December 12, 1997

For: The Commissioners

From: James L. Blaha,
Assistant for Operations, Office of the EDO /s/

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING DECEMBER 5, 1997

| **Contents** | **Enclosure** |
|---|---|
| Nuclear Reactor Regulation | A |
| Nuclear Material Safety and Safeguards | B |
| Nuclear Regulatory Research | C* |
| Analysis and Evaluation of Operational Data | D |
| General Counsel | E* |
| Administration | F |
| Chief Information Officer | G |
| Chief Financial Officer | H* |
| Human Resources | I |
| Small Business & Civil Rights | J* |
| Enforcement | K* |
| State Programs | L* |
| Public Affairs | M |
| International Programs | N* |
| Office of the Secretary | O |
| Region I | P |
| Region II | P |
| Region III | P |
| Region IV | P |
| Executive Director for Operations | Q* |
| Congressional Affairs | R* |

*No input this week

James L. Blaha
Assistant for Operations, OEDO

Contact: M. Leach, OEDO

---

ENCLOSURE A

Office of Nuclear Reactor Regulation
Items of Interest
Week Ending December 5, 1997

Quad Cities - Actions Being Taken to Resolve Post-fire Safe Shutdown Procedural Issues

On November 20, 1997, Ron Gardner, Chief, Engineering Specialist Branch 2, Region III, Patrick Madden, Senior Fire Protection Engineer, Fire Protection Engineering Section, Office of Nuclear Reactor Regulation (NRR) and Robert Pulsifer, Project Manager for Quad Cities, NRR, met with Commonwealth

Edison, the licensee, at Quad Cities. This was a working meeting to discuss the status of post-fire safe shutdown issues and review short term modifications and the re-assessment and rewrite of the post-fire safe shutdown procedures.

The Quad Cities Individual Plant Examination for External Events (IPEEE) identified a core damage frequency (CDF) due to fire events of 5E-03 $yr^{-1}$. The primary factor for this high CDF is attributed to outage induced post-fire safe shutdown equipment un-availabilities (due to the reliance on opposite unit plant equipment to safely shutdown the fire affected unit) and the large number of manual operator actions needed to achieve and maintain post-fire safe shutdown conditions.

On September 26, 1997, the licensee identified deficiencies with the Quad Cities post-fire safe shutdown procedures. Because of these deficiencies and the lack of confidence in their associated circuit analysis (fuse breaker coordination of 480VAC, 250VDC, and 125VDC shutdown loads), the licensee declared all Appendix R post-fire safe shutdown paths inoperable and entered its 67-day administrative limiting condition for operation (LCO) for both units. This LCO expired on December 2, 1997. As a result of this condition the licensee decided to shutdown Unit 2 (cold shutdown).

In response to this condition, the licensee instituted a reassessment program to evaluate the technical bases which support the post-fire shutdown logic and manual actions. This program identified several concerns related to adequacy of these procedures and their ability to aid the operators in achieving and maintaining post-fire safe shutdown conditions. To address these concerns, the licensee developed and implemented a new interim safe shutdown methodology (ISSM) until it can implement its long term post-fire safe shutdown modifications (i.e, unit specific dedicated post-fire safe shutdown system). To justify the continued operation of Unit 1, the licensee committed to have the ISSM operational by December 2, 1997. The intent of the ISSM was to simplify operator actions and reduce the fire-induced spurious operation concerns. To support ISSM implementation, the licensee had to make certain plant modifications (i.e., reroute the 4.1Kv station blackout (SBO) diesel generator power cables, provide a 3-hour fire resistive enclosure for a drywell penetration and associated ADS cables, address spurious start of HPCI and RCIC pumps).

The licensee field verified its ISSM procedures on November 24, 29, and 30. The plant modifications necessary to support ISSM implementation were scheduled to be completed by December 1, 1997. The licensee in the November 20, 1997, meeting stated that the ISSM method would comply with Appendix R. In addition, the licensee committed to perform an independent self assessment of the Quad Cities fire protection program. This self assessment will be based on the NRC's proposed FPFI inspection procedure and will be completed by May 1, 1998.

