
POLICY ISSUE

(Notation Vote)

October 29, 2014

SECY-14-0118

FOR: The Commissioners

FROM: Mark A. Satorius
Executive Director for Operations

SUBJECT: REQUEST BY DUKE ENERGY FLORIDA, INC., FOR EXEMPTIONS
FROM CERTAIN EMERGENCY PLANNING REQUIREMENTS

PURPOSE:

The purpose of this paper is to seek Commission approval for the staff to grant Duke Energy Florida, Inc.'s (DEF's) request for exemptions from certain emergency planning (EP) requirements of Part 50, "Domestic Licensing of Production and Utilization Facilities," of Title 10 of the *Code of Federal Regulations* (10 CFR). DEF's proposed exemptions would result in elimination of the requirements placed by the U.S. Nuclear Regulatory Commission (NRC) on the licensee for formal offsite radiological emergency plans at the Crystal River Unit 3 (CR-3) site, but would require the maintenance of certain onsite capabilities to communicate and coordinate with offsite response authorities. This paper does not address any new commitments or resource implications.

SUMMARY:

The EP requirements of 10 CFR 50.47, "Emergency Plans," and Appendix E, "Emergency Planning and Preparedness for Production and Utilization Facilities," to 10 CFR Part 50 continue to apply to a nuclear power reactor after permanent cessation of operations and removal of fuel from the reactor vessel. There are no explicit regulatory provisions distinguishing EP requirements for a power reactor that has been shut down from those for an operating power reactor.

CONTACTS: Michael Norris, NSIR/DPR
(301) 287-3754

To modify their emergency plans to reflect the risk commensurate with power reactors that have been permanently shut down, power reactor licensees transitioning to decommissioning must seek exemptions from certain EP regulatory requirements before amending these plans.

The staff has reviewed the technical basis for DEF's requested exemptions and is recommending the Commission approve the staff's proposal to grant the requested EP exemptions, as detailed in the enclosure.

BACKGROUND:

The regulations in 10 CFR 50.12(a)(2)(ii) provide that the NRC may, on application by a licensee or on its own initiative, grant exemptions from the requirements of the regulations in circumstances in which application of the regulation would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule¹. The risk of an offsite radiological release is significantly lower, and the types of possible accidents are significantly fewer, at a nuclear power reactor that has permanently ceased operations and removed fuel from the reactor vessel than at an operating power reactor.

On this basis, the NRC has previously granted similar exemptions from EP requirements for 12 permanently shut down and defueled power reactor licensees. The staff provided an exemption request for the Kewaunee Power Station to the Commission in SECY-14-0066 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14072A257), which was approved by the Commission in the staff requirements memorandum (SRM) to SECY-14-0066 (ADAMS Accession No. ML14219A366). Before the Kewaunee Power Station, the last approved exemptions that eliminated the requirements for formal offsite radiological EP were for the Zion facility in 1999 (ADAMS Legacy Accession No. 9908260192). The underlying technical basis for the approval of the Zion facility exemptions was based on demonstrating that the radiological consequences of design-basis accidents (DBAs) would not exceed the limits of the U.S. Environmental Protection Agency's (EPA's) Protective Action Guides (PAGs) at the exclusion area boundary and that the spent fuel stored in the spent fuel pool (SFP) would not reach the zirconium ignition temperature in fewer than 10 hours based on analysis that assumes no water or air cooling of the fuel. The staff concluded that if 10 hours was available to initiate mitigative actions or, if needed, offsite protective actions using a comprehensive emergency management plan (CEMP)² formal offsite radiological emergency plans are not necessary for permanently defueled nuclear power reactor licensees. In addition to CR-3, San Onofre Nuclear Generating Station and Vermont Yankee Nuclear Power Station have also applied for exemptions from certain EP requirements. The exemption requests by CR-3, as described in

¹ Notwithstanding the special circumstances of the exemption request, 10 CFR 50.12(a)(1) requires that the exemption must be authorized by law, not present an undue risk to the public health and safety, and be consistent with the common defense and security.

² A CEMP, in this context, also referred to as an emergency operations plan (EOP), is addressed in the Federal Emergency Management Agency's (FEMA) Comprehensive Preparedness Guide (CPG) 101, "Developing and Maintaining Emergency Operations Plans." CPG 101 is the foundation for State, territorial, Tribal, and local EP in the United States. It promotes a common understanding of the fundamentals of risk-informed planning and decisionmaking and helps planners at all levels of government in their efforts to develop and maintain viable, all-hazards, all-threats emergency plans. An EOP is flexible enough for use in all emergencies. It describes how people and property will be protected; details who is responsible for carrying out specific actions; identifies the personnel, equipment, facilities, supplies and other resources available; and outlines how all actions will be coordinated. A CEMP is often referred to as a synonym for "all hazards planning."

this paper, are consistent with those approved by the Commission in the SRM to SECY-14-0066.

The NRC requires a level of licensee EP commensurate with the potential consequences to public health and safety and common defense and security at the licensee's site. Under the current safety analysis in NUREG-1738, "Technical Study of Spent Fuel Pool Accident Risk at Decommissioning Nuclear Power Plants" (ADAMS Accession No. ML010430066), the event sequences important to risk at a decommissioning power reactor are limited to a large earthquake and cask-drop events. This is an important difference relative to an operating power reactor where typically a large number of different initiating events make significant contributions to risk. Additionally, physical security for special nuclear material at fixed sites, including decommissioning power reactors, is required by 10 CFR Part 73, "Physical Protection of Plants and Materials." Decommissioning power reactor licensees are required by 10 CFR 73.55(f) to develop target sets for use in the development and implementation of security strategies that protect against spent fuel sabotage. While both operating and decommissioning power reactors are required to develop target sets, the number of target sets at a decommissioning reactor is significantly reduced. Implementation of the protective strategy at a decommissioning reactor takes into account this reduction in target sets. With the significant reduction in radiological risk for a power reactor undergoing decommissioning, the NRC has historically approved exemptions to EP and security requirements based on site-specific evaluations and the objectives of the regulations.

The NRC prepared NUREG-1738 to provide a technical basis for SECY-00-0145, "Integrated Rulemaking Plan for Nuclear Power Plant Decommissioning" (ADAMS Accession No. ML003721626). The proposed rulemaking was later deferred in light of higher priority work after the terrorist attacks of September 11, 2001. Nonetheless, NUREG-1738 provides insights that the staff continues to find helpful for the evaluation of exemption requests regarding EP requirements. Specifically, NUREG-1738 identified a zirconium fire resulting from a substantial loss-of-water inventory in the SFP as the only postulated scenario at a decommissioning power reactor that, while highly unlikely, might result in a significant offsite release.

Previously granted exemptions from EP regulations reduced EP requirements for decommissioning power reactors to those consistent with the standards of: (1) 10 CFR 50.47(d), which states the requirements for a license authorizing fuel loading and low power testing only; and (2) 10 CFR 72.32(a), which establishes the information required in an emergency plan for an independent spent fuel storage installation. Examples of previously granted exemptions from EP regulations for decommissioning power reactors include: setting the highest emergency plan classification as an "Alert"; extending the timing requirements for notification of offsite authorities; requiring only onsite exercises with the opportunity for offsite response organization participation; and only maintaining arrangements for offsite response organizations (i.e., law enforcement, fire and medical services) that may respond to onsite emergencies. The existence of formal offsite radiological emergency plans is no longer a binding requirement on the licensee.

