



U.S. DEPARTMENT OF
ENERGY

Nuclear Energy

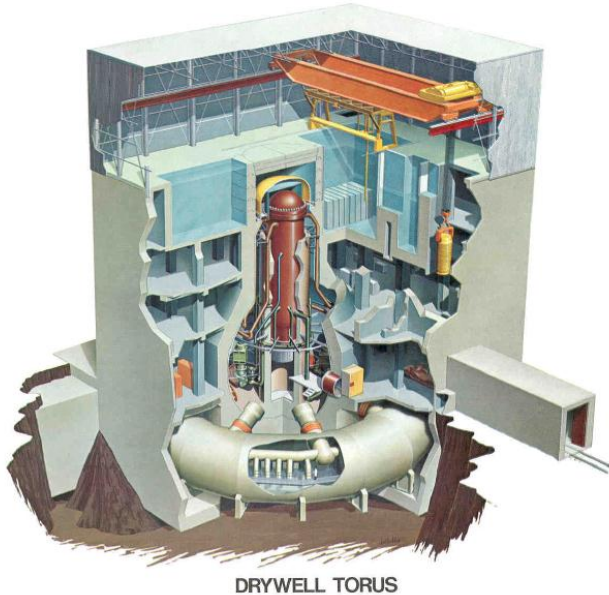
Severe Accident Analysis Research Based on Fukushima

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Joint DOE-NRC MELCOR reconstruction of Fukushima Daiichi Accident



■ Scope of Effort

- Collect and archive data on accidents (portal)
- Reconstruct accidents
- Assess validity of MELCOR models