During a conference call on December 1, 1997, the licensee informed NRR and Region III that it had performed an operability assessment and intended to exit the LCO by declaring its Unit 1 ISSM approach operable but degraded. During this call, the staff and the licensee discussed the degraded conditions and the adequacy of the interim compensatory measures and corrective actions. Specifically, the staff was concerned that the ISSM operability assessment, for alternative shutdown, did not address compliance with Section III.L.1 of Appendix R by demonstrating that the reactor coolant process variables will be maintained within those predicted for a loss of normal AC. For those conditions where the plant needs to be shutdown from outside the control room, the procedure relied on other manual control room actions (other than a reactor scram) to be completed prior to its abandonment. In addition, the staff was concerned with the lack of radiant energy heat shield protection for the temporary SBO diesel generator cables routed adjacent to the MG-sets on the turbine operating floor, the operational time line analysis and entry conditions for the ISSM, and the preventative maintenance schedule for the SBO diesel generators. The staff also expressed concern that, under certain conditions, the ISSM approach places the plant voluntarily in a station blackout (SBO) while busses are being stripped and essential safe shutdown loads are being aligned and loaded on the SBO diesel generator. The licensee indicated that it would consider these concerns in its final determination.

On December 1, 1997, around 21:00 hours (CST), the licensee declared the ISSM operable and exited the 67-day LCO.

On December 2, 1997, a follow-up conference call was held with the licensee. The licensee indicated that they would be installing radiant energy heat shields to protect the SBO cables from an MG-set fire, relying on the shift supervisor to make the determination as to when to implement the ISSM, that they are going to develop an action plan to evaluate the preventative maintenance program for the SBO diesel generators, and that they have initiated a problem identification form (PIF) which has requested engineering to evaluate the ISSM's ability to meet Section III.L.1 of Appendix R. Based on the information presented in the licensee's operability assessment, its additional compensatory measures, and their immediate short term modifications, the staff concludes that the ISSM provides the reasonable assurance needed to justify continued operation of Unit 1 until the planned dedicated shutdown system is incorporated into the plant design.

Update on Callaway Axial Offset Anomaly

The axial offset anomaly at Callaway was the subject of an inspection November 17-21, 1997. The team consisted of two Reactor Systems Branch (SRXB) members, the team leader from Region IV, a chemist from the Materials and Chemical Engineering Branch (EMCB), and a contractor. The purpose was to evaluate the licensee's actions with respect to axial offset anomaly including the licensee's review of vendor analysis, the shutdown margin calculations, the safety evaluation performed to reduce rod worth uncertainty and the impacts on reactor water and crud chemistry. The major findings of the inspection were: (1) The 50.59 evaluation to lower rod worth uncertainty to 3 percent was inadequate because it did not account for the uncertainty in the stuck rod worth. This evaluation has been approved by the plant safety committee, but reduction of the rod worth uncertainty has not been implemented. (2) Understanding of the calculations, particularly on rod swap, was very weak by the engineers responsible for the calculations. (3) The present method of predicting shutdown margin (SDM) appears to be conservative as long as axial offset does not change much between flux maps (offset is tracked daily). (4) Licensee's prediction of SDM might be non-conservative when coming back from a down power or if raising power above 70 percent. The licensee addressed this concern with a position paper stating the factors which need to be reviewed in such a case and important limits to be observed. Plant is currently limited to 70 percent power.

The latest update on SDM calculations, based on the November 13, 1997, flux map, was 1826 pcm as compared with the TS limit of 1300. Loss of shutdown margin has slowed to about 3 pcm per day. At this rate the TS limit will be reached at approximately the scheduled end of cycle, April 1998. If

SDM improves sufficiently, the licensee plans to increase power a few percent. The staff will continue to monitor shutdown margin.

Salem

Operational Status - Salem Unit 2 is operating at 100% power and Salem Unit 1 is shut down. Unit 1 refueling operations began on November 28, 1997, and currently about 2/3 of the core has been reloaded. The reload was slowed due to several problems with the manipulator mast and a containment lineup configuration control problem with a steam generator sample valve test connection. The licensee's target date for restart is February 1998.

