

March 9, 2000

For: The Commissioners  
 From: James L. Blaha, Assistant for Operations, Office of the EDO /RA/  
 Subject: SECY-00-0060 - WEEKLY INFORMATION REPORT - WEEK ENDING MARCH 3, 2000

<b>Contents</b>	<b>Enclosure</b>
<a href="#">Nuclear Reactor Regulation</a>	A
<a href="#">Nuclear Material Safety and Safeguards</a>	B
<a href="#">Nuclear Regulatory Research</a>	C
<a href="#">Analysis and Evaluation of Operational Data</a>	D
General Counsel	E*
Administration	F*
<a href="#">Chief Information Officer</a>	G
Chief Financial Officer	H*
<a href="#">Human Resources</a>	I
Small Business & Civil Rights	J*
Enforcement	K*
State Programs	L*
<a href="#">Public Affairs</a>	M
International Programs	N*
<a href="#">Office of the Secretary</a>	O
<a href="#">Region I</a>	P
<a href="#">Region II</a>	P
<a href="#">Region III</a>	P
<a href="#">Region IV</a>	P
Executive Director for Operations	Q*
<a href="#">Congressional Affairs</a>	R

\*No input this week

*/RA by Debra J. Corley acting for/*

James L. Blaha  
 Assistant for Operations, OEDO

Contact: S. Peterson, OEDO

ENCLOSURE A

Office of Nuclear Reactor Regulation  
 Items of Interest  
 Week Ending March 3, 2000

**Fitzpatrick and Indian Point 3**

On February 24, 2000, Dominion Resources, the parent company of VEPCO, made an unsolicited bid for the purchase of the Fitzpatrick and Indian Point 3 nuclear plants. The Dominion resources bid of about \$686 million is some \$75 million larger than the bid made by Entergy Corporation. The New York Power Authority has said that they will take the Dominion Resources bid into consideration. Both the New York Power Authority and Entergy boards of directors were scheduled to vote on the proposed sale of the plants to Entergy this week.

**Indian Point 2**

There was a steam generator technical meeting held on March 1, 2000, at Indian Point 2 site between Con Ed and NRC staff. The purpose of the meeting was to discuss the following matters related to the IP2's steam generator: steam generator inspection and testing program, nuclear industry standard for steam generators, onsite replacement steam generators, and post inspection findings and restart criterion. The meeting was conducted in two parts. The first was a formal meeting between Con Ed and NRC which included a technical question and answer session. The second half of the meeting was a question and answer session between NRC staff and members of the public. The majority of the questions received were related to steam generators. However, questions were received relating to the February 15, 2000, event. These were deferred to next public meeting. There were approximately 130 individuals present; of those about 30 were individuals not affiliated with nuclear power. An additional public meeting will be scheduled prior to restart to discuss inspection results and corrective

actions on steam generators.

## **Meeting with NEI to Discuss Risk Informing Part 50 Option 2**

On February 23, 2000, the Office of Nuclear Reactor Regulation (NRR) held a public meeting with the Nuclear Energy Institute (NEI) to discuss the NRC's plans for developing a proposed rule for risk-informing the special treatment requirements of [10 CFR Part 50](#) (RIP-50) and industry initiatives to support the rulemaking. Representatives of the Office of Nuclear Regulatory Research, the American Society of Mechanical Engineers, a number of reactor licensees, consultants, and others also attended and participated in the meeting.

The staff discussed the status of its efforts and its overall plans, including its plans to assess commercial standards and commercial-grade treatment. NEI described its efforts to develop guidance for classifying the safety significance and risk importance of structures, systems, and components (risk-informed safety categories) and for implementing changes to the special treatment requirements. NEI also discussed its efforts to solicit pilot plants. To date, the Boiling Water Reactor Owners Group has expressed interest in conducting "systems pilots" for two boiling water reactor systems and two licensees have expressed interest in participating as "whole-plant" pilots. No pilot activities have been confirmed. The staff stressed the importance of integrating pilot plant activities into the overall plan for developing the proposed rule and that the pilot activities should be conducted within a time frame that will support the rulemaking. NEI indicated that it is the industry's general view that better understanding of how treatment will be handled and favorable progress on the staff's review of the South Texas Project (STP) exemption requests will be needed before licensees commit to "whole-plant" pilot activities. NEI and several of the licensees in attendance suggested that favorable progress on the STP exemption requests is essential to demonstrate the viability of the RIP-50 effort itself.

