

October 15, 1998

For: The Commissioners  
 From: James L. Blaha, Assistant for Operations, Office of the EDO /s/  
 Subject: WEEKLY INFORMATION REPORT - WEEK ENDING OCTOBER 9, 1998

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\*No input this week

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ENCLOSURE A

Office of Nuclear Reactor Regulation  
 Items of Interest  
 Week Ending October 9, 1998

**Status of Old Licensing Actions**

Over the past few weeks extensive interaction has taken place within Associate Director for Projects (ADP) focusing on any concerns, problems or issues that may require management attention to begin reducing other inventory of licensing actions at an expedited pace. There are approximately 400 licensing actions greater than 1-year old. In the various meetings, management emphasis was placed on ensuring that the WISP database information is accurate and reflects, accurately, schedules and licensing actions requiring attention to reach timely resolution. Currently, a large portion of the licensing actions greater than 1-year old are tentatively scheduled to be closed within the next three months (about half). Only a few items have been identified that have reached an impasse that will require management attention. A follow up meeting will be held with the Project Director's during the first week of December to ensure we are still on track to complete the older licensing actions as scheduled. ADP will also undertake special initiatives on licensing

actions that are less than one year old, as well as the licensing actions being docketed in FY 1999. A schedule for this initiative will be established by the 1st of November.

### **Meeting with Sequoyah on EDG AOT Extension**

On September 30, 1998, the Probabilistic Safety Assessment Branch (SPSB) attended a meeting at headquarters with the representatives from the Tennessee Valley Authority (TVA). The purpose of the meeting was to discuss the issues associated with the staff review of proposed allowed outage time (AOT) extension for emergency diesel generator(s) (EDG)s. The licensee indicated that it would implement the configuration-risk management program (CRMP), consistent with the [Regulatory Guide 1.177](#). It also expressed the urgent need for the staff approval of the request to perform the upcoming 12-year inspection in January 1999, at power. The licensee indicated that multiple AOTs would be entered to perform the 12-year inspection at power, driving the EDG unavailabilities higher. Dual unit shutdown would be required if the licensee were to perform the inspection during shutdown. Currently, there are no major issues remaining, but further staff review is required to complete the evaluation.

### **SPSB Meeting with WOG**

SPSB staff (A. El-Bassioni and S. Long) supported the Division of Engineering (DE) in a meeting with the Westinghouse Owners Group (WOG) concerning the adequacy of their inspections of the dissimilar metal welds on part-length control rod drive housings. A very large, fabrication related flaw was found in one such weld at Prairie Island when it started to leak after 22 years of service. Affected licensees have now inspected 36% of the similar welds still in service. The WOG and Nuclear Regulatory Commission (NRC) agree that the risk due to the remaining uninspected welds is low, due to the confidence derived from the inspections, the small number of such welds per reactor, and the fact that they have survived at least 22 years of service already. However, agreement has not yet been reached on what constitutes the appropriate level of reinspection, given that the initial (100%) fabrication inspections were demonstrated to be ineffective in the case of the defect found at Prairie Island.

### **One Time Emergency Electrical Power Test at Oconee**

The Reactor Systems Branch (SRXB), Electrical Engineering Branch (EELB), and the Plant Systems Branch (SPLB) are reviewing an Oconee unreviewed safety question (USQ). The amendment request, associated with the USQ, will allow Oconee to conduct a one-time test of their emergency electrical power distribution system to determine if block loading the Keowee generator is preferred at 60% of rated voltage or 90% of rated voltage. The test involves operators removing offsite power to Unit 3 and initiating an Engineered Safeguards signal to simulate a simultaneous loss-of-coolant accident (LOCA) - loss-of-offsite power (LOOP). The test will result in three high-pressure injection (HPI) pumps, two low-pressure (LPI) pumps injecting into a fueled and vented-reactor coolant system (RCS) with the reactor head installed and torqued. Units 1 and 2 will remain at full power during the event. The submittal indicates that this special test is needed because, in evaluating a system modification to load the Keowee generators at 90% rated voltage, questions arose concerning frequency overshoot of the Keowee units. Information obtained from these special tests will allow evaluation of the frequency overshoot concerns.