Salem Assessment Panel - The next Salem Assessment Panel meeting is scheduled for December 16, 1997. A public meeting with the licensee is scheduled for December 4, 1997, at the plant to discuss the lessons learned during the Unit 2 restart and operation to date and to discuss the status of the Unit 1 restart preparations.

Crystal River 3 Restart Progress -- Licensing Submittals

Florida Power Corporation continues to maintain its target for completing the corrective actions and restarting the facility by December 1997. Early in 1997, FPC identified 37 potential licensing submittals that would require NRC approval prior to restart in December 1997. Additionally, the staff during its recent Safety System Functional Inspection (SSFI) identified several unresolved issues and the region submitted four Task Interface Agreements (TIAs) for NRR determination whether the issues must be resolved prior to restart. The current status of these issues is as follows:

STATUS OF LICENSING SUBMITTALS THAT ARE REQUIRED FOR RESTART

| | |
|---|---|
| Total Identified (including the four TIAs) | 41 |
| Awaiting initial submittal | 4 |
| Supplemental Information awaited | 3 |
| Received and review in progress | 14 |
| Completed / Closed | 20 |

On November 6, 1997, in a telephone conference with the staff, FPC stated that it intends to make operability determinations and complete justifications for continued operation (JCOs), consistent with GL 91-18, Revision 1, for those licensing actions that will be deferred until after restart. The NRC staff informed the licensee that the JCOs will be subject to audit. These will be reviewed prior to plant restart.

San Onofre Nuclear Generating Station, Units 2 and 3

San Diego Gas & Electric Company, a 20% co-owner of the San Onofre units, has announced that it is planning to sell its interest in the San Onofre units. The staff would review the qualifications of any new owners in accordance with 10 CFR 50.80.

Southern California Edison (SCE), the operator and majority owner of the San Onofre units, has announced the sale of 10 gas-fired generating plants with a combined generating capacity of 7,532 megawatts for a total price of $1.115 billion. The plants were sold through an auction process at 2.65 times their book value. The transactions are subject to regulatory approval (non-NRC) and are scheduled to close in time for the January 1, 1998 opening of the new Power Exchange and Independent System Operator, as part of California's restructuring electricity market. As a side note, a number of SCE's large customers have announced that they are planning to buy power from other utilities when the deregulation of electric power in California becomes effective.

Clinton Power Station -- SET Public Exit Meeting

On Thursday, December 11, 1997, at 10:00 a.m., the NRC Special Evaluation Team will conduct a public exit meeting with the Clinton licensee (Illinois Power Company).

A briefing of the EDO by the SET team leaders will be conducted on Tuesday, December 9, 1997, at 1:00 p.m.

---

ENCLOSURE B

Office of Nuclear Material Safety and Safeguards
Items of Interest
Week Ending December 5, 1997

Integrated Materials Performance Evaluation Program Training

On December 2-3, 1997, the Office of State Programs (OSP) and the Office of Nuclear Material Safety and Safeguards (NMSS), conducted training for the Integrated Materials Performance Evaluation Program (IMPEP) team leaders and team members. The first day of training, attended by 35 Nuclear Regulatory Commission staff members from OSP, NMSS and all four Regions, and by personnel from 12 Agreement States, covered the IMPEP process and materials performance indicators. The second day of training focused on IMPEP team leadership and was led by a trainer sponsored by the NRC Office of Human Resources. Comments from participants indicated the training was well received and helpful in preparation for the FY 1998 IMPEP reviews.

International Atomic Energy Agency Technical Committee Meeting

On November 24-28, 1997, Office of Nuclear Material Safety and Safeguards staff members participated in a technical committee meeting at the International Atomic Energy Agency (IAEA) in Vienna, Austria, to prepare a technical report entitled, "Methodology for Investigation of Accidents with Radiation Sources." The report provides a sample methodology for planning, initiating, conducting, and documenting investigations of accidents involving radiation sources. It is intended for use by licensees, registrants, regulatory authorities, and qualified experts participating at the international or national level in investigation of accidents involving exposure to radiation or radioactive substances.