General agreements reached between the staff and NEI include: (1) it will be important to document a clear understanding of commercial-grade practices and a reasonable commercial assurance standard (NEI plans to do so in its guidance document), (2) more effort and focus needs to be applied to the special treatment requirements than to the categorization process, and (3) "selective implementation" requires further consideration and discussion.

The staff and NEI agreed to continue to hold regular meetings and has scheduled the next meeting for March 30, 2000. At that meeting, NEI will give the staff a preliminary draft of its guidance document and will further discuss potential pilot plant activities.

The staff is continuing to develop a detailed action plan and schedule for this rulemaking. The staff will reflect in its plan the industry activities and any potential impacts caused by delays in developing the industry guidance document and initiating pilot plant activities. The staff is preparing a memorandum to inform the Commission that it is assessing the potential impacts of the delay in the industry guidance document and pilot plant activities on its schedule for developing a proposed rule.

## **University of Texas - Reflector Problem Update**

On February 25, 2000, the staff of the University of Texas successfully vented gases that had accumulated in their reflector assembly. The NRC project manager was onsite to witness the venting. The Texas staff used a special tool that was designed to drill through the reflector top and measure the pressure inside the reflector then allow the gas to be released to the pool in a controlled manner. Ventilation at the top of the pool carried the gases outside the building. The pressure inside the reflector was in the 110 to 130 psi range. It took over five hours to relieve the pressure. The licensee has seen a change in the geometry of the reflector. The bulging of the reflector that was the initial indication to the licensee of the problem has subsided. The licensee plans to drill additional holes to ensure that all gases have been released and that the reflector is filled with deionized reactor pool water to reduce the potential for further gas generation. The licensee will perform an evaluation under [10 CFR 50.59](#) of operation with the reflector flooded with water. The largest impact appears to be a significant decrease in flux in the beam tube experimental facilities. The licensee kept the non-power reactor community and the NRC informed of developments during the resolution of this problem. REXB will consider the need for a generic communication on this problem.

## **Arkansas Nuclear One, Unit 1, License Renewal Application**

On February 1, 2000, Entergy Operation, Inc. (Entergy), the licensee for Arkansas Nuclear One, Unit 1 (ANO-1), delivered its application requesting the extension of the ANO-1 reactor operating license for an additional 20 years beyond its current 40-year term pursuant to [10 CFR Parts 54](#) and [51](#).

The NRC staff has determined that Entergy has submitted an application for renewal of Operating License, DPR-51, for ANO-1 that is acceptable and sufficient for docketing in accordance with [10 CFR 54.19](#), [54.21](#), [54.22](#), [54.23](#), and [51.53\(c\)](#).

A notice of acceptance for docketing of the license renewal application, and a notice of opportunity for a hearing regarding the renewal of the applicant's license was forwarded on February 28, 2000, to the Office of the Federal Register for publication.

## **Edwin I. Hatch, Units 1 and 2, License Renewal Application**

In a letter dated February 29, 2000, Southern Nuclear Operating Company, Inc. (SNC), the licensee for the Edwin I. Hatch Nuclear Plant, Units 1 and 2 (HNP), submitted its application requesting the extension of the operating licenses for HNP for an additional 20 years beyond their current 40-year terms, pursuant to [10 CFR Parts 51](#) and [54](#).

The contents of SNC's application include a technical report describing how the effects of aging will be managed during the renewal term for the systems, structures, and components within the scope of [10 CFR Part 54](#). Additionally, the application

includes an environmental report.

Both units of HNP are General Electric nuclear steam supply systems originally licensed for commercial operation in 1975 and 1979, respectively. The current operating licenses for HNP expire on August 6, 2014 and June 13, 2018, respectively.

### **Reactor Oversight Process Workshop**

The Inspection Program Branch conducted the first public workshop on the revised reactor oversight process in Region III from February 22-24, 2000. About 150 people, including many representatives from utility, industry, State, NRC, and members of the public, attended the workshop. At this workshop, headquarters and regional representatives focused on the key attributes of the new oversight process and the associated program documents, and answered questions from the audience. Similar workshops in other regions will be completed by March 23, 2000.

