make sure before we appropriate that funding that the NRC would accept a generic analysis from many of these facilities.

1	CHAIRMAN JACZKO: I know we can probably pursue that
2	question, but my thought would be, I mean, it's principle. You should be able to
3	submit something. I mean, it's kind of a topical report, you get it reviewed, and
4	then it's approved, and then could be relied on. So, I think that should be
5	something that would work, but I, obviously, as I said, I think somebody else
6	would actually have to do the analysis, and then we would review it. But, we can

pursue that, I think, for sure. I think that makes a lot of sense.

The -- since I'm getting back to a question, I think, you raise this, and I think Commissioner Ostendorff touched on this, the issue of the database, which I think, again, is a good idea. And I, you know, I would think that perhaps this is a good activity that TRTR could do, because then, again, I don't know that we necessarily need to do that, but you all have, I mean, you all have all of the questions. You all have your responses. So, the information is out there, it's just a question of cataloguing it and publishing it and making it available. And, so, you know, I would, I think, again, I think it's a very good idea, and, you know, again, to the extent that that's easy for us to do. I think it's, you know, if we have those, we could, but again, some of those, I don't know to the extent that there may be some issues that are, well, may be of some proprietary issues, maybe one or two of the research reactors could fall in that category. But, you know, again, I would think that that's something that's out there that could be leveraged with some of your existing resources, so I think those were all good suggestions.

Mr. Miller, you mentioned, I think, that you did a licensing basis reconstitution in 1980 if I remember what you said, right? Why didn't that take in 1980? Did things, did our analysis, methodologies, evolve enough that that just wasn't valid anymore by the time you got around to this time, or --

1	STEPHEN MILLER: It's not that it wasn't valid, really, it was more
2	format and lack of some content that was when the new 1537 came out in the
3	'90s that we just didn't have. So, in fact, much of what we had is being used, but,
4	you know, I brought that up because a statement was made that we don't keep
5	our SER's up to date. We don't, and, in fact, some of us do.
6	CHAIRMAN JACZKO: Well, that's helpful, and I appreciate that.
7	So, in hindsight, probably, if you'd done that in 1995, then you may have been
8	better off this time around.
9	STEPHEN MILLER: The question would be why would we have
10	done it in 1995
11	CHAIRMAN JACZKO: Yeah, well, as opposed to 1980.
12	STEPHEN MILLER: But nothing had changed.
13	CHAIRMAN JACZKO: Yeah, but as you'd done it in 1995 versus
14	1980, I mean, I'm trying, you know, one of the things we're hanging our hat on, I
15	would say, is that we are going to not have to do this again in a couple of years.
16	But, if you had experience with an updated, a voluntary ultimately updating SAR,
17	but it didn't really get you where you needed to go. So, I guess the differences
18	when we put out the guidance in the '90s, that kind of gave a better template then
19	for going forward. So, as long as we're consistent with that guidance in 15 years,
20	then the SAR should be useful again.
21	STEPHEN MILLER: And I will concede that, you know, to state
22	that they broke one apart in 1960, and we don't really know why it behaved the
23	way it does, but, by god, it does. It probably wouldn't, it wouldn't pass public
24	muster today.

CHAIRMAN JACZKO: Well, that's helpful. The last question I just

- 1 ask you all, I mean, we are embarking on this activity to consider doing a rule
- 2 change, and I would just ask all of you, do you all want a rule? I mean, if, do you

RALPH BUTLER: Well, my suggestion is we've undertaken this

- 3 want, do you see value in a new rule? I mean, if you all don't and we've been
- 4 able to do what we've done now, I'm not sure that there's a reason to do it.
- 5 Here's your chance, okay? Speak up now or --

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7 relicense or new license activities to create these safety analysis reports. To 8 think somewhat differently, if the Commission was to require the community --9 and my colleagues may slap me here -- but to maintain the safety analysis 10 report, to develop an aging management program, why undergo a 20-year 11 license renewal process? Where does it say you have to go in every 20 years 12 and do the relicense, especially for the university research reactors? If we are 13 maintaining, if we are required to maintain our safety analysis reports current and 14 up to date, maybe a two year submittal of changes to the facility, an update on

aging management from time to time, then why undertake another round of relicensing of these, you know, university research records, when we are all pretty confident that there's no additional safety margin to be gained, there's no additional protection of the safety and health of the public if we maintain our safety analysis reports and our aging management program current, so.