Chem-Nuclear Systems Proposes South Carolina Trust Fund

Chem-Nuclear Systems, Limited Liability Corporation (CNS), operator of the Barnwell, South Carolina, low-level waste disposal facility, recently proposed to industry an agreement to establish a South Carolina Trust Fund. Facility customers would be asked to make a prepayment to the State in exchange for availability of 25 years of future disposal capacity at a predictable cost. An up-front tax payment of $200 per cubic foot of disposal capacity reserved (disposal allotment charge) would be deposited in trust to establish a fund of not less than $1 billion. A non-refundable commitment fee of up to $3.60 per cubic foot of reserved disposal capacity would also be payable upon receipt of the individual company commitments. Estimated fixed costs of operating the facility over the 25 years would be shared by those reserving capacity on a pro-rata basis and would be paid annually to CNS. This fee would be contractually set for the entire term of the agreement. Similarly, CNS would set a scheduled disposal fee for any variable costs. In the event of uncontrollable circumstances which deprived customers of the use of the facility, any portion of the disposal allotment charge relating to disposal capacity reserved but not used and interest thereon would be returned to the customer.

The funding agreements would not be effective unless legislation is enacted consistent with the proposal. To provide the State with assurance that the Trust will be funded, each user would be asked to sign a commitment letter by January 16, 1998. CNS will not proceed with the proposal if it does not receive commitments for at least five million cubic feet of disposal capacity. If this level of commitment is achieved, payment of the commitment fee ($3.60/ft$^3$) would be required on January 30, 1998. If the corresponding legislation is enacted, payment of the disposal allotment charge ($200/ft$^3$) would be required on October 30, 1998. Customers that do not enter into funding agreements will be required to make tax payments to the State in an amount determined by the State as the waste is delivered. The proposal notes that CNS cannot predict whether such taxes will be increased above the level currently in effect. CNS is taking this action because of reduced revenues to the State and the eroding economies of continued operation of the Barnwell facility. According to CNS, the proposal is intended to maintain the Barnwell disposal option while providing the State with economic benefits from its operation.

Meeting With Southern Nuclear

On December 2, 1997, Spent Fuel Project Office staff members met with representatives of Southern Nuclear (SN) to discuss SN's plans for dry cask storage at its Hatch Nuclear Plant near Baxley, Georgia. Representatives of Holtec International (Holtec), British Nuclear Fuels, Ltd., Westinghouse, Ibex, and Bechtel also attended the meeting. Holtec's Hi-Storm dual purpose cask system has been chosen by SN for use under a general license at the proposed Hatch independent spent fuel storage installation (ISFSI). At the meeting, SN also indicated that it has organized its spent fuel program and has plans in place for the development of ISFSIs at each of its nuclear plants. SN is also a member of the Private Fuel Storage L.L.C consortium which has applied for a license to build an ISFSI on the Reservation of the Skull Valley Band of Goshute Indians.

---

ENCLOSURE D

Office for Analysis and Evaluation of Operational Data
Items of Interest
Week Ending December 5, 1997

Maintenance Rule Training

As part of the process for transferring the conduct of Maintenance Rule training from the program office to the Technical Training Division, a member of the TTD staff participated in a Maintenance Rule training class held December 2-4, 1997 in Headquarters. The new Maintenance Rule course is being developed from the initial training developed by NRR. TTD and NRR are working together to develop formal standard course materials for the training. The course will use a workshop/case study approach. The objective is to provide inspectors with an understanding of how to review and evaluate implementation of a performance-based regulation like the Maintenance Rule, and how to use the final Maintenance Rule Inspection Procedure. Related to this project, a Maintenance Rule overview section has been developed and incorporated into the Inspecting for Performance course and development of a new Maintenance Rule Refresher course is planned.

PRELIMINARY NOTIFICATIONS

1. PNO-I-97-073, Connecticut Yankee Atomic Power Co. (Haddam Neck 1), STATUS OF OFFSITE SURVEYS FOR CONTAMINATED BLOCKS

2. PNO-III-97-094, Wisconsin Electric Power Co. (Point Beach 1), UNIT 1 RESTART

3. PNO-III-97-095, S. C. Johnson & Son, Inc., NOTIFICATION OF DAMAGED NUCLEAR GAUGE

---

ENCLOSURE F

Office of Administration

Items of Interest
Week Ending December 5, 1997

OWFN Rooftop Cooling Tower Overhaul

The main cooling towers on the OWFN rooftop are undergoing a complete overhaul to ensure energy efficient operation and to reduce/prevent leakage that can create roof damage from standing water. The work is expected to be completed by the end of December 1997.

