NRCR00011



Protecting People and the Environment

## **Presentation to the Commission**

# Combined License Application Review Vogtle Units 3 and 4

**SER Panel 2** September 27 – 28, 2011



United States Nuclear Regulatory Commission

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#### Chapter 3, Design of Structures, Systems, and Components (Continued) September 27 – 28, 2011

# **Seismic Design and Analysis**

#### • Issue:

- Vogtle site's Ground Motion Response Spectra (GMRS) exceed the AP1000 DCD Certified Seismic Design Response Spectra (CSDRS)
- Applicant asked to perform site-specific analysis to demonstrate structures, systems and components (SSCs) will remain functional under design-basis demands

### Resolution:

- Applicant:
  - Performed 3-D soil structure interaction (SSI) analysis for instructure response (per AP1000 DCD)
  - Used NI 15 system for analysis of SSI

# Seismic Design and Analysis (cont'd)

### Resolution:

- Staff:
  - Compared in-structure response spectra to the DCD at six (6) key locations
  - Performed comparisons which showed that:
    - $\,\circ\,$  Above 1 Hz, there were no exceedances
    - $_{\odot}$  Below 1 Hz, minimal exceedance was in 0.55 Hz range
  - Found exceedances not to be safety significant because there are no SSCs with resonant frequencies in this range
  - Reviewed Vogtle's application of 4% damping instead of 5% damping in design of SSCs, and verified response appropriately accounted for stress and strain levels in the design
  - Confirmed Vogtle's validation of computer models used
  - Assured that concurrent changes to design and models of shield building were consistent with Vogtle model
  - Concluded design of SSCs is acceptable

# Limited Work Authorization for Foundation Base Slab (Basemat)

#### Issue:

 The applicant submitted LWA request, which includes rebar and embedded items (i.e., reinforced steel, piping in basemat, concrete placement, etc.) in the basemat.

### Background

Basemat design in accordance with ACI 349 (same as AP1000 DCD).

## Resolution:

- Staff found that based on site-specific seismic demands and use of ACI 349, the basemat has significant strength, stiffness, and ductility.
- As a part of the AP1000 standard design review, the staff found the detailed design of the foundation base slab and detailed construction to be acceptable.

# **Squib Valves**

- Squib valves used in AP1000 as follows:
  - Automatic Depressurization System to reduce reactor pressure to allow gravity-driven system to provide cooling water to reactor core
  - Passive Core Cooling System to provide reactor cooling water injection, natural recirculation from containment sump to reactor coolant system, and higher containment water level if necessary
- AP1000 DCD specifies use of ASME Standard QME-1-2007 for qualification of safety-related valves, which is accepted in Revision 3 to RG 1.100
- AP1000 DCD includes ITAAC to confirm design-basis capability of squib valves
- NRC staff is monitoring design and qualification process and development of surveillance provisions for Vogtle squib valves
- NRC staff plans to conduct inspections in support of ITAAC closure process for squib valves to be used at Vogtle Units 3 and 4
- NRC staff will review ASME OM Code improvements for new reactors (when completed) including squib valve surveillance requirements

# Squib Valves (cont'd)

- Consistent with SECY-05-0197 for COL operational programs, NRC staff reviewed description of the Vogtle IST program in COL application
- Vogtle FSAR requires that the squib valve IST program incorporate lessons learned from design and qualification process such that surveillance activities provide reasonable assurance of operational readiness of squib valves
- Based on 10 CFR 50.55a and Vogtle FSAR, NRC staff found reasonable assurance of operational readiness of squib valves to perform their safety functions
- Prior to startup, NRC staff will inspect the Vogtle IST program to confirm that squib valve surveillance activities incorporate lessons learned from design and qualification process



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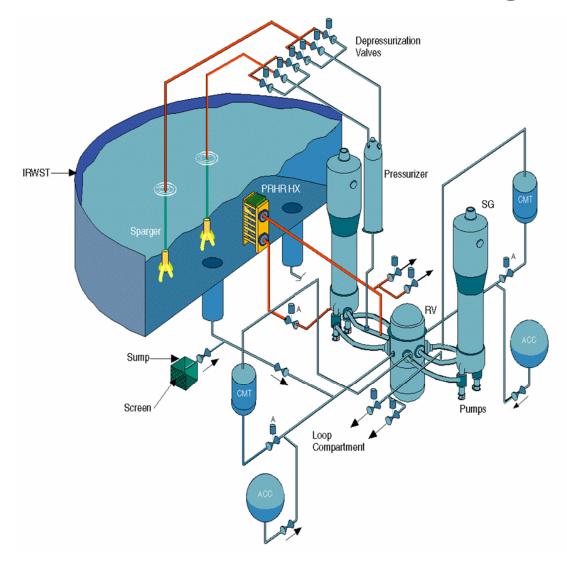
## **Presentation to the Commission**