■ Sandia-led effort with INL and ORNL

■ SOARCA Mark-I MELCOR model used as starting point to reconstruct accidents

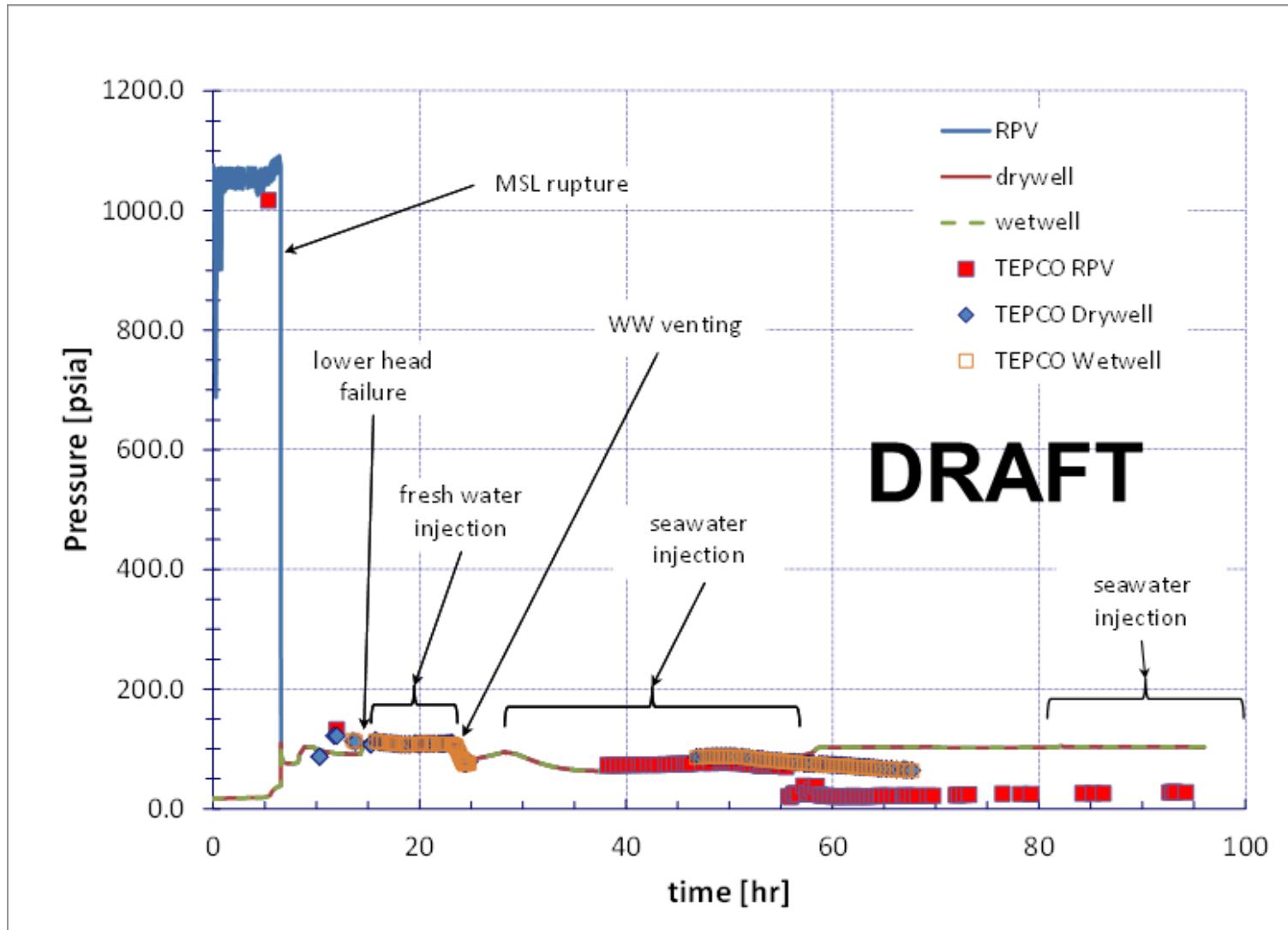
■ Collaboration with and peer review by

- NRC, JNES, TEPCO, EPRI

■ Report published in August 2012



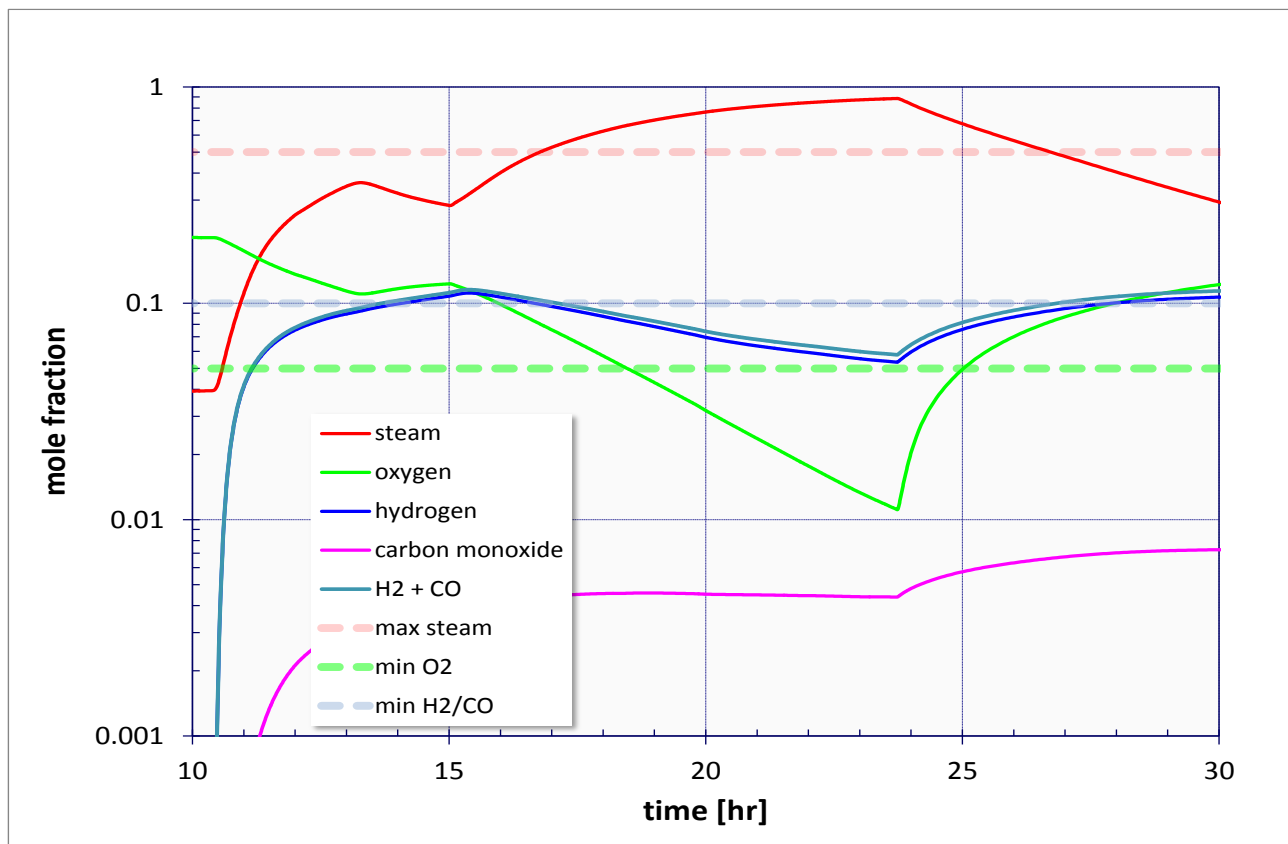
Comparison of MELCOR Unit 1 Analysis of RPV and Containment Pressures