Acquisition Training

On December 3, 1997, the Division of Contracts and Property Management (DCPM) piloted its Acquisition for Project Managers module, "Closing Out the Contract", for thirteen project managers (PM). This module focused on the NRC policies and procedures for deobligation of excess funds, closing contracts, DOE Laboratory Agreements, and evaluating the final performance of the contractor. Each participant received a course manual that will serve as a reference tool for PMs in performing their day-to-day responsibilities.

Exempt Distribution of a Radioactive Drug Containing One Microcurie of Carbon-14 Urea (Parts 30 and 32)

A final rule that permits NRC licensees to distribute a radioactive drug containing one microcurie of carbon-14 urea to any person for "in vivo" diagnostic use was published in the Federal Register on December 2, 1997 (62 FR 63634). This final rule, which grants PRM-35-12, makes the drug more widely available and reduces costs to patients, insurers, and the health care industry. The final rule becomes effective January 2, 1998.

Changes to Nuclear Power Plant Security Requirements (Part 73)

A final rule that amends the regulations applicable to nuclear power plant security by removing certain requirements associated with an internal threat was published in the Federal Register on December 2, 1997 (62 FR 63640). The action resulted from an NRC reconsideration of its nuclear power plant physical security requirements to identify those that are marginal to safety, redundant, or no longer effective. The final rule becomes effective January 16, 1998.

Criticality Accident Requirements (Parts 50 and 70)

A direct final rule that amends the regulations to provide light-water nuclear power reactor licensees with greater flexibility in meeting the requirement that licensees authorized to possess more than a small amount of special nuclear material maintain a criticality monitoring system in each area where the material is handled, used, or stored was published in the Federal Register on December 3, 1997 (62 FR 63825). The action is taken as a result of experience gained in processing and evaluating a number of exemption requests from power reactor licensees and NRC's safety assessments in response to these requests. The direct final rule becomes effective February 17, 1998.

The companion proposed rule to this direct final rule was published in the Federal Register on December 3, 1997 (62 FR 63911). The comment period closes January 2, 1998.

Industry Codes and Standards ( Part 50)

A proposed rule that would amend NRC's regulations that incorporate by reference a number of industry codes and standards applicable to nuclear power plant licensees was published in the Federal Register on December 3, 1997 (62 FR 63892). The proposed rule would revise the requirements applicable to construction, inservice inspection, and inservice testing of nuclear power plant components. The comment period on this proposed rule closes March 3, 1998.

---

ENCLOSURE G

Chief Information Officer
Items of Interest
Week Ending December 5, 1997

Freedom of Information and Privacy Act Requests Received during the 5-Day Period of November 28 - December 4, 1997:

| | |
|---|---|
| Contract, janitorial services. | (FOIA/PA-97-463) |
| Contract, SOL RS-ADM-98-141, building security services. | (FOIA/PA-97-464) |
| IMPAC listing. | (FOIA/PA-97-465) |
| IMPAC listing. | (FOIA/PA-97-466) |
| Personnel and security records on named individuals. | (FOIA/PA-97-467) |
| Vacancy, Region IV Radiation Specialist/Health Physicist. | (FOIA/PA-97-468) |
| Millstone assessment panel (MAP). | (FOIA/PA-97-469) |

| | |
|---|---|
| Contract, NRC-33-95-173, Ruland Assoc. | (FOIA/PA-97-470) |

**Requests Completed:**

97-313
97-387
97-396
97-407 (No PDR)
97-410 (No PDR)
97-423 (No PDR)
97-431 (No PDR)
97-439
97-444
97-446 (No PDR)
97-448 (No PDR)
97-450 (No PDR)
97-454 (No PDR)
97-462 (No PDR)
97-463 (No PDR)