CHAIRMAN JACZKO: So, I would take that as you want a rule change, because I think we would have -- that would require a rule change. Or, no. Yeah. That would require a rule change. So, that's a yes for a rule change. Anybody else?

- 24 STEPHEN MILLER: Yeah. I would agree with that.
- 25 CHAIRMAN JACZKO: You'd want a rule change?

1	STEPHEN MILLER: It's been suggested, in fact, that on many
2	levels, as we all talk about what we're doing that perhaps an enhancement to the
3	inspection program that would include those things that a relicensing gets you
4	would be adequate, and just keep going until its, the facility's no longer needed.
5	LEO BOBEK: I've been involved with this process for 25 years. I'm
6	always wary of rulemaking. However, I will echo the sentiments of Steve and
7	Ralph. I think for a majority of the facilities, it would be beneficial. Many of these
8	smaller facilities, it has been a very difficult process relicensing, and so therefore
9	if we could eliminate the relicensing process by just keeping an updated FSAR
10	process, I think it would be very beneficial.
11	CHAIRMAN JACZKO: So, Trip, is there a statutory requirement on
12	the 20-year
13	TRIP ROTHSCHILD: Well, no, but you've got, we've never issued
14	licenses beyond 40 years, you know, we haven't issued licenses beyond 40
15	years, as you know, and the power reactor renewals have been for 20 years
16	increments. And, you know, I think
17	CHAIRMAN JACZKO: But, so, we don't, the statutory
18	TRIP ROTHSCHILD: I think you've probably got some discretion if
19	you wanted to change the time limits, I would think.
20	CHAIRMAN JACZKO: To an indefinite timeline, essentially?
21	TRIP ROTHSCHILD: I don't know whether we've gone beyond 40.
22	CHAIRMAN JACZKO: In the spirit of getting out in front of all of my
23	headlights, you know, why don't you all send us a petition for rulemaking that
24	says that, and then we can get in the process of looking at a specific proposal. I
25	mean, I can't tell you we'll agree with it, but, you know, you have to make that

1	calculus whether it's	cost-beneficial for	vou to ao throuah	n that, and	. but it ma
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- 2 give us something specific to focus on as a particular rule change and go from
- 3 there. Any comments?
- Well, again, I appreciate this, and I think it's been very helpful. It's
- 5 always an interesting discussion. Oh!
- 6 COMMISSIONER MAGWOOD: Just, first, let me second your
- 7 suggestion for a petition. I think that's probably a smart way to proceed so they --
- 8 we can see what you think needs to be done, and then we can react to that. I
- 9 think that's a good idea. Also, I wanted to pick up on your line of questioning
- 10 about the generic items that could be analyzed. I wonder if it would be useful to
- 11 have staff provide maybe a CA note to give us their thoughts as to what that
- 12 process might look like you, I'd like to sort of understand how we would receive
- 13 something like that. Thank you.
- 14 CHAIRMAN JACZKO: Sure. Okay. Well, thanks, and, yeah,
- 15 again, thanks everybody for being here. I think it was very helpful, and as
- 16 Commissioner Svinicki said when -- things have certainly gotten a lot better since
- 17 2008, and I think, you know, and as you've said, the staff has done very well in
- the work that they've done, and so we'll continue to plug away. Thank you.
- 19 We're adjourned.
- [Whereupon, the proceedings were concluded]