While the staff considers the exemptions from certain EP requirements, as requested by DEF and described above, to be reasonable for a power reactor that has been permanently shut down and defueled, the resulting set of EP requirements could be viewed as a reduction in effectiveness when compared to the operating reactor emergency plan currently in effect at

CR-3. In the SRM to SECY-08-0024, "Delegation of Commission Authority to Staff to Approve or Deny Emergency Plan Changes That Represent a Decrease in Effectiveness," dated May 19, 2008 (ADAMS Accession No. ML081400510), the Commission directed that the staff should request Commission approval for any reduction in effectiveness of a licensee's emergency plan that requires an exemption from the requirements of 10 CFR 50.47(b) and Appendix E to 10 CFR Part 50. In a manner consistent with the SRM's direction, this paper seeks Commission approval for the staff to process and grant, as appropriate, DEF's requested exemptions from the EP requirements as detailed in the enclosure, which provides a summary of DEF's exemption request and a brief description of the staff's basis for recommending approval.

DISCUSSION:

DEF is the holder of Operating License No. DPR-72, issued under the Atomic Energy Act of 1954, as amended, and 10 CFR Part 50, which authorizes the licensee to possess and store spent nuclear fuel and greater-than-class C radioactive waste at the CR-3 facility, which has been permanently shut down and defueled. On September 26, 2009, the CR-3 reactor permanently ceased power generation. All fuel assemblies were removed from the reactor vessel and placed in the SFP on May 28, 2011.

By letter dated February 20, 2013 (ADAMS Accession No. ML13056A005), DEF submitted certifications to the NRC of permanent cessation of power operations at CR-3 and that fuel has been permanently removed from the CR-3 reactor vessel, under 10 CFR 50.82(a)(1)(i) and 10 CFR 50.82(a)(1)(ii). Upon docketing of these certifications, the 10 CFR Part 50 license for CR-3 no longer authorized operation of the reactor or emplacement or retention of fuel into the reactor vessel, as specified in 10 CFR 50.82(a)(2).

By letter dated September 26, 2013, "Request for Exemption to Certain Radiological Emergency Response Plan Requirements Defined by 10 CFR 50" (ADAMS Accession No. ML13274A584), DEF requested exemptions from specific EP requirements of 10 CFR Part 50 for CR-3. The staff issued a request for additional information (RAI) in an e-mail dated February 20, 2014 (ADAMS Accession No. ML14118A287). In a letter dated March 28, 2014 (ADAMS Accession No. ML14098A072), DEF provided a response to the RAI. The exemptions requested by DEF and the staff's review are detailed in the enclosure.

The staff also sent RAIs for an associated proposed change to technical specifications, specifically in reference to the license condition for 10 CFR 50.54(h)(h) mitigation strategies, in an e-mail dated April 10, 2014 (ADAMS Accession No. ML14114A279). In a letter dated May 7, 2014, DEF provided a response to those RAIs (ADAMS Accession No. ML14139A006), which contained information applicable to the SFP inventory makeup strategies for mitigating the loss-of-water inventory. In an e-mail, also dated April 10, 2014 (ADAMS Accession No. ML14113A363), the staff delivered RAIs in reference to the proposed permanently defueled emergency plan and emergency action level scheme change. In a letter dated May 23, 2014 (ADAMS Accession No. ML14154A408), DEF provided responses to those RAIs. The information provided by DEF included justifications for each exemption requested. The staff found the application complete and the licensee's associated technical justification provides a basis for the Commission's consideration of the requested exemption.

In Enclosure 6 to the September 26, 2013, letter, "Crystal River Unit 3 – License Amendment

Request #315, Revision 0, Permanently Defueled Emergency Plan and Emergency Action Level Scheme, and Request for Exemption to Certain Radiological Emergency Response Plan Requirements Defined by 10 CFR 50” (ADAMS Accession No. ML13274A584), DEF provided the accident analyses associated with DBAs and beyond DBAs as a basis for justifying the request for approval of the CR-3 Permanently Defueled Emergency Plan. DEF’s exemption request included radiological analyses to show that the radiological consequences of DBAs will not exceed the limits of the EPA PAGs at the exclusion area boundary. Additionally, DEF performed analyses for beyond DBA loss-of-coolant inventory events for the SFP. These analyses show that for events in which the SFP is drained, air cooling will currently prevent the fuel from reaching the lowest temperature at which incipient cladding failure may occur (565 degrees Celsius (C)).

In the unlikely event that air cooling is not possible, 19.7 hours is available, from the time the fuel is uncovered until it reaches a temperature of 900 degrees C, to initiate mitigative actions consistent with plant conditions and, if necessary, for offsite authorities to employ their CEMP to take protective actions. In addition, significant decay of short-lived radionuclides has occurred since the September 2009 shutdown. As indicated by the results of research conducted for NUREG-1738 and more recently NUREG-2161, “Consequence Study of a Beyond-Design-Basis Earthquake Affecting the Spent Fuel Pool for a U.S. Mark I Boiling Water Reactor” (ADAMS Accession No. ML14255A365), while other consequences can be extensive, accidents from SFPs with significant decay time have little potential to cause offsite early fatalities.

As noted above, DEF furnished information in its exemption request concerning its SFP inventory makeup strategies. Several sources of makeup to the pool are available, such as the fire service system, using the diesel driven fire service pump for loss of electrical power. If available fresh water sources are depleted, salt water sources with inexhaustible inventory from the CR-3 intake and discharge canal, using portable diesel powered pumps, is available. Pool inventory addition can be implemented without accessing the elevation of the pool deck. In a letter dated May 7, 2014, “Crystal River Unit 3 - Response to Requests for Additional Information and Supplement 1 to License Amendment Request #316, Revision 0” (ADAMS Accession No. ML14139A006), DEF withdrew its request to remove License Condition 2.C.(14), “Mitigation Strategy License Condition”, from its Facility Operating License. This license condition requires CR-3 to maintain its SFP inventory makeup strategies as discussed above.

In the unlikely situation that a radiological release is expected, elements of the revised emergency plan would facilitate the ability of offsite authorities to take protective actions under a CEMP. The licensee must still maintain an ability to determine if a radiological release is occurring, and if a release is occurring or expected to occur, promptly communicate that information to offsite authorities. The licensee staff uses the State Hot Ring Down (SHRD) telephone system to notify the State Watch Office, who will assume the responsibility for notifying the State Emergency Management Department and County agencies of a declared emergency. The Florida Nuclear Plant Emergency Notification Form was developed to facilitate timely notifications, and contains information that identifies the station, date and time of the event, emergency classification, emergency action level, weather data, release status/significance and SFP status.

The staff reviewed DEF’s exemption request against the requirements in 10 CFR 50.47, Appendix E to 10 CFR Part 50, and 10 CFR 72.32, “Emergency Plan.” The review considered the status of the facility, which is permanently shut down and defueled, and the low likelihood of

any credible accident resulting in radiological releases requiring offsite protective measures. The staff based its evaluation of the DEF request for exemptions from EP requirements on site-specific analyses provided. The staff verified DEF's analyses and its calculations. The analysis provides reasonable assurance that in granting the requested exemption to DEF: (1) an offsite radiological release will not exceed the EPA PAGs at the site boundary for a DBA; and (2) in the unlikely event of a beyond DBA resulting in a loss of all SFP cooling, there is sufficient time to initiate appropriate mitigating actions; and if a release is projected to occur, there is sufficient time for offsite agencies to take protective actions using a CEMP to protect the health and safety of the public.

Consistent with the June 17, 1993, memorandum of understanding between the NRC and the Federal Emergency Management Agency (FEMA), contained in Appendix A to 44 CFR 353, the staff has discussed and coordinated our review of requests for exemptions to EP regulations with FEMA. As part of the staff's evaluation of the recent EP exemption request for the Kewaunee Power Station, the staff provided FEMA with a copy of SECY-14-0066 and the opportunity to ask questions, obtain clarification, and comment on the paper prior to the Commission receiving it for review. FEMA provided the following comments in response to the EP exemption proposed in SECY-14-0066:

FEMA is not taking a position on the technical arguments presented by the licensee or the NRC's assessments. FEMA recognizes the NRC's role to analyze the possibility of incidents that could result in offsite dose impacts. FEMA acknowledges that individual states and local governments have the primary authority and responsibility to protect their citizens and respond to disasters and emergencies. The exemption, if issued, could create a transitional environment for off-site emergency planners in how they consider radiological hazards. FEMA will continue to support offsite organizations as they adjust their plans, capabilities, and resources to the changing radiological threat. Among the resources available to support FEMA stakeholders during the transition process include, but are not limited to, the National Preparedness System guidance materials, the Federal Radiological Preparedness Coordinating Committee, and assistance from FEMA Headquarters and Regional Staff.