---

ENCLOSURE I

Office of Human Resources
Items of Interest
Week Ending December 5, 1997

| | | |
|---|---|---|
| **Arrivals** | | |
| BURROWS, Ronald | PROJECT MANAGER (PFT) | NRR* |
| CHANG, Lydia | CHEMICAL ENGINEER (PFT) | NMSS |
| ENGLAND, Jennifer | REACTOR ENGINEER (PFT) | RI |
| HIPSCHMAN, Thomas | REACTOR ENGINEER (PFT) | RI |
| STEWART, Belinda | SECRETARY (OA) (PFT) | RIV |
| **Retirements** | | |
| WIEDER, Arnold | RESOURCE ANALYST (PFT) | RIV |
| **Departures** | | |
| PLETTNER, Elden | REACTOR ENGINEER (PFT) | RIII |
| VACCARO, Michael | PROPERTY MGMT ASST (PFT) | ADM |

*Transferred from Region III.

---

ENCLOSURE M

Office of Public Affairs
Items of Interest
Week Ending December 5, 1997

Media Interest

Peter Crane, OGC, will be interviewed live on WCCO radio, Minneapolis, MN, on the potassium iodide issue.

The Oregonian is printing a lengthy article on the economic situation at WNP-2, including the NRC oversight panel, capacity factors, and the plant's enforcement history.

**Press Releases**

**Headquarters:**

97-178 NRC Considers Changes to Regulations on Inadvertent Criticality Requirements for Nuclear Power Plants

| | |
|---|---|
| 97-179 | NRC Issues Safety Evaluation Report on New Mexico Uranium Mining Project |

Regions:

| | |
|---|---|
| I-97-152 | NRC to Meet With PSE&G to Discuss Apparent Harassment of Security Officer at Artificial Island |
| I-97-153 | Note to Editors: Commission Briefing and Public Meeting on Millstone Scheduled |
| I-97-154 | NRC, Utility to Discuss Apparent Violations at Beaver Valley |
| II-97-84 | NRC and FPC to Meet December 12 to Discuss Restart Status of Crystal River Nuclear Power Plant in Florida |
| III-97-104 | NRC's Special Evaluation Team to Discuss Preliminary Results of Clinton Nuclear Power Station Performance Review |
| III-97-105 | NRC Names New Senior Resident at Kewaunee Plant |
| IV-97-73 | NRC Staff Proposes $110,000 Fine Against NPPD for Violations at Cooper Nuclear Plant |
| IV-97-74 | NRC, STP to Discuss Apparent Violations at South Texas Nuclear Plant |

---

ENCLOSURE O

Office of the Secretary
Items of Interest
Week Ending December 5, 1997

| Decision Documents Released to the Public | | |
|---|---|---|
| Document | Date | Subject |
| 1. SECY-97-183 | 8/7/97 | Criteria for Funding Agreement State Training, Travel and Technical Assistance |
| - SRM on 97-183 | 11/19/97 | (same) |
| - Commission Voting Record on 97-83 | 11/17/97 | (same) |
| 2. SECY-97-227 | 10/3/97 | Status of Staff Actions on Standard Review Plans for Antitrust Reviews and Financial Qualifications and Decommissioning-Funding-Assurance Reviews |
| - SRM on 97-227 | 11/18/97 | (same) |
| - Commission Voting Record on 97-227 | 11/18/97 | (same) |
| 3. SECY-97-237 | 10/16/97 | Memorandum of Understanding with the Department of Energy |
| - SRM on 97-237 | 11/13/97 | (same) |
| - Commission Voting Record on 97-237 | 11/13/97 | (same) |
| **Information Papers Release to the Public** | | |
| 1. SECY-97-269 | 11/19/97 | Weekly Information Report - Week Ending November 14, 1997 |
| 2. SECY-97-271 | 11/20/97 | Update on Status of Safety Evaluation Report on Proposed Reduction in Augmented Examination Requirements for Boiling Water Reactor Pressure Vessels Pursuant to 10 CFR 50.55a(g)(6)(ii)(A) |
| **Memoranda Release to the Public** | | |

| | | |
|---|---|---|
| 1. M971030A&B | 12/2/97 | Staff Requirements: All Employees Meeting, 10:30 A.M. and 1:35 P.M. Thursday, October 30, 1997 |