### **North Anna Unit 1 Fuel Failures**

During the present refueling outage, it was determined that there were 19 failed (leaking) fuel rods in 9 assemblies in the completed cycle at North Anna Unit 1. Coolant activity had indicated fuel failures, but the number was unknown until the outage. These failures were all in third-cycle, ZIRLO clad fuel, located near the baffle. The root cause investigation is underway, with the most likely cause being grid to rod fretting at the mid grids. These failures appear to be similar to vibration related failures that were experienced in the 1993 time frame at two other plants. The North Anna fuel had incorporated the rotated grids which was the fix for the previous problem. All affected assemblies were scheduled for discharge during this outage. Virginia Power has installed vibration suppression devices on all peripheral locations for the upcoming cycle. The Reactor Systems Branch (SRXB) will continue to follow this issue.

### **Participation in the Nuclear Utilities Software Management Group Meeting and Symposium on the Year 2000 Problem in Nuclear Power Plants**

From October 7 - 9, 1998, staff from the Instrumentation and Controls Branch (HICB) participated in the semiannual Nuclear Utilities Software Management Group (NUSMG) meeting and symposium on the Year 2000 (Y2K) problem in nuclear power plants in Wilmington, NC. Most nuclear power plant licensees were in attendance at the meeting. The meeting included presentations by industry representatives on a number of topics related to the use of digital (software-based) systems in nuclear power plants including graded approaches to software quality, software documentation requirements and the Y2K problem including assessment activities, embedded chip testing and contingency planning. Deirdre Spaulding of the HICB staff made a presentation on the current status of the NRC efforts on the Y2K problem in nuclear power plants including the recently begun sample audits of licensee Y2K programs. Jared Wermiel, Chief, HICB, made a presentation on the overall NRC perspective on the Y2K problem and future efforts to address the issue.

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ENCLOSURE B

Office of Nuclear Material Safety and Safeguards  
Items of Interest  
Week Ending October 9, 1998

### **Meeting with the U.S. Enrichment Corporation**

On October 1, 1998, staff from the Division of Fuel Cycle Safety and Safeguards (FCSS) met with the U.S. Enrichment Corporation (USEC) to discuss current regulatory issues affecting enrichment activities. At that meeting, USEC informed FCSS that modifications being installed to increase the seismic resistance of two process buildings at Paducah are now estimated to cost more than twice the original cost estimate (original estimate was \$23 million), and the completion date of June 30, 1999, will likely be substantially exceeded. In June 1997, USEC informed the Nuclear Regulatory Commission (NRC) that it was stopping work on the seismic modifications; NRC subsequently informed USEC that any actions taken by USEC which jeopardize meeting the completion date for the modifications (which at that time was December 31, 1997), such as stopping work, were taken at USEC's own risk. In March 1998, the Commission, in response to a petition from USEC, ordered (CLI-98-02) USEC to install the seismic modifications "without delay as stated in the Director's decision." The Director's decision required the modifications to be completed by June 30, 1999. The Commission was informed about the seismic issue at Paducah in SECY-98-022. USEC has hired another consultant to review the situation and provide recommendations on how to proceed and will inform NRC of its plans regarding installation of the seismic modifications within the next few weeks. Regarding the Atomic Vapor Laser Isotope Separation (AVLIS) facility, USEC informed the staff that it now anticipates submitting the AVLIS license application in late CY 1999, instead of April 1999, as currently scheduled.

### **New Project for BWX Technologies**

On October 1, 1998, Division of Fuel Cycle Safety and Safeguards and Region II staff participated in a video conference with BWX Technologies (BWXT) to discuss BWXT's new project to downblend 50 metric tons of high-enriched uranium metal and oxides from the U.S. Enrichment Corporation and Oak Ridge National Laboratory. BWXT described the contract and scheduling plans; receipt, sampling, and storage facilities; processing facilities; and safety analysis and licensing plans. This project will be similar to the Project Sapphire downblending project, but much larger. This project is expected to take approximately 6 years. License changes are projected to be necessary, and initial applications will be submitted in early December 1998.