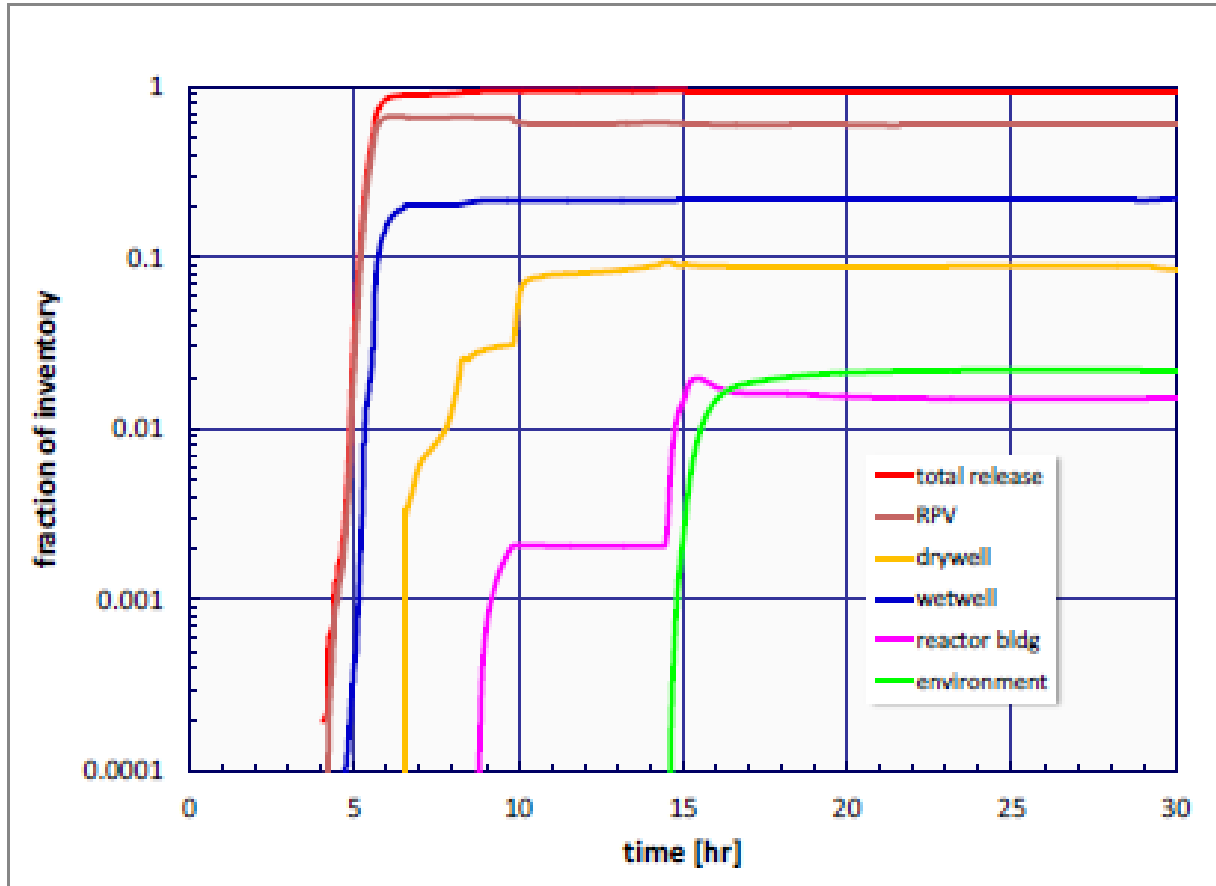
Unit 1 Results – Refueling Bay Vapor/Gas Molar Concentrations

Gas Composition in Refueling Bay





Predicted Cesium Release at 1F1



Radiation monitors at front gate jump at ~15 hr

Cs Environmental release about 400,000 Ci

Figure 49 MELCOR-predicted CsI distribution – note that release to environment from reactor building explosion is not reflected in this distribution.

MELTSPREAD & COREQUENCH

Best-estimate Ex-Vessel analysis of Unit 1

- **1F1 ex-vessel behavior using best estimate codes for spreading (MELTSPREAD) and debris coolability (CORQUENCH)**
 - MELCOR and MAAP melt pour conditions used as input
 - MELTSPREAD calculates extent of spreading and shell heat-up
 - COREQUENCH evaluates coolability of debris
 - Predictions support TEPCO planning for 1F1 disassembly
- **Principal results:**
 - Axial ablation of the concrete below the reactor vessel predicted to be up to 60 cm out of a total thickness of 140 cm in the most extreme case
 - While there was significant concrete ablation the debris was coolable for all scenarios



NEA Fukushima Daiichi Project

- **NEA is organizing an international project on Fukushima**
 - Phase 1 is a code benchmarking study of Fukushima accident
 - Phase 2 would be to gather data as reactors are defueled
- **In Phase 1, numerous severe accident codes are being used to reconstruct accident and then cross-compared (MELCOR, MAAP, SAMPSON, SOCRAT, ASTEC)**
- **Objectives of this study**
 - Improve severe accident codes via benchmarking with actual data
 - Use code results to guide planning of defueling operations
- **Participating Countries Include:** US, Switzerland, Spain, France, Russian, Germany, Korea, France & Japan
- **Phase 2 is under discussion**
 - Program similar to post-TMI project is being considered
 - DOE conducting uncertainty quantification study to aid



DOE December 2012 Visit to Fukushima

DOE team on top of Unit 4



Kelly peering into Spent Fuel Pool



Aftermath of H₂ explosion