Commission Correspondence Released to the Public

1. Letter to Representative Bart Stupak dtd 11/26/97 responds to constituent concerns about the storage of spent nuclear fuel in VSC-24 dry storage casks
2. Letter to Dr. Fabio Morales, National Atomic Energy Commission (Nicaragua), dtd 11/26/97, responds to request for regulations, information on training programs, and disposition of a cobalt teletherapy machine

Federal Register Notices Issued

1. Notice of receipt of petition for rulemaking; Chromalloy Tallahassee

---

ENCLOSURE P

Region I
Items of Interest
Week Ending December 5, 1997

Seabrook Site Visit

C. Hehl and C. Cowgill from Region I DRP, A. Blough from DNMS, and J. Zwolinski, C. Smith and A. DeAgazio from NRR/DRPE conducted a site visit at Seabrook Station on December 1-2, 1997. Tours of the facility and walkdowns of selected plant systems were conducted with the resident inspectors, and interviews/discussions were conducted with selected plant management and staff.

Management Meeting at Salem Unit 2

On Thursday, December 4, 1997, NRC Region I senior managers met with management and staff of Salem Unit 2. The meeting was open to public observation. The meeting was held to review the lessons learned during the restart of Salem Unit 2 and to discuss how these lessons learned will be applied to Salem Unit 1 restart. This meeting satisfied the sole remaining Confirmatory Action Letter (CAL) item relative to Salem Unit 2 restart, although the CAL remains open for Salem Unit 1 restart.

Oyster Creek Site Visit

On December 3 and 4, 1997, senior Region I management visited the Oyster Creek Nuclear Generating Station in Forked River, New Jersey. William Axelson, Deputy Regional Administrator; Richard Crlenjak, Deputy Director, DRP; Peter Eselgroth, Chief, Projects Branch 7; Gene Kelly, Chief, Systems Engineering Branch, DRS; and Ronald Eaton, Project Manager, NRR, toured the plant, met with the resident inspectors, and interviewed plant personnel and management.

Meeting With JAPEIC

Michael Modes and Jacque Durr, Region I Branch Chief in DRS and DRP, respectively, met with two representatives of Japan Power Engineering and Inspection Corporation (JAPEIC) on December 3 in Region I to discuss issues associated with post weld heat treatment. Bob Hermann of NRR took part by phone. This meeting was requested by JAPEIC, which is supporting the Ministry of International Trade and Industry on the issue.

Calvert Cliffs Radiation Protection Inspection

On December 3, 1997, three Ukranian visitors, Mikolai Dmitrenko - Head of Inspections for Radiological Protection - Main State Inspectorate, Ljudmyla Shevchenko - State Inspector, and Tanya Smirnova - interpreter, were briefed on the Region I organization and responsibilities. Topics included the inspection and enforcement process, the Systematic Assessment of Licensee Performance and Plant Performance Review processes, the role of the Incident Response Center and Emergency Preparedness capabilities, and radiation safety inspection programs. The visitors also accompanied Region I Senior Radiation Specialist, Ron Nimitz, on a radiation protection inspection at Calvert Cliffs.

Whittaker Corporation

On December 2, 1997, NRC Region I issued a Confirmatory Action Letter to Whittaker Corporation for their site in Greenville, PA. This site is listed on the Site Decommissioning Management Plan due to thoriated slag remaining onsite from past metallurgical melting operations. Manufacturing operations involving source material were terminated in 1974. A recent NRC inspection found that licensed radioactive material in the form of this thoriated slag has migrated beyond the fenced area and possibly into a nearby river. Although the material is in a form that does not pose an immediate or near term health and safety hazard, the licensee has agreed to recover all licensed material outside the owner-controlled fenced area by December 19, 1997, and also to generate a plan to ensure that the potential for any future migration outside the owner-controlled fenced area is minimized. This plan will be submitted to the NRC for review and approval by January 31, 1998.