### **Maintenance Rule Workshop on Maintenance Effectiveness**

On February 22 and 23, 2000, members of the Quality Assurance, Vendor Inspection, Maintenance and Allegations Branch (IQMB) held a Maintenance Rule Workshop on Maintenance Effectiveness with NRC staff from the four regional offices, the Office of Enforcement, and other NRR program offices whose programs are dependent on proper licensee implementation of the Maintenance Rule. The workshop theme was the role of the Maintenance Rule in other agency programs. To that end, workshop speakers discussed the new reactor oversight process, shutdown risk, technical specifications, and license renewal. Related discussion topics included the revised rule and guidance, unavailability and reliability monitoring, and updated enforcement guidance. Regional DRS staff presented the status of inspector training for the maintenance rule and maintenance effectiveness inspections.

---

ENCLOSURE B

Office of Nuclear Material Safety and Safeguards  
Items of Interest  
Week Ending March 3, 2000

### **Presentation at the National Bioethics Advisory Commission Meeting**

On March 1, 2000, a staff member from the Division of Industrial and Medical Nuclear Safety made a presentation to the National Bioethics Advisory Commission (NBAC) on "Regulation of Nuclear Material by the Nuclear Regulatory Commission (NRC)." The NBAC is considering alternative federal regulatory systems to the current human subjects protection system. Therefore, NBAC wanted to hear about NRC's authority and regulatory structure for ensuring the safe use of nuclear materials in the U.S. NBAC was interested particularly in the relationship between NRC and other federal agencies, and in how NRC develops and enforces its regulations.

### **Meeting on Proposed Spent Fuel Storage Facilities at Diablo Canyon and Humboldt Bay**

On March 1, 2000, staff from the Spent Fuel Project Office and the Office of Nuclear Reactor Regulation met with Pacific Gas and Electric Company (PG&E) staff to discuss PG&E's plans for licensing independent spent fuel storage installations (ISFSIs) at Diablo Canyon and Humboldt Bay. PG&E currently plans to select a cask vendor(s) by May 2000 and submit site-specific [10 CFR Part 72](#) license applications in October 2000.

PG&E staff described planned approaches to performing seismic analyses at the two sites. For Diablo Canyon, the seismic analysis will evaluate the acceptance criteria for all four existing design basis earthquakes to meet 10 CFR Part 50 licensing requirements. Soil investigations have been performed to confirm the similarity of the soils underlying the power plant and the proposed ISFSI site.

For Humboldt Bay, a combination of probabilistic and deterministic design analysis methods will be used. Probabilistic insights will be applied to the spent fuel loading and transfer activities, based on their short duration and the reduced hazard presented by fuel that has been cooled for more than 24 years. Deterministic analyses will be used to establish the design criteria of the storage system. PG&E and the Nuclear Regular Commission staff agreed to hold additional meetings in the interest of ensuring high quality applications.

### **International Meeting of Working Group on Physical Protection**

During the week of February 21-25, 2000, a representative from the Division of Fuel Cycle Safety and Safeguards served as a member of the U.S. delegation in an International Atomic Energy Agency (IAEA) working group meeting in Vienna, Austria. This working group was formed as a result of the IAEA experts meeting in November 1999 to consider whether there is a need to revise the Convention on the Physical Protection of Nuclear Material to, among other things, impose INF/CIRC-225, Revision 4, on member states. The working group established an agenda and aggressive work plan to provide recommendations for a follow up experts meeting by May 2001. The Nuclear Regulatory Commission will continue to participate in the various work plan activities to ensure that U.S. views are adequately addressed.

### **Public Meeting Regarding General Electric Vallecitos Nuclear Center**

On February 24, 2000, representatives from the Division of Fuel Cycle Safety and Safeguards, the Spent Fuel Project Office,

the Office of Nuclear Reactor Regulation, and Region IV met with members of the public, representatives from local government, General Electric, and regulators from the state of California, Department of Health Services Radiological Control Division, at the Pleasanton Public Library in the San Francisco area. The meeting focused on concerns about transportation of irradiated fuel samples to and from the Vallecitos Nuclear Center (VNC), renewal of the VNC Special Nuclear Material (SNM) license, the Nuclear Regulatory Commission's (NRC's) oversight of VNC operations, and decommissioning of the shut-down reactors. The meeting was arranged by NRC to respond to questions raised by stakeholders, including petitioners who requested a hearing on the SNM license renewal. The Atomic Safety and Licensing Board Panel dismissed the hearing request in a decision issued on February 17, 2000.