# Combined License Application Review Vogtle Units 3 and 4

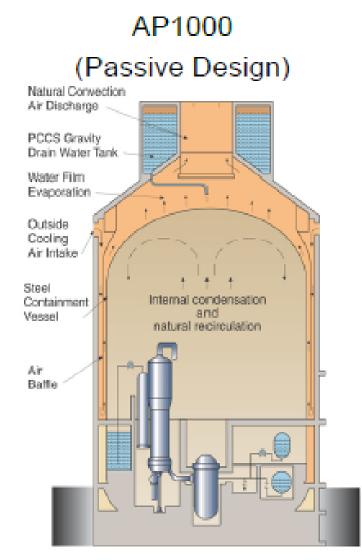
**Chapter 6, Engineered Safety Features** 

September 27 – 28, 2011

## Information Incorporated by Reference: Passive Core Cooling



# Information Incorporated by Reference: Passive Containment Cooling



# Information Incorporated by Reference: Containment Sumps and Pressure

## • Containment sump (GSI-191)

- AP1000 design attributes ensure very low debris source term
- Key design attributes are verified through ITAAC
- Strainer and fuel assembly head loss tests were performed to demonstrate adequate core cooling
- Debris limits designated as Tier 2\* in AP1000 DCD

### Containment Pressure Analysis

- Errors in existing Westinghouse analysis discovered after submittal of Rev. 18
- Revised analyses submitted for staff review
- Limited credit taken for gratings as heat sinks (Tier 2\*)

# Information Incorporated by Reference: Control Room Habitability

### Control Room Habitability Design

- Design has an active non-safety ventilation system with two trains of fans, filters and chillers and requires ac power.
- The safety-related passive system for control room habitability uses compressed air and an eductor-driven filter train to maintain a habitable environment.
- DCD changes submitted to permit development of an achievable control room integrity program.
- DCD changes reviewed and approved by staff.

# Chapter 6 of the Vogtle COL FSAR

#### Containment Cleanliness Program

- Administrative procedures implement the containment cleanliness program
- Implementation of the program minimizes the amount of debris left in containment following personnel entry and exits
- Advisory Committee on Reactor Safeguards (ACRS) letter regarding the Vogtle COL included recommendation regarding containment debris limits
- As a result of ACRS letter, containment debris limits made Tier
   2\* in the AP1000 Design Control Document

#### Toxic Gas Overview

 Control room habitability reviewed to ensure toxic chemicals do not pose a threat to the operator's ability to control the plant

# Acronyms

ACI 349	<ul> <li>American Concrete Institute Code</li> <li>Requirements for Nuclear Safety-</li> <li>Related Concrete Structures &amp;</li> <li>Commentary</li> </ul>
ACRS	<ul> <li>Advisory Committee on Reactor</li> <li>Safeguards</li> </ul>
ASME	<ul> <li>American Society of Mechanical Engineers</li> </ul>
COL	– Combined License
DCD	<ul> <li>Design Control Document</li> </ul>
ESP	<ul> <li>Early Site Permit</li> </ul>
FSAR	<ul> <li>Final Safety Analysis Report</li> </ul>
GSI-191	<ul> <li>Generic Safety Issue 191, "Assessment of Debris Accumulation on PWR Sump Performance"</li> </ul>
Hz	<ul> <li>unit of frequency defined as cycles per second</li> </ul>
IBR	<ul> <li>Incorporated by Reference</li> </ul>
IST	<ul> <li>in-service testing</li> </ul>
ITAAC	<ul> <li>Inspections, Tests, Analyses, and Acceptance Criteria</li> </ul>
LWA	– Limited Work Authorization
QA	<ul> <li>Quality Assurance</li> </ul>

	Deference Combined Licence
RCOL	<ul> <li>Reference Combined License</li> </ul>
RG	<ul> <li>Regulatory Guide</li> </ul>
SCOL	<ul> <li>Subsequent Combined License</li> </ul>
SER	<ul> <li>Safety Evaluation Report</li> </ul>
SNC	<ul> <li>Southern Nuclear Operating Company</li> </ul>
SSC	– Structures, Systems, and Components
SSI	<ul> <li>Soil Structure Interaction</li> </ul>
VEGP	<ul> <li>Vogtle Electric Generating Plant</li> </ul>
10 CFR	<ul> <li>Title 10 of the Code of Federal</li> </ul>
	Regulations