### **Meetings to Discuss "Orphan Sources"**

Members of the Division of Industrial and Medical Nuclear Safety met with representatives of the Department of Energy and the Environmental Protection Agency on September 24 and October 1, 1998, respectively, to discuss issues concerning "Orphan Sources." The primary purpose of these meetings was an attempt to define the regulatory responsibilities and jurisdictions of each of the agencies in addressing the orphan source problem, and to discuss their experiences in this area. Areas of particular importance included preventing orphan sources, responding to the notification of a lost source, and response to a "found" source where the responsible party may or may not be identified. In addition, current initiatives in place and future planned initiatives for dealing with the orphan source issue by each agency were discussed. These include recycling programs, the use of contractor(s) for responding to orphan sources (for both recovery and disposal), prevention of stray radioactive materials from entering the U.S., and release of contaminated materials and activated metals below certain concentrations or limits. Additional meetings and dialogue are planned to further discuss and address these issues.

### **Draft Regulations for Disposal of High-Level Radioactive Wastes at a Proposed Geologic Repository at Yucca Mountain, Nevada--Draft Proposed 10 CFR Part 63**

Staff recommendations for regulations for disposal of high-level radioactive wastes at a proposed geologic repository at Yucca Mountain, Nevada, submitted to the Commission in SECY-98-225, September 28, 1998, were placed on the Nuclear Regulatory Commission's technical conference website. This action, in response to a Staff Requirements Memorandum dated September 22, 1998, will enable all stakeholders to have preliminary access to the staff recommendations. The Commission Paper and the draft Federal Register notice can be accessed at: <http://techconf.llnl.gov/cgi-bin/topics>.

### **Holtec International HI-STAR 100 Storage Cask System**

Staff from the Spent Fuel Project Office have completed the preliminary safety review of the Holtec International (Holtec) HI-STAR 100 spent fuel storage cask system (HI-STAR 100). A preliminary Safety Evaluation Report and Certificate of Compliance are now being prepared for the rulemaking process required to add new storage casks to the list of casks in 10 CFR Part 72, Subpart K, which those utilities holding general licenses for on-site dry cask storage can use. Holtec has two related applications before the Nuclear Regulatory Commission. One is the application for certification of the HI-STAR 100 for transportation of spent fuel, pursuant to 10 CFR Part 71. The other is an application for certification for storage, pursuant to 10 CFR Part 72 of the HI-STORM concrete overpack that is compatible with the HI-STAR 100. Private Fuel Storage, LLC, plans to use the HI-STAR/HI-STORM combination for transportation to, and storage at, its proposed Private Fuel Storage Facility on the Reservation of the Skull Valley Band of Goshute Indians.

### **Electric Power Research Institute Meeting on Handling Accident Deceleration Analysis for Dry Storage Cask**

On October 5, 1998, staff from the Spent Fuel Project Office met with representatives from the Electric Power Research Institute (EPRI), nuclear power licensees and spent fuel cask vendors to obtain industry response to Nuclear Regulatory Commission (NRC) sponsored work at Lawrence Livermore National Lab (LLNL) on handling accident deceleration analysis for dry storage casks. The meeting was held at EPRI's offices in Palo Alto, California. For each dry cask storage system design, the Certificate of Compliance holder must provide, as part of their Safety Analysis Report, an evaluation of impact decelerations of a cask subject to drop and tipover accidents. NRC has contracted with LLNL to develop a report tabulating or plotting cask response to drop and tipover accidents for cask system design parameters varied over specific ranges. The meeting discussed the LLNL approach to developing the parameters and obtained industry's feedback on the proposed work.

The meeting was productive and concluded with EPRI recommending that the industry, through the Nuclear Energy Institute, provide NRC with a topical report on this subject. The topical report would describe a methodology for cask vendors to select bounding decelerations for accident handling analysis for a dry storage cask. The topical report will be the basis for a follow-up meeting on specific topics and other actions to clarify staff and licensee positions. EPRI is to compare the code used to develop their simplified methodology with experimental data from LLNL, Sandia and British Nuclear Fuels,

Limited. NRC will compare its audit model with as much of the data as practicable.