The NRC staff considered FEMA's comments as part of SECY-14-0066 and believes that the technical and safety basis for the exemption demonstrates reasonable assurance in the two areas mentioned above.

The staff offered FEMA an opportunity to review a copy of this paper prior to the Commission receiving it for review. FEMA advised the staff that their review was not necessary since the evaluation of the CR-3 EP exemption request was consistent with that used in SECY-14-0066 for Kewaunee. As such, FEMA's comments as described above for Kewaunee are applicable to CR-3.

The decommissioning facility, at the time the exemption is granted, would pose significantly less of a radiological risk to public health and safety than an operating power reactor, which should result in a straightforward transition to a more streamlined CEMP. Aspects of existing offsite radiological emergency preparedness plans may remain in place, at the State's discretion, prior to completion of any adjustments to State and local CEMPs that are appropriate for the reduced radiological risk and can be adopted to minimize burden on the State and local governments. DEF will still be required to maintain an onsite emergency plan, which would provide for the

notification of, and coordination with, offsite organizations commensurate with the approved exemptions.

The staff's exemption recommendation, if approved by the Commission, would not affect the authority that FEMA has under its regulations in 44 CFR Chapter I for overall emergency management and assistance to State and local response organizations, nor would it affect the responsibilities of State and local governments to establish and maintain CEMPs. The NRC would base its finding of reasonable assurance on its review of licensee onsite emergency preparedness and would not require a finding from FEMA on the adequacy of State and local CEMPs. Under its role as described in the National Response Framework, the NRC remains ready to support FEMA by providing it and State and local governments technical advice related to the safety and security of operations at the plant.

In the September 26, 2013, letter referenced above, DEF also requested a license amendment to approve its emergency plan implementing changes that reflect the permanently shut down and defueled status of CR-3. The revised emergency plan also includes changes consistent with the proposed exemptions discussed in this paper. The staff is awaiting a decision on this paper before issuing a decision on the amendment request.

CONCLUSION:

The NRC staff concludes that granting the exemption request, as provided in the enclosure, would provide: (1) an adequate basis for an acceptable state of emergency preparedness; and (2) in conjunction with arrangements made with offsite response agencies, reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency at CR-3.

The NRC has determined that pursuant to 10 CFR 50.12, "Specific Exemptions," the exemptions described in the enclosure are authorized by law, will not present an undue risk to the public health and safety, and will be consistent with the common defense and security, and special circumstances are present.

RECOMMENDATION:

The exemption request is consistent with previously granted exemptions and SECY-14-0066 for the Kewaunee Power Station, and is commensurate with the risk associated with the facility. The changes in regulatory requirements are appropriate because the traditional accident sequences that dominate operating reactor risk are no longer applicable. Continued application of the regulation to the licensee, to maintain its current level of EP, is not necessary to achieve the underlying purpose of the regulation. Therefore, the staff recommends that the Commission:

Approve: The staff's proposal to grant DEF's requested EP exemptions from certain requirements of 10 CFR 50.47(b) and Appendix E to 10 CFR Part 50 consistent with the discussion above.

COORDINATION:

The Office of the General Counsel reviewed this paper and has no legal objection. The Office of the Chief Financial Officer reviewed this paper for resource implications and has no objection.

/RA/

Mark A. Satorius
Executive Director
for Operations

Enclosure:

[Exemptions to Rule Language](#)

Exemptions to Rule Language

Strikethrough text indicates requested exemptions to rule language.

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---|--|
| <p>(b) The onsite and, except as provided in paragraph (d) of this section, offsite emergency response plans for nuclear power reactors must meet the following standards:</p> | <p>In the Statement of Considerations (SOC) for the final rule for emergency planning (EP) requirements for independent spent fuel storage installations (ISFSIs) and for monitor retrievable storage (MRS) facilities (60 FR [<i>Federal Register</i>] 32430; June 22, 1995), the Commission responded to comments concerning offsite EP for ISFSIs or an MRS and concluded that, “the offsite consequences of potential accidents at an ISFSI or an MRS would not warrant establishing Emergency Planning Zones [EPZs].” In a nuclear power reactor’s permanently defueled state, the accident risks are more similar to an ISFSI than an operating nuclear power plant. The EP program would be similar to that required for an ISFSI under Section 72.32(a) of Title 10 of the <i>Code of Federal Regulations</i> (10 CFR) when fuel stored in the spent fuel pool (SFP) and would not change substantially when all the fuel is transferred from the SFP to an onsite ISFSI. Exemptions from offsite EP requirements have previously been approved when the site-specific analyses show that at least 10 hours is available from a partial drain-down event where cooling of the spent fuel is not effective until the hottest fuel assembly reaches 900°C. The technical basis that underlied the approval of the exemption request is based partly on the analysis of a time period that spent fuel stored in the SFP is unlikely to reach the zirconium ignition temperature in less than 10 hours. This time period is based on a heat-up calculation which uses several simplifying assumptions. Some of these assumptions are conservative (adiabatic conditions), while others are non-conservative (no oxidation below 900°C). Weighing the conservatisms and non-conservatisms, the staff judges that this calculation reasonably represents conditions which may occur in the event of an SFP accident.</p> |

Enclosure

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---------------------|--|
| | <p>The staff concluded that if 10 hours is available to initiate mitigative actions or, if needed, offsite protective actions using a comprehensive emergency management plan (CEMP), formal offsite radiological emergency plans are not necessary for these permanently defueled nuclear power reactor licensees.</p> <p>As supported by the licensee's SFP analysis, the staff believes an exemption to the requirements for formal offsite radiological emergency plans is justified for a zirconium fire scenario considering the low likelihood of this event together with time available to take mitigative or protective actions between the initiating event and before the onset of a postulated fire.</p> <p>The Duke Energy Florida (DEF) analysis has demonstrated that due to the considerable time since shutdown, approximately 4 years, the radiological consequences of design-basis accidents will not exceed the limits of the U.S. Environmental Protection Agency's (EPA's) Protective Action Guides (PAGs) at the exclusion area boundary. These analyses also show that, for beyond design basis events where the SFP is drained, air cooling will prevent the fuel from reaching the lowest temperature where incipient cladding failure may occur (565°C). In the event that air cooling is not possible, 19.7 hours is available to take mitigative or, if needed, offsite protective actions using a CEMP from the time the fuel is uncovered until it reaches the auto-ignition temperature of 900°C.</p> <p>DEF has also furnished information on its SFP inventory makeup strategies for mitigating the loss of water inventory. Several sources of makeup to the pools are available, such as the fire service system, using the diesel driven fire service pump for loss of electrical power. If available fresh water sources are depleted, salt water sources with inexhaustible inventory from the Crystal River Unit 3 (CR-3) intake and discharge canal, using portable diesel powered pumps are available.</p> |