NRC presenters discussed the role of NRC, the SNM license renewal process, the NRC inspection oversight program, the NRC reactor licensing and decommissioning program, and the regulation of transportation activities. After the presentations and audience questions were completed, all 28 questions e-mailed by the petitioners were answered. At the meeting, the stakeholders raised concerns about over reliance on web-based systems to inform the public and requested NRC to establish mechanisms to share information about VNC on a routine and timely basis. The meeting was video recorded by Community Cable TV.

---

ENCLOSURE C

Office of Nuclear Regulatory Research  
Items of Interest  
Week Ending March 3, 2000

### **Dry Cask Storage Characterization Program**

In 1985, 21 pressurized water reactor fuel assemblies from the Surry nuclear power plant were placed in dry cask storage at the Idaho National Engineering and Environmental Laboratory in a Castor-V/21 cask. During the week of February 21, 2000, 12 rods were removed from one assembly and visually inspected to observe the long-term effects of dry storage. No evidence of corrosion, major deformations, or unexpected conditions were observed. Shortly, the fuel rods will be sent to the Argonne National Laboratory for detailed examinations.

The visual inspection of the Castor-V/21 steel cask, cask internals, and contents began in September 1999 in support of an NMSS User Need Request. This RES program is expected to provide the technical bases for license renewal decisions pertaining to independent spent fuel storage installation (ISFSI) applications. An application from Virginia Power for the renewal of the Surry Nuclear Power Plant ISFSI license is anticipated in June 2001.

In addition to the NRC, other participants in this cooperative program include the Electric Power Research Institute, the Department of Energy Offices of Civilian Radioactive Waste Management and Environmental Management, and the Department of Energy-Idaho Operations Office.

### **Public Workshop on Performance-Based Regulatory Approaches**

A public workshop was held on March 1, 2000, in the NRC auditorium to discuss high-level guidelines proposed for performance-based activities. The guidelines were published on January 24, 2000, in the *Federal Register* (65 FR **EXIT** 3615). Participants at the workshop included representatives from NRC licensees, stakeholders, and public interest groups. All three arenas of NRC activities (reactors, materials and waste) were represented.

The NRC staff presented an overview of the performance-based regulatory initiative and solicited input on the guidelines and questions published in the Federal Register notice. The all-day workshop provided time for extensive discussion by the participants. As the public comment period for the guidelines ends on March 24, 2000, it is expected that many of the views expressed during the workshop will also be submitted as written comments. The comments and input received will be used to draft plans on developing performance-based initiatives. One more workshop is planned before the guidelines are finalized and plans are presented to the Commission.

### **Final Review Meeting on RES Program, "Evaluation of Degradation of BWR Internals"**

The final technical review meeting of this RES program took place on February 8, 2000 in the NRC headquarters. The Principal Investigator from the Idaho National Engineering and Environmental Laboratory (INEEL) made a presentation to the NRC staff (from NRR and RES) about the findings of this program.

This program was established in response to a user need request from NRR. The objective of this program was to assess the potential risk associated with failures of BWR reactor internal components induced by intergranular stress corrosion cracking (IGSCC), both singly and in combination with the failure of other reactor vessel internal components. The study focused on the mechanical design of vessel internals, potential failure locations, failure consequences (e.g., cascading effects), potential accident scenarios, and the characterization of risk.