---

ENCLOSURE P

Region II
Items of Interest
Week Ending December 5, 1997

Tennessee Valley Authority - Sequoyah, Browns Ferry

On December 2, NRR and Region II management conducted a plant tour and attended a management meeting at the Sequoyah Nuclear Plant regarding the Operational Performance of the Sequoyah units. TVA management presented performance indicators showing improved performance during a recent refueling outage, as well as a reduction in the frequency of personnel errors.

On December 5, representatives from Tennessee Valley Authority were in the Regional Office for a management meeting to discuss Browns Ferry self-assessments of performance. TVA management presented critical evaluations of performance in the areas of operations, maintenance, engineering, plant support, and the effectiveness of the Nuclear Assurance organization.

Florida Power and Light Company - St. Lucie

On December 3, representatives from Florida Power and Light Company attended a Predecisional Enforcement Conference in the Region II Office to discuss apparent violations regarding deficiencies in the maintenance, testing, and surveillance of containment cooling fans identified in St. Lucie NRC Inspection Report 50-335, 389/97-15.

Virginia Electric and Power Company - Surry

Representatives from Virginia Electric and Power Company were in the Regional Office on December 4 for an Enforcement Conference to discuss Surry's apparent violations involving inadequate vital bus isolation and breaker coordination of Appendix R, Fire Protection Equipment, Reporting Requirements and Inadequate Corrective Actions.

Duke Power - Oconee

Team leaders from two Augmented Inspections and one special inspection briefed the ACRS on the findings of their respective inspections. These three inspections were in response to the crack in a high pressure injection nozzle on Unit 2, the loss of two high pressure injection pumps on Unit 3, and a problem with the emergency electrical supply generators.

Florida Power Corporation - Crystal River 3

On December 3, the Deputy Regional Administrator and the Director of the Division of Reactor Safety responded to a request from the Office of Congressional Affairs and briefed Congresswoman Thurman (FL) on the status of Crystal River 3 restart activities.

---

ENCLOSURE P

Region III
Items of Interest
Week Ending December 5, 1997

Predecisional Enforcement Conference - Zion

On December 1, 1997, a Predecisional Enforcement Conference was conducted in the Region III Office, Lisle, Illinois, between management representatives of Commonwealth Edison Company and members of the NRC staff. The conference was held to discuss possible violations of the utility's fitness-for-duty program at the Zion Nuclear Power Station. The conference covered several instances between September of last year and June of this year when the Zion plant staff apparently did not follow the fitness-for-duty procedures when an employee may have consumed alcohol prior to reporting for work.

Braidwood SALP Public Meeting

On December 2, 1997, the NRC Region III Regional Administrator A. Bill Beach and other members of the regional and headquarters staffs met at the Braidwood Nuclear Power Station with representatives of Commonwealth Edison Company to review the recently issued Systematic Assessment of Licensee Performance (SALP) report for the Braidwood plant.

---

ENCLOSURE P

Region IV
Items of Interest
Week Ending December 5, 1997

Fort Calhoun Station Control Room Fire

At approximately 5 a.m. (CST) on November 29, operators in the Fort Calhoun Station control room detected an odor similar to an electrical component overheating. During the investigation of the odor, determined to be coming from a control room panel, a nonsafety-related relay in the pressurizer level control circuitry caught fire. The operators immediately extinguished the fire with a hand-held fire extinguisher. Failure of the relay caused an isolation of the letdown system when indicated pressurizer level failed low. Pressurizer level indication was shifted to an alternate channel and the letdown system was restored to service. The failed relay was replaced and the system was restored to normal operation. The failed relay was a 25-year old General Electric Model CR120A relay. The resident inspectors are reviewing the licensee's actions and potential generic implications.

Predecisional Enforcement Conference - River Bend Station

On December 2, a predecisional enforcement conference was conducted with representatives of Entergy Operations, Inc. from River Bend Station in the Region IV office in Arlington, Texas. The conference discussed the circumstances surrounding an inadvertent mode change which occurred on September 13, 1997, and a loss of shutdown cooling which occurred on October 4, 1997. The NRC was represented by the Regional Administrator, members of the Region IV staff, and representatives of the Office of Nuclear Reactor Regulation.