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---|--|
| | <p>Pool inventory addition can be implemented without accessing the elevation of the pool deck. In a letter dated May 7, 2014, "Crystal River Unit 3 - Response to Requests for Additional Information and Supplement 1 to License Amendment Request #316, Revision 0" (ADAMS Accession No. ML14139A006), DEF withdrew its request to remove License Condition 2.C.(14), "Mitigation Strategy License Condition," from its Facility Operating License. This license condition requires CR-3 to maintain its SFP inventory makeup strategies as discussed above</p> |
| <p>(1) Primary responsibilities for emergency response by the nuclear facility licensee and by State and local organizations within the Emergency Planning Zones have been assigned, the emergency responsibilities of the various supporting organizations have been specifically established, and each principal response organization has staff to respond and to augment its initial response on a continuous basis.</p> | <p>Refer to basis for 50.47(b).</p> |
| <p>(3) Arrangements for requesting and effectively using assistance resources have been made, arrangements to accommodate State and local staff at the licensee's Emergency Operations Facility have been made, and other organizations capable of augmenting the planned response have been identified.</p> | <p>Decommissioning power reactors present a low likelihood of any credible accident resulting in a radiological release together with the time available to take mitigative or, if needed, offsite protective actions using a CEMP between the initiating event and before the onset of a postulated fire. As such, an emergency operations facility would not be required. The "nuclear island," control room, or other onsite location can provide for the communication and coordination with offsite organizations for the level of support required.</p> <p>Also refer to basis for 50.47(b).</p> |

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---|--|
| <p>(4) A standard emergency classification and action level scheme, the basis of which include facility system and effluent parameters, is in use by the nuclear facility licensee, and State and local response plans call for reliance on information provided by facility licensees for determinations of minimum initial offsite response measures.</p> | <p>Decommissioning power reactors present a low likelihood of any credible accident resulting in a radiological release together with the time available to take mitigative or, if needed, offsite protective actions using a CEMP between the initiating event and before the onset of a postulated fire. As such, formal offsite radiological emergency response plans are not required.</p> <p>The Nuclear Energy Institute (NEI) document NEI 99-01, "Development of Emergency Action Levels for Non-Passive Reactors" (Revision 6), was found to be an acceptable method for development of emergency action levels (EALs) and was endorsed by the U.S. Nuclear Regulatory Commission (NRC) in a letter dated March 28, 2013 (ADAMS Accession No. ML12346A463). NEI 99-01 provides EALs for non-passive operating nuclear power reactors, permanently defueled reactors and ISFSIs.</p> <p>Also refer to basis for 10 CFR 50.47(b).</p> |
| <p>(5) Procedures have been established for notification, by the licensee, of State and local response organizations and for notification of emergency personnel by all organizations; the content of initial and follow up messages to response organizations and the public has been established; and means to provide early notification and clear instruction to the populace within the plume exposure pathway Emergency Planning Zone have been established.</p> | <p>Refer to basis for 10 CFR 50.47(b).</p> |
| <p>(6) Provisions exist for prompt communications among principal response organizations to emergency personnel and to the public.</p> | <p>Refer to basis for 10 CFR 50.47(b).</p> |

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---|--|
| <p>(7) Information is made available to the public on a periodic basis on how they will be notified and what their initial actions should be in an emergency (e.g., listening to a local broadcast station and remaining indoors); [T]he principal points of contact with the news media for dissemination of information during an emergency (including the physical location or locations) are established in advance, and procedures for coordinated dissemination of information to the public are established.</p> | <p>Refer to basis for 10 CFR 50.47(b).</p> |
| <p>(9) Adequate methods, systems, and equipment for assessing and monitoring actual or potential offsite consequences of a radiological emergency condition are in use.</p> | <p>Refer to basis for 10 CFR 50.47(b).</p> |
| <p>(10) A range of protective actions has been developed for the plume exposure pathway EPZ for emergency workers and the public. In developing this range of actions, consideration has been given to evacuation, sheltering, and, as a supplement to these, the prophylactic use of potassium iodide (KI), as appropriate. Evacuation time estimates have been developed by applicants and licensees. Licensees shall update the evacuation time estimates on a periodic basis. Guidelines for the choice of protective actions during an emergency, consistent with Federal guidance, are developed and in place, and protective actions for the ingestion exposure pathway EPZ appropriate to the locale have been developed.</p> | <p>In the unlikely event of an SFP accident, the iodine isotopes, which contribute to an off-site dose from an operating reactor accident, are not present, so potassium iodide distribution would no longer serve as an effective or necessary supplemental protective action.</p> <p>The Commission responded to comments in its SOC for the final rule for EP requirements for ISFSIs and MRS facilities (60 FR 32435), and concluded that, “the offsite consequences of potential accidents at an ISFSI or an MRS would not warrant establishing emergency planning zones.” Additionally, in the SOC for the final rule for EP requirements for ISFSIs and for MRS facilities (60 FR 32430), the Commission responded to comments concerning site-specific EP that includes evacuation of surrounding population for an ISFSI not at a reactor site, and concluded that, “The Commission does not agree that as a general matter emergency plans for an ISFSI must include evacuation planning.”</p> <p>Also refer to basis for 10 CFR 50.47(b).</p> |