This program was conducted in two phases. Phase I documented available information on degradation of BWR reactor internal components and identified 148 potentially important accident scenarios associated with IGSCC-induced failures. Because of the large number of potential component failures and accident sequences, design diversity among the BWR reactor types, difficulty in estimating crack growth rates, and lack of failure data to assist in estimating probabilities, a multi-discipline NRC Review Panel with members from both RES and NRR (with expertise in reactor systems, PRA, materials, mechanical, and

structural engineering) was formed to provide guidance and to review the program on a regular basis. Initially, the Phase II program was to investigate a representative single plant (a BWR/4 plant) with potential failure modes and risk consequences of a single component type (jet pumps). Two outside expert panels on Materials and Structural/PRA were also formed to provide expert opinions. After completing the initial study for failure of jet pumps, the methodology was also applied to failure of other reactor internal components and the cascading effects. This was completed in December 1999.

A number of BWR reactor internal components, if allowed to be severely degraded by IGSCC, could fail either in a common mode or in a cascading manner under a severe internal or external event (large LOCAs and/or earthquakes), and could lead to an unacceptable increase in the plant core damage frequency (CDF). However, it was concluded from the program that, with the current program of inspection, monitoring, and repair implemented by the industry group and NRR's regulation and review, no significant increase should occur in CDF caused by IGSCC failures of BWR reactor internal components.

### **Public Meeting on Failure of Low-Voltage I & C Cables During LOCA Tests**

In support of the resolution of GSI-168, "Environmental Qualification of Electric Equipment," RES is conducting a series of LOCA tests on low-voltage I&C cables at Wyle Laboratories. The tests involve new, artificially aged, and naturally aged samples of cables. The cables are both single conductor and multi-conductor configurations and involve bonded and unbonded jacket construction. The results for LOCA Test #5 showed unacceptable degradation of the jacket and insulation for bonded, single conductor cables (manufactured by the Okonite Company) that were aged to simulate 20 and 40 years of service. The failed cables were of Hypalon jacket construction bonded to Ethylene Propylene Rubber (EPR) insulation.

More than 50 representatives from NEI, NUGEQ (Nuclear Group on Environmental Qualification), utilities, manufacturers, A/Es, EPRI consultants, Brookhaven National Laboratory, and the staff (both RES & NRR) participated in a public meeting on February 16, 2000 to discuss the results of RES-sponsored environmental qualification testing.

The NUGEQ representatives agreed to survey their member utilities to find applications of EPR/Hypalon bonded cables in harsh environments. Utility representatives and the EPRI consultant stressed that the majority of cables that could be in question would not experience the harsh environments of the qualification testing. In addition, these representatives stated that cases of significant cable degradation caused by thermal or irradiation aging were typically outside containment and that such cables would be replaced. Periodic I&C cable replacements to main steam isolation valve limit switches were cited as an example. The industry representatives suggested that the NRC staff should consider issuing an Information Notice.

---

ENCLOSURE D

Incident Response Operations  
Items of Interest  
Week Ending March 3, 2000

### **Preliminary Notifications:**

1. PNO-I-00-006, Columbia University, DELIBERATE INTERNAL CONTAMINATION OF STUDENT
2. PNO-I-00-007, Allegheny General Hospital, AN AMERICIUM-241 SEALED SOURCE MISSING FROM AN IMAGING CAMERA
3. PNO-II-00-009, SRB Technologies, MISSING EXIT SIGNS
4. PNO-II-00-010, Westinghouse Electric Corporation, IDENTIFICATION OF ONSITE CONTAMINATION
5. PNO-III-00-006, Wisconsin Electric Power Co., POINT BEACH UNIT 1 SHUT DOWN TO CHECK FOR A LOOSE PART STEAM GENERATOR
6. PNO-II-00-011, Halifax Medical Center, MEDICAL MISADMINISTRATION
7. PNO-III-00-008, Solar Testing Laboratories, STOLEN MOISTURE-DENSITY GAUGE
8. PNO-IV-00-008, International Uranium (Usa) Corporation, EMPTY TRANSPORTATION CONTAINERS REPORTED EXCEEDING DEPARTMENT OF TRANSPORTATION CONTAMINATION LIMITS

---

ENCLOSURE G

Chief Information Officer  
Items of Interest  
Week Ending March 3, 2000

### **Freedom of Information and Privacy Act Requests received during the 5-Day Period of February 25, 2000 through March 2, 2000:**

OIG case 99-41G.