### **Meeting with National Institute of Standards and Technology Staff Concerning Accreditation of Calibration Laboratories**

Spent Fuel Project Office joined the Office of Nuclear Reactor Regulation in meeting with the staff of the National Voluntary Laboratory Accreditation Program (NVLAP) at the National Institute of Standards and Technology (NIST). The purpose of the meeting was to discuss the NVLAP program elements regarding whether the quality programs of NVLAP-accredited laboratories provide sufficient controls to prevent the need for licensees to audit the quality programs of these laboratories. NIST staff reported that, currently, NVLAP laboratory accreditation is accepted in lieu of licensee inspection of labs processing dosimetry required by [10 CFR Part 20](#).

### **Staff Participates in Low-Level Waste Forum Meeting**

On September 28-30, 1998, staff from the Division of Waste Management participated in a meeting of the Low-Level Radioactive Waste (LLW) Forum in Annapolis, Maryland. The LLW Forum meets three times a year to share information with one another on LLW management and disposal issues, and to exchange views with officials of federal agencies and other interested groups. Staff gave a presentation on effects of nuclear power plant life span decisions on LLW disposal projections, and participated in a panel on the continuing rights and obligations of states and compacts under the Low-Level Radioactive Waste Policy Amendments Act (LLRWPA) of 1985. Other sessions at the meeting included presentations on assured isolation; renewal of the Envirocare license for LLW disposal, which could allow Envirocare to accept most Class A waste generated in the U.S.; and the scope and background of a new General Accounting Office study on alternatives to the LLRWPA that was recently requested by Congress.

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ENCLOSURE C

Office of Nuclear Regulatory Research  
Items of Interest  
Week Ending October 9, 1998

### **Collection of Soil and Water Samples from Kaiser Aluminum Site Decommissioning Management Plan (SDMP) Site**

On September 14 and 15, 1998, a Division of Regulatory Applications staff member, and Pacific Northwest National Laboratory (PNNL) scientists collected soil and water samples at the Kaiser Aluminum Special Products SDMP site in Tulsa, Oklahoma. The samples were collected as part of a research project to examine solubilities of radionuclides for SDMP site and LLW facility performance assessments. Representatives of Kaiser Aluminum participated in the site visit and sample collection. The Kaiser Aluminum site occupies approximately 10 acres and comprises a freshwater pond, a five acre retention pond, and an old filled and grass covered retention pond. From 1958 to 1970, the site smelted magnesium and thorium metal alloys. The magnesium was recovered and the remaining solids containing radioactive material were broken up, crushed and disposed (sluiced) to the five acre retention pond. Samples of soil/sludge, water, and vegetation were collected from several locations at the site and shipped to PNNL for analysis and study. The data and information collected from these samples will be used to understand radionuclide solubility, release, and behavior at SDMP sites. Radionuclide solubility data will be used in source term computer codes to assess the performance of SDMP sites and the leach rate data will be used to establish guidance on methods to determine compliance with dose standards for SDMP sites. The measurements of radionuclides in surface and ground waters will allow a comparison to be made between the concentrations calculated with a computer codes and the actual values as measured from the field data.

### **International Steam Generator Tube Integrity Program (ISG-TIP-2) NDE Task Group Meeting Summary**

A Division of Engineering Technology staff member chaired a 5-day meeting (September 28-October 2, 1998) of the Non-Destructive Examination Task Group meeting at Argonne National Laboratory involving participating members from EPRI, Framatome Technologies Inc., Westinghouse, ABB-CE, Northern States Power, Duke Power, and Commonwealth Edison. The objective of this meeting was to develop a plan for round-robin inspection of the SG tube mock-up fabricated by ANL for this research program. Careful attention is being given to treat the mock-up as an actual operating steam generator and develop an inspection plan consistent with industry practices and guidelines.

The results from the round-robin should benefit both the industry and regulator by providing benchmarks for eddy current test capabilities for the wide variety of degradation types that currently exist in the field.

### **LPSD COOPRA Meeting**

The low power shutdown (LPSD) working group of the International Cooperative PRA Research (COOPRA) committee met last week in Munich, Germany. Two staff members of the Division of Systems Technology participated. Member countries include Czech Republic, France, Germany, Italy, Japan, Korea, South Africa, Spain, Taiwan, Great Britain, Croatia, Finland, Hungary and the United States. The working group decided to pursue three topics associated with LPSD including (1) identification and quantification of initiating events, (2) screening of operational modes, and (3) supporting analyses, physics, thermal hydraulics. The working group plans on issuing technical reports on this first set of topics in the next year.