| 10 CFR 50.47 | Staff Review of Licensee Justification |
|---|--|
| <p>(c)(2) Generally, the plume exposure pathway EPZ for nuclear power plants shall consist of an area about 10 miles (16 km) in radius and the ingestion pathway EPZ shall consist of an area about 50 miles (80 km) in radius. The exact size and configuration of the EPZs surrounding a particular nuclear power reactor shall be determined in relation to local emergency response needs and capabilities as they are affected by such conditions as demography, topography, land characteristics, access routes, and jurisdictional boundaries. The size of the EPZs also may be determined on a case-by-case basis for gas-cooled nuclear reactors and for reactors with an authorized power level less than 250 MW thermal. The plans for the ingestion pathway shall focus on such actions as are appropriate to protect the food ingestion pathway.</p> | <p>Refer to basis for 10 CFR 50.47(b)(10).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>1. The applicant's emergency plans shall contain, but not necessarily be limited to, information needed to demonstrate compliance with the elements set forth below, i.e., organization for coping with radiological emergencies, assessment actions, activation of emergency organization, notification procedures, emergency facilities and equipment, training, maintaining emergency preparedness, and recovery, and onsite protective actions during hostile action. In addition, the emergency response plans submitted by an applicant for a nuclear power reactor operating license under this Part, or for an early site permit (as applicable) or combined license under 10 CFR Part 52, shall contain information needed to demonstrate compliance with the standards described in § 50.47(b), and they will be evaluated against those standards.</p> | <p>The EP rule published in the <i>Federal Register</i> (76 FR 72560; November 23, 2011) amended certain requirements in 10 CFR Part 50. Among the changes, the definition of "hostile action" was added as an act directed toward a nuclear power plant or its personnel. This definition is based on the definition of "hostile action" provided in NRC Bulletin 2005-02, "Emergency Preparedness and Response Actions for Security-Based Events." NRC Bulletin 2005-02 was not applicable to nuclear power reactors that had permanently ceased operations and had certified that fuel has been removed from the reactor vessel.</p> <p>The NRC excluded non-power reactors from the definition of "hostile action" at the time of the rulemaking because, as defined in 10 CFR 50.2, a non-power reactor is not considered a nuclear power reactor and a regulatory basis had not been developed to support the inclusion of non-power reactors in the definition of "hostile action." Similarly, a decommissioning power reactor or ISFSI is not a "nuclear reactor" as defined in the NRC's regulations. A decommissioning power reactor also has a low likelihood of a credible accident resulting in radiological releases requiring offsite protective measures. For all of these reasons, the staff concludes that a decommissioning power reactor is not a facility that falls within the definition of "hostile action."</p> <p>Similarly, for security, risk insights can be used to determine which targets are important to protect against sabotage. A level of security commensurate with the consequences of a sabotage event is required and is evaluated on a site-specific basis. The severity of the consequences declines as fuel ages and, thereby, removes over time the underlying concern that a sabotage attack could cause offsite radiological consequences.</p> <p>Although this analysis provides a justification for exempting CR-3 from "hostile action" related requirements, some EP requirements for security-based events are maintained. The classification of</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|--|
| | security-based events, notification of offsite authorities and coordination with offsite agencies under a CEMP concept are still required. |
| <p>2. This nuclear power reactor license applicant shall also provide an analysis of the time required to evacuate various sectors and distances within the plume exposure pathway EPZ for transient and permanent populations, using the most recent U.S. Census Bureau data as of the date the applicant submits its application to the NRC.</p> | Refer to basis for 10 CFR 50.47(b)(10). |
| <p>3. Nuclear power reactor licensees shall use NRC approved evacuation time estimates (ETEs) and updates to the ETEs in the formulation of protective action recommendations and shall provide the ETEs and ETE updates to State and local governmental authorities for use in developing offsite protective action strategies.</p> | Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2. |
| <p>4. Within 365 days of the later of the date of the availability of the most recent decennial census data from the U.S. Census Bureau or December 23, 2011, nuclear power reactor licensees shall develop an ETE analysis using this decennial data and submit it under § 50.4 to the NRC. These licensees shall submit this ETE analysis to the NRC at least 180 days before using it to form protective action recommendations and providing it to State and local governmental authorities for use in developing offsite protective action strategies</p> | Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2. |
| <p>5. During the years between decennial censuses, nuclear power reactor licensees shall estimate EPZ permanent resident population changes once a year, but no later than 365 days from the date of the previous estimate, using the most recent U.S. Census Bureau annual resident population estimate and State/local government population data, if available. These licensees shall maintain these estimates so that they are available</p> | Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2. |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|--|
| for NRC inspection during the period between decennial censuses and shall submit these estimates to the NRC with any updated ETE analysis. | |
| 6. If at any time during the decennial period, the EPZ permanent resident population increases such that it causes the longest ETE value for the 2-mile zone or 5-mile zone, including all affected Emergency Response Planning Areas, or for the entire 10-mile EPZ to increase by 25 percent or 30 minutes, whichever is less, from the nuclear power reactor licensee's currently NRC approved or updated ETE, the licensee shall update the ETE analysis to reflect the impact of that population increase. The licensee shall submit the updated ETE analysis to the NRC under § 50.4 no later than 365 days after the licensee's determination that the criteria for updating the ETE have been met and at least 180 days before using it to form protective action recommendations and providing it to State and local governmental authorities for use in developing offsite protective action strategies. | Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2. |
| A.1. A description of the normal plant operating organization. | Based on the permanently shut down and defueled status of the reactor, a decommissioning reactor is not authorized to operate under 10 CFR 50.82(a). Because the licensee cannot operate the reactor, the licensee does not have a "plant operating organization." |
| A.3. A description, by position and function to be performed, of the licensee's headquarters personnel who will be sent to the plant site to augment the onsite emergency organization. | The number of staff at decommissioning sites is generally small but is commensurate with the need to safely store spent fuel at the facility in a manner that is protective of public health and safety. Decommissioning sites typically have a level of emergency response that does not require response by the licensee's headquarters personnel. |
| A. 4. Identification, by position and function to be performed, of persons within the licensee organization who will be responsible for making offsite dose projections, and a description of how | Although the likelihood of events that would result in doses in excess of the EPA PAGs to the public beyond the owner controlled area boundary based on the permanently shut down and defueled status of the reactor is extremely low, the licensee still |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>these projections will be made and the results transmitted to State and local authorities, NRC, and other appropriate governmental entities.</p> | <p>must be able to determine if a radiological release is occurring. If a release is occurring, then the licensee staff should promptly communicate that information to offsite authorities for their consideration. The offsite organizations are responsible for deciding what, if any, protective actions should be taken based on comprehensive EP.</p> |
| <p>A. 5. Identification, by position and function to be performed, of other employees of the licensee with special qualifications for coping with emergency conditions that may arise. Other persons with special qualifications, such as consultants, who are not employees of the licensee and who may be called upon for assistance for emergencies shall also be identified. The special qualifications of these persons shall be described.</p> | <p>The number of staff at decommissioning sites is generally small but should be commensurate with the need to operate the facility in a manner that is protective of public health and safety.</p> |
| <p>A. 7. By June 23, 2014, identification of, and a description of the assistance expected from, appropriate State, local, and Federal agencies with responsibilities for coping with emergencies, including hostile action at the site. For purposes of this appendix, "hostile action" is defined as an act directed toward a nuclear power plant or its personnel that include the use of violent force to destroy equipment, take hostages, and/or intimidate the licensee to achieve an end. This includes attack by air, land, or water using guns, explosives, projectiles, vehicles, or other devices used to deliver destructive force.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.1.</p> |
| <p>A. 8. Identification of the State and/or local officials responsible for planning for, ordering and controlling appropriate protective actions, including evacuations when necessary.</p> | <p>Offsite emergency measures are limited to support provided by local police, fire departments, and ambulance and hospital services, as appropriate. Due to the low probability of design-basis accidents or other credible events to exceed the EPA PAGs, protective actions such as evacuation should not be required, but could be implemented at the discretion of offsite authorities using a CEMP.</p> <p>Also refer to basis for 10 CFR 50.47(b)(10).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|--|
| <p>A.9. By December 24, 2012, for nuclear power reactor licensees, a detailed analysis demonstrating that on-shift personnel assigned emergency plan implementation functions are not assigned responsibilities that would prevent the timely performance of their assigned functions as specified in the emergency plan.</p> | <p>Responsibilities should be well defined in the emergency plan and procedures, regularly tested through drills and exercises audited and inspected by the licensee and the NRC. The duties of the onshift personnel at a decommissioning reactor facility are not as complicated and diverse as those for an operating power reactor.</p> <p>The staff considered the similarity between the staffing levels at a permanently shut down and defueled reactor and staffing levels at an operating power reactor site. The minimal systems and equipment needed to maintain the spent nuclear fuel in the SFP or in a dry cask storage system in a safe condition requires minimal personnel and is governed by Technical Specifications. In the EP final rule published in the <i>Federal Register</i> (76 FR 72560; November 23, 2011), the NRC concluded that the staffing analysis requirement was not necessary for non-power reactor licensees due to the small staffing levels required to operate the facility.</p> <p>The staff also examined the actions required to mitigate the very low probability design-basis events for the SFP. Several sources of makeup to the pools are available, such as the fire service system, using the diesel driven fire service pump for loss of electrical power. If available fresh water sources are depleted, salt water sources with inexhaustible inventory from the CR-3 intake and discharge canal, using portable diesel powered pumps are available. Pool inventory addition can be implemented without accessing the elevation of the pool deck. DEF believes these diverse strategies provide defense-in-depth and ample time to provide makeup or spray to the SFP prior to the onset of zirconium cladding ignition when considering very low probability beyond design-basis events affecting the SFP. In a letter dated May 7, 2014, "Crystal River Unit 3 - Response to Requests for Additional Information and Supplement 1 to License Amendment Request #316, Revision 0" (ADAMS Accession No. ML14139A006), DEF withdrew its request to remove License Condition 2.C.(14), "Mitigation</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|--|
| | <p>Strategy License Condition,” from its Facility Operating License. This license condition requires CR-3 to maintain its SFP inventory makeup strategies as discussed above.</p> |
| <p>B.1. The means to be used for determining the magnitude of, and for continually assessing the impact of, the release of radioactive materials shall be described, including emergency action levels that are to be used as criteria for determining the need for notification and participation of local and State agencies, the Commission, and other Federal agencies, and the emergency action levels that are to be used for determining when and what type of protective measures should be considered within and outside the site boundary to protect health and safety. The emergency action levels shall be based on in-plant conditions and instrumentation in addition to onsite and offsite monitoring. By June 20, 2012, for nuclear power reactor licensees, these action levels must include hostile action that may adversely affect the nuclear power plant. The initial emergency action levels shall be discussed and agreed on by the applicant or licensee and State and local governmental authorities, and approved by the NRC. Thereafter, emergency action levels shall be reviewed with the State and local governmental authorities on an annual basis.</p> | <p>NEI 99-01 was found to be an acceptable method for development of EALs. No offsite protective actions are anticipated to be necessary, so classification above the alert level is no longer required, which is consistent with ISFSI facilities.</p> <p>Also refer to basis for 10 CFR Part 50, Appendix E, Section IV.1.</p> |