(FOIA/PA 2000-0147)

Department of State record referral.	(FOIA/PA 2000-0148)
Combustion Engineering, Inc., license #06-00217-06, all non-public records.	(FOIA/PA 2000-0149)
Perry plant, OI report 3-98-007, all records provided in response to FOIA/PA 1999-0244.	(FOIA/PA 2000-0150)
Enforcement Action and Individual Action records related to EA-99-067 & IA-99-021 respectively.	(FOIA/PA 2000-0151)
Rep. James Maloney, Connecticut, all correspondence, 1/97 to present.	(FOIA/PA 2000-0152)
MLTS database.	(FOIA/PA 2000-0153)

ENCLOSURE I

Office of Human Resources  
Items of Interest  
Week Ending March 3, 2000

**Savings Bonds Campaign Kick-off Breakfast and Workshop Attended**

On February 29, 2000, Henry Rubin attended the Greater Washington Area 2000 Campaign Kick-Off Breakfast and Workshop held at the U.S. Chamber of Commerce, in Washington, D.C. The guest speaker was Russell Turner, President and CEO, United Space Alliance and National Savings Bonds Chairman. NRC will be conducting its annual savings bonds campaign this Spring. Additional information regarding savings bonds may be obtained on the internet at [www.savingsbonds.gov](http://www.savingsbonds.gov).

Arrivals		
BENNETT, Margaret	SECRETARY (OA)	RES
TARDIFF, Albert	SECURITY SPECIALIST*	NRR
Retirements		
CHAPPELL, Ross	SECTION CHIEF	NMSS
McPHERSON, Donald	SR THERMAL HYDRAULICS & TESTING EXPERT	RES
TEN EYCK, Elizabeth	DIRECTOR	NMSS

\* Effective 2/28/00

ENCLOSURE M

Office of Public Affairs  
Items of Interest  
Week Ending March 3, 2000

**Media Interest**

Chairman Meserve was interviewed by The Hill newspaper (serving Capitol Hill).

There was media interest in the Calvert Cliffs meeting at headquarters on license renewal.

There was some press interest in leap year problems at reactors, and in the contamination incident at Columbia University in New York.

The New Orleans Times-Picayune interviewed Robert Wood, NRR, for an article on Entergy's acquisition plans.

The Los Angeles Times and the North County Times, Calif., interviewed Bill Huffman, NRR, for articles on San Onofre's decommissioning plans.

Press Releases	
<b>Headquarters:</b>	
<u>00-34</u>	NRC Seeking Public Comment on Draft Strategic Plan
<u>00-35</u>	Working Group Meeting on Control of Solid Materials Cancelled
<u>00-36</u>	NRC Advisory Committee on Reactor Safeguards to Meet in Rockville, Maryland, March 15
<u>00-37</u>	NRC Advisory Committee on Reactor Safeguards to Meet in Rockville, Maryland, on March 1
<u>00-38</u>	NRC Advisory Committee on Reactor Safeguards to Meet in Rockville, Maryland, March 14-15

<b>Regions:</b>	
I-00-19	NRC to Meet With Consolidated Edison Tomorrow Night to Discuss IP2 Steam Generator Examination
II-00-14	NRC to Hold Workshop on its New Reactor Inspection and Assessment Program
IV-00-4	NRC to Hold Workshop on its New Reactor Inspection and Assessment Program

ENCLOSURE O

Office of the Secretary  
Items of Interest  
Week Ending March 3, 2000

	Document Released to Public	Date	Subject
<b>Decision Documents</b>			
1.	<a href="#">SECY-00-0049</a>	2/24/00	Results of the Revised Reactor Oversight Process Pilot Program
2.	<a href="#">M000302A</a>	3/2/00	Staff Requirements - Affirmation Session, March 2, 2000 SECY-00-0028 and SECY-00-0036
<b>Information Papers</b>			
1.	<a href="#">SECY-00-0046</a>	2/23/00	SECY-00-0046 Weekly Information Report - Week Ending February 18, 2000
2.	<a href="#">SECY-00-0038</a>	2/15/00	The 1999 NRC Annual Report
3.	<a href="#">SECY-00-0041</a>	2/14/00	Use of Rubblized Concrete Dismantlement to Address 10 CFR Part 20, Subpart E, Radiological Criteria for License Termination
5.	<a href="#">SECY-00-0054</a>	3/1/00	SECY-00-0054 Weekly Information Report - Week Ending February 25, 2000
<b>Memoranda</b>			
1.	Memo to W. Travers fm Comm. McGaffigan	3/1/00	Follow-up Questions to the Public Commission Briefing on the Status of Spent Fuel Projects Held on February 23, 2000
2.	Memo to Commissioners fm Chmn. Meserve	2/18/00	Supplemental Cost Information on SECY-99-244 - Location of the NRC's Technical Training Center and Appropriate Number of Simulators
3.	<a href="#">M000223A</a>	3/3/00	Briefing on Status of Spent Fuel Projects