### **Publication of Research Results on Critical Heat Flux (CHF) Phenomenon on a Downward Facing Curved Surface: Effect of Thermal Insulation**

In the unlikely event of a severe accident, a significant amount of molten core material can relocate into the lower head of the reactor pressure vessel (RPV), potentially resulting in the failure of the reactor vessel. However, if the reactor vessel integrity can be maintained, the potential for large releases

due to ex-vessel severe accident phenomena can be substantially reduced. In the AP600, provisions were included in the design to prevent RPV failure by providing ex-vessel cooling of the RPV by flooding the reactor cavity.

NUREG/CR-5534, "Critical Heat Flux (CHF) Phenomenon on a Downward Facing Curved Surface: Effect of Thermal Insulation" was recently issued, which documents the results of the final series of experiments performed at Penn State on ex-vessel boiling heat transfer in the presence of thermal insulation. The ex-vessel boiling heat transfer experiments were carried out on an approximate 1/13 scale hemispherical vessel surrounded by a simulated thermal insulation similar to that proposed in the AP600 design. (Previous experiments in this program investigated the ex-vessel boiling heat transfer (CHF) on an uninsulated vessel.) The results of the recent experiments indicated an enhanced heat transfer (higher CHF) from the vessel outer surface than the earlier experiments where the insulation was not present. The results of this program have provided confirmatory data which supported the design review of the AP600.

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ENCLOSURE F

Office of Administration  
Items of Interest  
Week Ending October 9, 1998

#### Reporting Requirements for Nuclear Power Reactors; Public Meeting (Part 50)

A document announcing a public meeting to discuss rulemaking to modify power reactor reporting requirements was published in the Federal Register on October 2, 1998 (63 FR 52990). The purpose of the meeting is to test key aspects of the contemplated rulemaking for clarity and consistency. The meeting will be held November 13, 1998.

#### X-Ray Training

DFS arranged for mail/package screening (x-ray) re-certification training which was conducted on October 1, 1998, for the mail room staff. Re-certification requires demonstrated competence through written examination and practical exercises in x-ray equipment operation and interpretation. The technicians must be familiar with mechanical and radiation safety precautions and with NRC incident management, suspicious package handling, and related evacuation procedures.

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ENCLOSURE G

Chief Information Officer  
Items of Interest  
Week Ending October 9, 1998

#### Freedom of Information and Privacy Act Requests received during the 5-Day Period of October 2, 1998 - October 8, 1998:

Unisys Bldg, Sheppard Rd, St. Paul, MN, radiation exposure.	(FOIA/PA-99-003)
Contracts, verbatim stenographic court reporting services.	(FOIA/PA-99-004)
Contracts, NRC-10-94-141, operations and maintenance.	(FOIA/PA-99-005)
Jones & Laughlin Steel Corp/LTV Steel in Pennsylvania, seal source device licenses.	(FOIA/PA-99-006)
MLTS database on diskette.	(FOIA/PA-99-007)
Contracts, custodial operations services, solicitation RS-ADM-99142.	(FOIA/PA-99-008)
Named individual, 1985 investigation, case no. 1-85-019, Peach Bottom.	(FOIA/PA-99-009)

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ENCLOSURE I

Office of Human Resources  
Items of Interest  
Week Ending October 9, 1998

#### Departures

SELBURG, Kara	RADIATION SPECIALIST	RIII
STINSON, Narvaez	RADIATION SPECIALIST	RII

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ENCLOSURE M

Office of Public Affairs  
Items of Interest  
Week Ending October 9, 1998

**Media Interest**

Five reporters attended Hub Miller's press conference at Millstone.

Region I received calls regarding the Maine Yankee Notice of Violation.