| <p>C.1. The entire spectrum of emergency conditions that involve the alerting or activating of progressively larger segments of the total emergency organization shall be described. The communication steps to be taken to alert or activate emergency personnel under each class of emergency shall be described. Emergency action levels (based not only on onsite and offsite radiation monitoring information but also on readings from a number of sensors</p> | <p>Containment parameters do not provide an indication of the conditions at a defueled facility and emergency core cooling systems are no longer required. Other indications, such as SFP level or temperature, can be used at sites where there is spent fuel in the SFPs.</p> <p>In the SOC for the final rule for EP requirements for ISFSIs and for MRS facilities (60 FR 32430), the Commission responded to comments concerning a general emergency at an ISFSI and MRS, and concluded that, “...an essential element of a</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>that indicate a potential emergency, such as the pressure in containment and the response of the Emergency Core Cooling System) for notification of offsite agencies shall be described. The existence, but not the details, of a message authentication scheme shall be noted for such agencies. The emergency classes defined shall include: (1) notification of unusual events, (2) alert, (3) site area emergency, and (4) general emergency. These classes are further discussed in NUREG-0654/FEMA-REP-1.</p> | <p>General Emergency is that a release can be reasonably expected to exceed EPA Protective Action Guidelines exposure levels off site for more than the immediate site area.”</p> <p>The probability of a condition reaching the level above an emergency classification of alert is very low. In the event of an accident at a defueled facility that meets the conditions for exemption from formal offsite EP requirements, there will be available time for event mitigation and, if necessary, implementation of offsite protective actions using a CEMP.</p> <p>NEI 99-01 was found to be an acceptable method for development of EALs. No offsite protective actions are anticipated to be necessary, so classification above the alert level is no longer required.</p> |
| <p>C.2. By June 20, 2012, nuclear power reactor licensees shall establish and maintain the capability to assess, classify, and declare an emergency condition within 15 minutes after the availability of indications to plant operators that an emergency action level has been exceeded and shall promptly declare the emergency condition as soon as possible following identification of the appropriate emergency classification level. Licensees shall not construe these criteria as a grace period to attempt to restore plant conditions to avoid declaring an emergency action due to an emergency action level that has been exceeded. Licensees shall not construe these criteria as preventing implementation of response actions deemed by the licensee to be necessary to protect public health and safety provided that any delay in declaration does not deny the State and local authorities the opportunity to implement measures necessary to protect the public health and safety.</p> | <p>In the EP rule published in the <i>Federal Register</i> (76 FR 72560), non-power reactor licensees were not required to assess, classify and declare an emergency condition within 15 minutes. An SFP and an ISFSI are also not nuclear power reactors as defined in the NRC’s regulations. A decommissioning power reactor has a low likelihood of a credible accident resulting in radiological releases requiring offsite protective measures. For these reasons, the staff concludes that a decommissioning power reactor should not be required to assess, classify and declare an emergency condition within 15 minutes.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|---|
| <p>D.1. Administrative and physical means for notifying local, State, and Federal officials and agencies and agreements reached with these officials and agencies for the prompt notification of the public and for public evacuation or other protective measures, should they become necessary, shall be described. This description shall include identification of the appropriate officials, by title and agency, of the State and local government agencies within the EPZs.</p> | <p>Refer to basis for 10 CFR 50.47(b) and 10 CFR 50.47(b)(10).</p> |
| <p>D.2. Provisions shall be described for yearly dissemination to the public within the plume exposure pathway EPZ of basic emergency planning information, such as the methods and times required for public notification and the protective actions planned if an accident occurs, general information as to the nature and effects of radiation, and a listing of local broadcast stations that will be used for dissemination of information during an emergency. Signs or other measures shall also be used to disseminate to any transient population within the plume exposure pathway EPZ appropriate information that would be helpful if an accident occurs.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.D.1.</p> |
| <p>D.3. A licensee shall have the capability to notify responsible State and local governmental agencies within 45 minutes after declaring an emergency. The licensee shall demonstrate that the appropriate governmental authorities have the capability to make a public alerting and notification decision promptly on being informed by the licensee of an emergency condition. Prior to initial operation greater than 5 percent of rated thermal power of the first reactor at the site, each nuclear power reactor licensee shall demonstrate that administrative and physical means have been established for alerting and providing prompt instructions to the public with the plume exposure pathway EPZ. The design objective of the</p> | <p>While the capability needs to exist for the notification of offsite government agencies within a specified time period, previous exemptions have allowed for extending the State and local government agencies' notification time up to 60 minutes based on the site-specific justification provided.</p> <p>DEF's exemption request provides that CR-3 will make notifications to the State of Florida and the NRC within 60 minutes of declaration of an event. The State Watch Office will perform the notification to the County (Citrus), as well as the Florida Department of Emergency Management. In the permanently defueled condition of the reactor, the rapidly developing scenarios associated with events initiated during reactor power operation are no longer credible.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|--|
| <p>prompt public alert and notification system shall be to have the capability to essentially complete the initial alerting and notification of the public within the plume exposure pathway EPZ within about 15 minutes. The use of this alerting and notification capability will range from immediate alerting and notification of the public (within 15 minutes of the time that State and local officials are notified that a situation exists requiring urgent action) to the more likely events where there is substantial time available for the appropriate governmental authorities to make a judgment whether or not to activate the public alert and notification system. The alerting and notification capability shall additionally include administrative and physical means for a backup method of public alerting and notification capable of being used in the event the primary method of alerting and notification is unavailable during an emergency to alert or notify all or portions of the plume exposure pathway EPZ population. The backup method shall have the capability to alert and notify the public within the plume exposure pathway EPZ, but does not need to meet the 15 minute design objective for the primary prompt public alert and notification system. When there is a decision to activate the alert and notification system, the appropriate governmental authorities will determine whether to activate the entire alert and notification system simultaneously or in a graduated or staged manner. The responsibility for activating such a public alert and notification system shall remain with the appropriate governmental authorities.</p> | <p>Also refer to basis for 10 CFR 50.47(b) and 10 CFR 50.47(b)(10).</p> |
| <p>D.4. If FEMA has approved a nuclear power reactor site's alert and notification design report, including the backup alert and notification capability, as of December 23, 2011, then the backup alert</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.D.3 regarding the alert and notification system requirements.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|---|
| <p>and notification capability requirements in Section IV.D.3 must be implemented by December 24, 2012. If the alert and notification design report does not include a backup alert and notification capability or needs revision to ensure adequate backup alert and notification capability, then a revision of the alert and notification design report must be submitted to FEMA for review by June 24, 2013, and the FEMA-approved backup alert and notification means must be implemented within 365 days after FEMA approval. However, the total time period to implement a FEMA-approved backup alert and notification means must not exceed June 22, 2015.</p> | |
| <p>E.8.a.(i) A licensee onsite technical support center and an emergency operations facility from which effective direction can be given and effective control can be exercised during an emergency;</p> | <p>Due to the low probability of design-basis accidents or other credible events to exceed the EPA PAGs at the site boundary, the available time for event mitigation at a decommissioning reactor and, if needed, to implement offsite protective actions using a CEMP, an emergency operations facility (EOF) would not be required to support offsite agency response. Onsite actions may be directed from the control room or other location, without the requirements imposed on a technical support center (TSC).</p> |
| <p>E.8.a. (ii) For nuclear power reactor licensees, a licensee onsite operational support center;</p> | <p>NUREG-0696, "Functional Criteria for Emergency Response Facilities," provides that the operational support center (OSC) is an onsite area separate from the control room and the TSC where licensee operations support personnel will assemble in an emergency. For a decommissioning power reactor, an OSC is no longer required to meet its original purpose of an assembly area for plant logistical support during an emergency. The OSC function can be incorporated into another facility.</p> |
| <p>E.8.b. For a nuclear power reactor licensee's emergency operations facility required by paragraph 8.a of this section, either a facility located between 10 miles and 25 miles of the nuclear power reactor site(s), or a primary facility located less than 10 miles from the nuclear power reactor site(s) and a backup facility</p> | <p>Refer to basis for 10 CFR 50.47(b)(3).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>located between 10 miles and 25 miles of the nuclear power reactor site(s). An emergency operations facility may serve more than one nuclear power reactor site. A licensee desiring to locate an emergency operations facility more than 25 miles from a nuclear power reactor site shall request prior Commission approval by submitting an application for an amendment to its license. For an emergency operations facility located more than 25 miles from a nuclear power reactor site, provisions must be made for locating NRC and offsite responders closer to the nuclear power reactor site so that NRC and offsite responders can interact face-to-face with emergency response personnel entering and leaving the nuclear power reactor site. Provisions for locating NRC and offsite responders closer to a nuclear power reactor site that is more than 25 miles from the emergency operations facility must include the following:</p> <ul style="list-style-type: none"> (1) Space for members of an NRC site team and Federal, State, and local responders; (2) Additional space for conducting briefings with emergency response personnel; (3) Communication with other licensee and offsite emergency response facilities; (4) Access to plant data and radiological information; and (5) Access to copying equipment and office supplies; | |
| <p>E.8.c. By June 20, 2012, for a nuclear power reactor licensee's emergency operations facility required by paragraph 8.a of this section, a facility having the following capabilities:</p> <ul style="list-style-type: none"> (1) The capability for obtaining and displaying plant data and radiological information for each reactor at a nuclear power reactor site and for each nuclear | <p>Refer to basis for 10 CFR 50.47(b)(3).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|--|
| <p>power reactor site that the facility serves; (2) The capability to analyze plant technical information and provide technical briefings on event conditions and prognosis to licensee and offsite response organizations for each reactor at a nuclear power reactor site and for each nuclear power reactor site that the facility serves; and (3) The capability to support response to events occurring simultaneously at more than one nuclear power reactor site if the emergency operations facility serves more than one site; and</p> | |
| <p>E.8.d. For nuclear power reactor licensees, an alternative facility (or facilities) that would be accessible even if the site is under threat of or experiencing hostile action, to function as a staging area for augmentation of emergency response staff and collectively having the following characteristics: the capability for communication with the emergency operations facility, control room, and plant security; the capability to perform offsite notifications; and the capability for engineering assessment activities, including damage control team planning and preparation, for use when onsite emergency facilities cannot be safely accessed during hostile action. The requirements in this paragraph 8.d must be implemented no later than December 23, 2014, with the exception of the capability for staging emergency response organization personnel at the alternative facility (or facilities) and the capability for communications with the emergency operations facility, control room, and plant security, which must be implemented no later than June 20, 2012.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.1 regarding hostile action.</p> |