**Commission Correspondence**

1. Letter to Congress dated February 24, 2000, to advise of the unanimous decision of the NRC concerning the Technical Training Center in Chattanooga, Tennessee.
2. Letter to Congress dated February 23, 2000, provides the January monthly report on the status of NRC's licensing and regulatory duties.
3. Letter to John Carlson, Australian Safeguards and Non-Proliferation Office, concerns a visit by a U.S. team of experts to the SILEX Systems Limited facility before the second week of March 2000.

**Federal Register Notices Issued**

1. [10 CFR Parts 21, 50, 52, 54 and 100](#); Risk-Informing Special Treatment Requirements; Advance Notice of Proposed Rulemaking.
2. Report to Congress on Abnormal Occurrences; Fiscal Year 1999; Dissemination of Information.
3. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Thermal-Hydraulic Phenomena; Revised.
4. Application for a License to Export Radioactive Waste (Transnuclear, Inc.).

ENCLOSURE P

Region I  
Items of Interest  
Week Ending March 3, 2000

**Briefing of Congressman LoBiondo**

On March 2 NRC representatives briefed Congressman Frank LoBiondo (NJ 2<sup>nd</sup> district) on topics of interest regarding the Salem and Hope Creek plants. In response to the fire protection team inspection report issued on February 14, 2000, the Congressman had visited the site on February 22, had been briefed by PSEG on fire protection issues, and had toured portions of the plant. On March 2, in the Congressman's office, Elinor Adensam and Patrick Madden of NRR, Glenn Meyer of Region I, and Tom Combs of Congressional Affairs briefed the Congressman and his chief-of-staff on fire protection findings at Salem, operational performance of Salem and Hope Creek, and the pilot program of the revised reactor oversight program. In addition to prepared remarks on these topics, the NRC representatives addressed questions on the Indian Point 2 steam generator tube leak and the GAO survey of NRC staff. The briefing lasted approximately 30 minutes. The Congressman stated his intention to visit the site and be briefed by NRC approximately twice a year, in order to remain current on concerns raised by his constituents.

### **Confirmatory Inspection Public Exit Meeting with GPU Nuclear, Forked River, New Jersey**

On February 29, 2000, NRC staff conveyed to GPU Nuclear (the Oyster Creek Nuclear Generating Station (OCNGS) licensee) the results of the NRC's radiological assessment of the Forked River Property at a public exit meeting held in Forked River, NJ. This property is a 657-acre area adjacent to OCNGS that may be sold by GPU Nuclear. The meeting was open to the public and included a question-and-answer period. In addition to the licensee representatives, five representatives from the New Jersey Department of Environmental Protection and four members of the public, including two news reporters attended the meeting. The inspection found that GPU adequately characterized the Forked River Property and conducted adequate surveys to demonstrate that any residual radioactive material from OCNGS-related activities were well below the release criteria of 10 CFR 20, subpart E (i.e., 25 mrem/yr). The soil samples analyzed by the NRC's contractor, and independently by the State of New Jersey, did not find any plant-related materials that were distinguishable from background, although the State did identify naturally occurring activity (radium) in some groundwater samples that may exceed EPA drinking water standards.

---

ENCLOSURE P

Region II  
Items of Interest  
Week Ending March 3, 2000

### **FBI/NRC Meeting in Johnson City, Tennessee**

On March 1, 2000, representatives of Region II, the Incident Response Operations (IRO), and the Office of Nuclear Material Safety and Safeguards (NMSS) met with representatives of the Federal Bureau of Investigations (FBI **EXIT**) and Nuclear Fuel Services (NFS) in the FBI Office near Johnson City, Tennessee. The purpose of the meeting was to plan for an FBI/NRC limited field exercise with NFS on August 16, 2000.