**Press Releases**

**Headquarters:**

- 98-179 NRC to Hold Public Meeting on Reporting Requirements for Nuclear Power Plants
- 98-180 NRC Makes Available Staff Recommendations for Proposed Rule on Yucca Mountain Site
- 98-181 NRC Issues Notice of Violation But No Civil Penalty to Maine Yankee Atomic Power Company
- 98-182 Note to Editors: ACRS Meeting November 4
- 98-183 NRC Approves Decommissioning Plan for B&W Facility in Parks Township, Pennsylvania

**Regions:**

- I-98-114 Note to Editors: NRC Postpones Meetings on Indian Point 1 Decommissioning
- II-98-58 NRC Seeks Input for Environmental Statement on Proposed Oconee Nuclear Power Plant License Renewal
- IV-98-38 Scott Schwind Assigned Resident Inspector for NRC at Comanche Peak

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ENCLOSURE N

Office of International Programs  
Items of Interest  
Week Ending October 9, 1998

**Czech Republic Visitors**

On October 9, the Chairman met with the Deputy Chairman of the State Office for Nuclear Safety (SUJB) of the Czech Republic and with the SUJB Temelin Licensing Manager. The Czechs wanted to discuss the possibility of NRC supporting SUJB in the 1999-2001 time frame when they expect several European countries to challenge the operational safety of the Temelin NPP. The Chairman stated that NRC's commitment to Temelin dates back to the early 1990s and, within certain parameters that have yet to be agreed upon, the NRC will continue to offer support to SUJB during the commissioning phases of Temelin. In a subsequent meeting, the SUJB visitors met with William Travers, OEDO, and senior NRR management to discuss the matter in greater detail. Dr. Travers was very supportive and suggested that the Czechs list key safety issues where they expect the most problems. NRR offered to conduct a comparison/assessment of how SUJB handled certain safety cases to see if they are using a philosophy similar to NRC's.

**Nuclear Safety Account, Chernobyl Sarcophagus Fund and G-7 Nuclear Safety Working Group Meetings**

From October 5 to 9, John Ramsey, OIP, participated in Nuclear Safety Account, Chernobyl Sarcophagus fund and G-7 Nuclear Safety Working Group meetings in London, England. The purpose of these meetings was to discuss the progress made in implementing Nuclear Safety Account-funded safety improvements at Soviet-designed reactors operating in Bulgaria, Lithuania, Russian and Ukraine, to discuss the progress made in strengthening the Chernobyl Sarcophagus and to discuss the progress made under the Memorandum of Understanding between the G-7 and Ukraine to close Chernobyl by the year 2000, respectively.

**IAEA Advisory Group Meeting to Review Progress and Future Activities of the Extrabudgetary Program on the Safety of Nuclear Installations in South East Asia, Pacific and Far East Countries; Vienna, October 5-6, 1998**

The IAEA held its first Advisory Group Meeting of the Extrabudgetary Program on the Safety of Nuclear Installations in Asia on October 5-6, 1998, in Vienna, Austria. The meeting was chaired by Dr. Terry Lash, Department of Energy, and attended by 11 participating countries and 3 observer countries. Kevin Burke, OIP, attended the meeting for the NRC and was accompanied by Robert Senseney, Department of State, and James Milkey, DOE. The Advisory Group reviewed documents, including Country Nuclear Safety Profiles, and endorsed the strategy to prioritize its safety activities to first assist programs with a nuclear power reactor and research reactor under construction and to strengthen their regulatory bodies. Because of the dynamic nature of China's nuclear power program and the recent significant changes in its government organization, the China Profile was rejected and will be updated. The Group will meet in one year to review the status of the Extrabudgetary Program for Asia and instructed the Secretariat to provide an interim report in six months.

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ENCLOSURE O

Office of the Secretary

Items of Interest  
Week Ending October 9, 1998

Documents Released to Public	Date	Subject
<b>Decision Documents</b>		
1. <a href="#">SECY-98-225</a>	9/28/98	Proposed Rule: 10 CFR Part 63-"Disposal of High-Level Radioactive Wastes in a Proposed Geologic Repository at Yucca Mountain, Nevada
<b>Negative Consent Documents</b>		
1. <a href="#">Comm. McGaffigan's comments on SECY-98-226</a>	10/5/98	Issuance of a Section 274f, Atomic Energy Act Order to Exempt Envirocare of Utah, Inc. From the Licensing Requirements for Special Nuclear Material in Diffuse Waste That Will be Regulated by the State of Utah
<b>Information Papers</b>		
- <a href="#">SECY-98-220</a>	9/24/98	Weekly Information Report - Week Ending September 18, 1998
- <a href="#">SECY-98-222</a>	9/24/98	Status Report of the U.S. Nuclear Regulatory Commission Task Force on Oversight of the U.S. Department of Energy, In Response to COMSECY-96-053-DSI 2 (Report No. 5)
- <a href="#">SECY-98-219</a>	9/21/98	Completion of Staff's Review and Issuance of the Final Safety Evaluation Report on Reduction in Augmented Examination Requirements for Boiling Water Reactor Pressure Vessels Pursuant to 10 CFR 50.55a(g)(6)(ii)(A)