| <p>E.8.e. A licensee shall not be subject to the requirements of paragraph 8.b of this section for an existing emergency operations facility approved as of December 23, 2011;</p> | <p>Refer to basis for 10 CFR 50.47(b)(3).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|--|
| <p>E.9.a. Provisions for communications with contiguous State/local governments within the plume exposure pathway EPZ. Such communication shall be tested monthly.</p> | <p>Refer to basis for 10 CFR 50.47(b) and 10 CFR 50.47(b)(10).</p> <p>The State and the local governments in which the nuclear facility is located need to be informed of events and emergencies, so lines of communication must be maintained.</p> |
| <p>E.9.c. Provision for communications among the nuclear power reactor control room, the onsite technical support center, and the emergency operations facility; and among the nuclear facility, the principal State and local emergency operations centers, and the field assessment teams. Such communications systems shall be tested annually.</p> | <p>Because of the low probability of design-basis accidents or other credible events that would be expected to exceed the EPA PAGs and the available time for event mitigation and, if needed, implementation of offsite protective actions using a CEMP, there is no need for the TSC, EOF, or offsite field assessment teams.</p> <p>Also refer to justification for 10 CFR 50.47(b)(3). Communication with State and local emergency operations centers is maintained to coordinate assistance on site if required.</p> |
| <p>E.9.d. Provisions for communications by the licensee with NRC Headquarters and the appropriate NRC Regional Office Operations Center from the nuclear power reactor control room, the onsite technical support center, and the emergency operations facility. Such communications shall be tested monthly.</p> | <p>The functions of the control room, EOF, TSC, and OSC may be combined into one or more locations due to the smaller facility staff and the greatly reduced required interaction with State and local emergency response facilities.</p> <p>Also refer to basis for 10 CFR 50.47(b).</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|--|
| <p>F.1. The program to provide for: (a) The training of employees and exercising, by periodic drills, of radiation emergency plans to ensure that employees of the licensee are familiar with their specific emergency response duties, and (b) The participation in the training and drills by other persons whose assistance may be needed in the event of a radiation emergency shall be described. This shall include a description of specialized initial training and periodic retraining programs to be provided to each of the following categories of emergency personnel:</p> <ul style="list-style-type: none"> i. Directors and/or coordinators of the plant emergency organization; ii. Personnel responsible for accident assessment, including control room shift personnel; iii. Radiological monitoring teams; iv. Fire control teams (fire brigades); v. Repair and damage control teams; vi. First aid and rescue teams; vii. Medical support personnel; viii. Licensee's headquarters support personnel; ix. Security personnel. <p>In addition, a radiological orientation training program shall be made available to local services personnel; e.g., local emergency services/Civil Defense, local law enforcement personnel, local news-media persons.</p> | <p>Decommissioning power reactor sites typically have a level of emergency response that does not require additional response by the licensee's headquarters personnel. Therefore, the staff considers exempting licensee's headquarters personnel from training requirements to be reasonable.</p> <p>Due to the low probability of design-basis accidents or other credible events to exceed the EPA PAGs, offsite emergency measures are limited to support provided by local police, fire departments, and ambulance and hospital services, as appropriate. Local news media personnel no longer need radiological orientation training since they will not be called upon to support the formal Joint Information Center. The term "Civil Defense" is no longer commonly used; references to this term in the examples provided in the regulation are, therefore, not needed.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|--|
| <p>F.2. The plan shall describe provisions for the conduct of emergency preparedness exercises as follows: Exercises shall test the adequacy of timing and content of implementing procedures and methods, test emergency equipment and communications networks, test the public alert and notification system, and ensure that emergency organization personnel are familiar with their duties.</p> | <p>Because of the low probability of design-basis accidents or other credible events that would be expected to exceed the limits of EPA PAGs and the available time for event mitigation and offsite protective actions from a CEMP, the public alert and notification system will not be used and, therefore, requires no testing.</p> <p>Also refer to basis for 10 CFR 50.47(b).</p> |
| <p>F.2.a. A full participation exercise which tests as much of the licensee, State, and local emergency plans as is reasonably achievable without mandatory public participation shall be conducted for each site at which a power reactor is located. Nuclear power reactor licensees shall submit exercise scenarios under § 50.4 at least 60 days before use in a full participation exercise required by this paragraph 2.a.</p> <p>F.2.a.(i), (ii), and (iii) are not applicable.</p> | <p>Due to the low probability of design-basis accidents or other credible events that would be expected to exceed the limits of EPA PAGs, the available time for event mitigation and, if necessary, implementation of offsite protective actions using a CEMP, no formal offsite radiological emergency plans are required.</p> <p>The intent of submitting exercise scenarios at an operating power reactor site is to check that licensees utilize different scenarios in order to prevent the preconditioning of responders at power reactors. For decommissioning power reactor sites, there are limited events that could occur and, as such, the previously routine progression to general emergency in an operating power reactor site scenario is not applicable.</p> <p>The licensee would be exempt from 10 CFR Part 50, Appendix E, Section IV.F.2.a.(i)-(iii) because the licensee would be exempt from the umbrella provision of 10 CFR Part 50, Appendix E, Section IV.F.2.a.</p> |
| <p>F.2.b. Each licensee at each site shall conduct a subsequent exercise of its onsite emergency plan every 2 years. Nuclear power reactor licensees shall submit exercise scenarios under § 50.4 at least 60 days before use in an exercise required by this paragraph 2.b. The exercise may be included in the full participation biennial exercise required by paragraph 2.c. of this section. In addition, the licensee shall take actions necessary to ensure that adequate emergency response capabilities are maintained</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.F.2.a.</p> <p>The low probability of design-basis accidents or other credible events that would exceed the EPA PAGs, the available time for event mitigation and, if necessary, implementation of offsite protective actions using a CEMP, render a TSC, OSC and EOF unnecessary. The principal functions required by regulation can be performed at an onsite location that does not meet the requirements of the TSC, OSC or EOF.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>during the interval between biennial exercises by conducting drills, including at least one drill involving a combination of some of the principal functional areas of the licensee's onsite emergency response capabilities. The principal functional areas of emergency response include activities such as management and coordination of emergency response, accident assessment, event classification, notification of offsite authorities, and assessment of the onsite and offsite impact of radiological releases, protective action recommendation development, protective action decision making, plant system repair and mitigative action implementation. During these drills, activation of all of the licensee's emergency response facilities (Technical Support Center (TSC), Operations Support Center (OSC), and the Emergency Operations Facility (EOF)) would not be necessary, licensees would have the opportunity to consider accident management strategies, supervised instruction would be permitted, operating staff in all participating facilities would have the opportunity to resolve problems (success paths) rather than have controllers intervene, and the drills may focus on the onsite exercise training objectives.</p> | |
| <p>F.2.c. Offsite plans for each site shall be exercised biennially with full participation by each offsite authority having a role under the radiological response plan. Where the offsite authority has a role under a radiological response plan for more than one site, it shall fully participate in one exercise every two years and shall, at least, partially participate in other offsite plan exercises in this period. If two different licensees each have licensed facilities located either on the same site or on adjacent, contiguous sites, and share most of the elements defining co-located</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.F.2.a.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>licensees, then each licensee shall:</p> <p>(1) Conduct an exercise biennially of its onsite emergency plan;</p> <p>(2) Participate quadrennially in an offsite biennial full or partial participation exercise;</p> <p>(3) Conduct emergency preparedness activities and interactions in the years between its participation in the offsite full or partial participation exercise with offsite authorities, to test and maintain interface among the affected State and local authorities and the licensee. Co-located licensees shall also participate in emergency preparedness activities and interaction with offsite authorities for the period between exercises;</p> <p>(4) Conduct a hostile action exercise of its onsite emergency plan in each exercise cycle; and</p> <p>(5) Participate in an offsite biennial full or partial participation hostile action exercise in alternating exercise cycles.</p> | |
| <p>F.2.d. Each State with responsibility for nuclear power reactor emergency preparedness should fully participate in the ingestion pathway portion of exercises at least once every exercise cycle. In States with more than one nuclear power reactor plume exposure pathway EPZ, the State should rotate this participation from site to site. Each State with responsibility for nuclear power reactor emergency preparedness should fully participate in a hostile action exercise at least once every cycle and should fully participate in one hostile action exercise by December 31, 2015. States with more than one nuclear power reactor plume exposure pathway EPZ should rotate this participation from site to site.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|--|---|
| <p>F.2.e. Licensees shall enable any State or local Government located within the plume exposure pathway EPZ to participate in the licensee's drills when requested by such State or local Government.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.2.</p> |
| <p>F.2.f. Remedial exercises will be required if the emergency plan is not satisfactorily tested during the biennial exercise, such that NRC, in consultation with FEMA, cannot (1) find reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency or (2) determine that the Emergency Response Organization (ERO) has maintained key skills specific to emergency response. The extent of State and local participation in remedial exercises must be sufficient to show that appropriate corrective measures have been taken regarding the elements of the plan not properly tested in the previous exercises.</p> | <p>The U.S. Federal Emergency Management Agency is responsible for evaluating the adequacy of offsite response during an exercise. No action is expected from State or local government organizations in response to an event at a decommissioning power reactor site other than onsite firefighting, law enforcement and ambulance/medical services support. A memoranda of understanding should be in place for those services. Offsite response organizations will continue to take actions on a comprehensive EP basis to protect the health and safety of the public as they would at any other industrial site.</p> |
| <p>F.2.i. Licensees shall use drill and exercise scenarios that provide reasonable assurance that anticipatory responses will not result from preconditioning of participants. Such scenarios for nuclear power reactor licensees must include a wide spectrum of radiological releases and events, including hostile action. Exercise and drill scenarios as appropriate must emphasize coordination among onsite and offsite response organizations.</p> | <p>Due to the low probability of design-basis accidents or other credible events to exceed the EPA PAGs, the available time for event mitigation and, if needed, implementation of offsite protective actions using a CEMP, the previously routine progression to general emergency in power reactor site scenarios is not applicable to a decommissioning site. Therefore, the licensee is not expected to demonstrate response to a wide spectrum of events.</p> <p>Also refer to basis for 10 CFR Part 50, Appendix E, Section IV.1 regarding hostile action.</p> |
| <p>F.2.j. The exercises conducted under paragraph 2 of this section by nuclear power reactor licensees must provide the opportunity for the ERO to demonstrate proficiency in the key skills necessary to implement the principal functional areas of emergency response identified in paragraph 2.b of this section. Each exercise must provide the opportunity for</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.F.2.</p> |