### **Duke Energy Corporation - Oconee License Renewal Exit Meeting**

On March 3, 2000, the Regional Administrator and selected staff attended the Oconee License Renewal inspection exit meeting in Seneca, South Carolina. The exit was for the third and final inspection of license renewal issues at the Oconee site.

### **Carolina Power and Light - Brunswick Plant**

On March 3, 2000, at 9:50 a.m., the licensee declared an unusual event and reported this to the NRC and the State of North Carolina. The basis of the unusual event was a loss of offsite power to the 4 kV emergency, electrical buses for Unit 1. Unit 1 was in cold shutdown for a refueling outage at the time of the event. The licensee's emergency diesels started and provided emergency power to the buses until the offsite power source was recovered at 6:40 p.m. The licensee exited the Notice of Unusual Event, emergency classification at this time.

Region II staffed the regional incident response center to effectively monitor the licensee's response to the loss of offsite power and associated plant activities. The regional incident response center was deactivated at 7:00 p.m., on March 3. The licensee issued two press releases regarding the event. Region II issued a preliminary notification and responded to inquiries from State and local agencies and the media.

---

ENCLOSURE P

Region III  
Items of Interest  
Week Ending March 3, 2000

### **Staff Training on New Inspection and Assessment Program**

Region III staff members received extensive training February 28 - March 3, 2000, on the implementation of the new Reactor Inspection and Assessment Program.

---

ENCLOSURE P

Region IV  
Items of Interest  
Week Ending March 3, 2000

**Cooper Nuclear Station Management Public Meeting**

Senior managers from the Nebraska Public Power District met with NRC, Region IV managers on February 28 at the Brownville, Nebraska, concert hall. The meeting, open to public observation, was held to discuss Cooper lessons learned from recent problems in communicating information to the NRC. The problems occurred in a licensing submittal, during discussions about a Notice of Enforcement Discretion, and in several other minor instances. The utility managers provided frank discussion that revealed insight into the underlying causes of the miscommunications. The causes included lack of appropriate oversight and not following established processes in preparing for discussions on key issues. Nebraska Public Power District managers participating in the meeting included the President and Chief Executive Officer, the Vice President of Nuclear Generation, the Plant Manager, and the Senior Manager of Engineering. The Director, Division of Reactor Projects, Region IV, the Chief, Reactor Projects Branch C, the NRR Project Manager, and the resident inspectors also participated in the meeting.

ENCLOSURE R

Office of Congressional Affairs  
Items of Interest  
Week Ending March 3, 2000

<b>CONGRESSIONAL HEARING SCHEDULE, No. 6</b>					
OCA CONTACT	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Portner	03/08/00 2362 RHOB	10:00	Secretary Richardson	DOE's FY01 Budget	Reps. Packard/Visclosky Energy & Water Development Appropriations
Portner	03/09/00 SD-406	10:00	Commission and Others	FY01 NRC Authorization and Oversight	Senators Inhofe/Graham Clean Air, Wetlands, Private Property and Nuclear Safety Environment and Public Works
Gerke	03/09/00 SD-342	10:00	GAO, OPM	Status of Federal Employees	Senators Voinovich/Durbin Oversight of Gov't Mgmt, Restructuring & the District of Columbia Governmental Affairs
Portner	03/16/00 2362 RHOB	10:00	Office of Nuclear Energy	FY01 Budget	Reps. Packard/Visclosky Energy and Water Development Appropriations
Combs	03/21/00 2362 RHOB	10:00	OCRWM	FY01 Budget	Reps. Packard/Visclosky Energy and Water Development Appropriations
Combs	03/22/00 SD-342	10:00	TBA	DOE's Management of Health & Safety Issues at GDP's	Senators Thompson/Lieberman Governmental Affairs
Gerke	03/31/00 TBA	TBA	TBA	Accuracy of Federal Financial Audits	Reps. Horn/Turner Gov't Mgmt, Info & Technology Governmental Affairs
Gerke	04/4/00 TBA	TBA	TBA	Effectiveness of GPRA	Reps. Horn/Turner Gov't Mgmt, Info &