**Commission Correspondence**

1. Letter to Ms. Annetta Cheek, National partnership for Reinventing Government, dated September 28, 1998 provides the Plain language Action Plan for the NRC (Incoming memorandum dated July 28, 1998 also released)
2. Letter to Secretary of Energy Bill Richardson dated October 1, 1998 concerns the privatization review for the United States Enrichment Corporation

**Federal Register Notices Issued**

1. [10 CFR Part 50](#); Preliminary Criterion on the Use of Non-Owner Operating Companies
2. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Planning and Procedures; Notice of Meeting on November 4, 1998
3. Advisory Committee on Reactor Safeguards; Meeting Notice for November 4-7, 1998
4. Shaun P. O'Hern; Docket No. 55-32442-SP; Designation of Presiding Officer (Peter B. Bloch)
5. International Uranium (USA) Corporation; Docket No. 40-8681-MLA-4; Memorandum and Order (Notice of Opportunity for Hearing)
6. Advisory Committee on Reactor Safeguards; Joint Meeting of the ACRS Subcommittees on Reliability and Probabilistic Risk Assessment and on Regulatory Policies; Notice of Meeting on October 29 and 30, 1998
7. State of Arkansas Relinquishment of Sealed Source and Device Evaluation and Approval Authority and Reassumption by the Commission

ENCLOSURE P

Region I  
Items of Interest  
Week Ending October 9, 1998

**Millstone Site**

On Monday and Tuesday, October 5 and 6, 1998, senior NRC Regional Management visited the Millstone Nuclear Power Station. The Regional Administrator held a press conference at the Waterford Town Hall on October 5. Senior management toured the facility, conducted interviews with selected licensee personnel and management, attended an NRC Independent Corrective Action Verification Program (ICAVP) exit meeting, and conducted an evening meeting with members of the public.

**Licensing Practices and Procedures Course**

On September 28 and 29, 1998, two Senior Health Physicists from Region I presented training at the "Licensing Practices and Procedures" course in Pittsburgh, PA. The course was sponsored by the NRC Technical Training Center for NRC, Agreement State, and other persons who review and issue materials licenses. Betsy Ullrich presented "Overview of Licensing" and Pam Henderson presented "Medical Licensing Issues."

#### **Shieldalloy Enforcement Conference**

On October 1, 1998, a predecisional enforcement conference was held with Shieldalloy Metallurgical Corporation to discuss apparent violations identified during an inspection at the licensee's Newfield, NJ facility in July, 1998. Issues discussed included the transfer of ownership of the license without approval by the NRC; inadequate surveys performed; and performing activities that are not authorized by the license.

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ENCLOSURE P

Region II  
Items of Interest  
Week Ending October 9, 1998

#### **Florida Power and Light Company - St. Lucie**

On October 7, representatives of Florida Power and Light Company met in the Regional Office to discuss operational status of the St. Lucie Nuclear facility. Areas discussed included recent improvements made in the working relationship between supervisors and operators in the Operations Department as well as training and development of the Operations Staff.

#### **Southeastern Nuclear Plant Licensing Managers Meeting**

On October 7, the Regional Administrator addressed the Southeastern Nuclear Plant Licensing Managers at a meeting held in Atlanta. Issues discussed included the changes pending in the NRC's inspection, enforcement, and assessment program.

#### **Institute of Nuclear Power Operations (INPO)**

On October 8, 1998, the Regional Administrator was the luncheon speaker at INPO's Seminar for New Maintenance Managers and discussed recent initiatives in the NRC.