| 10 CFR Part 50, Appendix E, Section IV | Staff Review of Licensee Justification |
|---|---|
| <p>the ERO to demonstrate key skills specific to emergency response duties in the control room, TSC, OSC, EOF, and joint information center. Additionally, in each eight calendar year exercise cycle, nuclear power reactor licensees shall vary the content of scenarios during exercises conducted under paragraph 2 of this section to provide the opportunity for the ERO to demonstrate proficiency in the key skills necessary to respond to the following scenario elements: hostile action directed at the plant site, no radiological release or an unplanned minimal radiological release that does not require public protective actions, an initial classification of or rapid escalation to a Site Area Emergency or General Emergency, implementation of strategies, procedures, and guidance developed under § 50.54(hh)(2), and integration of offsite resources with onsite justification. The licensee shall maintain a record of exercises conducted during each eight year exercise cycle that documents the content of scenarios used to comply with the requirements of this paragraph. Each licensee shall conduct a hostile action exercise for each of its sites no later than December 31, 2015. The first eight year exercise cycle for a site will begin in the calendar year in which the first hostile action exercise is conducted. For a site licensed under Part 52, the first eight year exercise cycle begins in the calendar year of the initial exercise required by Section IV.F.2.a.</p> | |
| <p>I. By June 20, 2012, for nuclear power reactor licensees, a range of protective actions to protect onsite personnel during hostile action must be developed to ensure the continued ability of the licensee to safely shut down the reactor and perform the functions of the licensee's emergency plan.</p> | <p>Refer to basis for 10 CFR Part 50, Appendix E, Section IV.E.8.d.</p> |