October 16, 1997

For: The Commissioners

From: James L. Blaha, Assistant for Operations, Office of the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING OCTOBER 10, 1997

James L. Blaha
Assistant for Operations, OEDO

Contact: G. Tracy, OEDO

Office of Nuclear Reactor Regulation
Items of Interest
Week Ending October 10, 1997

Visit to EPRI NDE Center to Observe Thermal Pressurization Tests

On September 23, 24, and 25, 1997, Gary Hammer, DE/EMEB, visited the Electric Power Research Institute (EPRI) Nondestructive Examination (NDE) Center in Charlotte, NC to observe the thermal pressurization testing of water solid sections of piping. The two sections of piping tested were both constructed of 3-inch SA312 TP304 stainless steel with standard end caps welded on the pipe ends. The first test section is a straight Schedule 40 pipe, and the second section has a Schedule 40 to Schedule 80 transition weld midway along the length. A third section constructed of 8-inch SA106B carbon steel pipe was scheduled to be tested in the near future. Also, an unofficial "pre-test" specimen, identical to the first test specimen observed, was tested during the previous week on September 19, 1997. All test specimens are five diameters in length, not including the end caps. EPRI also plans to perform burst tests in the near future on similar sections of pipe to verify the expected margin to rupture due to internal pressure.

Generic Letter (GL) 96-06 requires licensees to evaluate isolated sections of piping which are subjected to containment environment heating and take

appropriate corrective actions. This test program is being carried out with support from five utilities. EPRI's stated purpose for this testing is to establish a basis for obtaining schedular relief from implementing design modifications, to validate nonlinear structural analysis methods and models, and to obtain data which could refine pressure load estimates in ASME Appendix F pipe and valve integrity assessments.

The two tests observed were conducted by insulating and heating the outside of the pipe with an electrical resistance blanket type of heater. The heat was applied very slowly over several hours to allow the inside water to heat to within approximately 4 degrees F of the pipe temperature. Since the pressurization is caused by the expansion of the water being greater than that of the surrounding pipe, it was considered important to allow the water to be as hot as possible relative to the pipe. The specimens were heated from room temperature of about 70 F to over 300 F, which was believed to be representative of limiting conditions for sections of plant piping. The results of the tests are as follows:

Test	Date	Material & Size	Temp, F	Pres, psig	Hoop Strain, %
*	9/19/97	304SS 3"Sch 40	305	5600	2.4
#1	9/23/97	304SS 3"Sch 40	323	5700	3.0
#2	9/24/97	304SS 3"Sch 40/80	314	5660	5.3

Note: * A "pre-test" specimen

Prior to beginning the above tests, EPRI performed non-linear structural analyses of the above test configurations, which generally overpredicted the pressures and strains observed during the tests, except for the Schedule 40 to Schedule 80 transition pipe test. EPRI indicated that further refinements of the analyses would be made to better agree with the tests.

ERPI expects to brief the staff and issue a report within the next one to two months which would contain the results of all of the tests and their findings.

Level of Detail for AP600 Piping Design Certification

On September 23, 1997, the NRR staff performed a review at Westinghouse's office in Monroeville, Pennsylvania to determine the status of completion of the AP600 piping design. In previous reviews of advanced light water reactor (ALWR) plants for design certification (ABWR and System 80+), the piping design was not completed prior to design certification. As a result, the staff developed and the Commission approved the use of design acceptance criteria (DAC) in lieu of piping design details (SECY-92-053). However, Westinghouse is proposing for the AP600 plant to complete the piping design prior to design certification and, thus, eliminate the need for DAC. The purpose of the staff's review at Westinghouse's office was to determine the extent to which the level of detail of piping design will be completed by design certification and the current status of completion of the AP600 piping design.

Westinghouse is proposing to complete the stress analyses of all ASME Code Class 1, 2, and 3 large-bore piping systems (3 inches nominal pipe diameter and greater) by design certification. At this time, 80% of the piping systems have been completed and the remaining piping systems will be finished by the end of October 1997. Westinghouse is not planning to complete the stress analyses of the small-bore piping systems (less than 3 inches nominal pipe diameter) although most of these piping systems will be preliminarily routed and space allocated in the plant for them.

In addition, the staff found that Westinghouse did not analyze the large-bore piping systems for LOCA dynamic loads. These limiting loads are caused by the mass-energy release resulting from a postulated double-ended guillotine break of the feedwater piping at the steam generator nozzle. Nor has Westinghouse analyzed the submerged piping in the in-containment refueling water storage tank (IRWST) for the dynamic transient loads caused by the discharge of the first three stages of the pressurizer ADS valves. The IRWST loads were found by Westinghouse to be significant (up to 13 g's on the tank wall). Westinghouse did evaluate the impact of the IRWST loads on the reactor coolant loop and on the passive residual heat removal heat exchanger tubing, and found it to be small - mainly because these major components are not near or in direct line with the discharge loads.

In the area of high- and moderate-energy line break analyses, Westinghouse has completed its analyses of the 12 auxiliary lines and the reactor coolant loop piping for which leak-before-break (LBB) is assumed. For those non-LBB high-energy lines, Westinghouse has essentially completed the high-energy line break analyses (except for pipe whip restraint design details which will be completed by the COL applicant). Similarly, Westinghouse plans to complete the environmental qualification evaluation (e.g., flooding, subcompartment pressurization, temperature, and humidity) within the next month.

The staff discussed with Westinghouse its concerns with the above areas where the piping design has not been completed or finalized and also discussed with Westinghouse areas where the SSAR needs to be revised to clarify the responsibilities of the COL applicant in completing the AP600 piping design. The staff noted that for design certification the design needs to be complete to the extent that the staff is able to reach a final safety determination on the adequacy of the design as required by 10 CFR 52.47(a)(2). Westinghouse agreed to develop a plan of action to address how it intends to resolve the staff's concerns related to (1) unanalyzed LOCA dynamic loads in large-bore piping stress analyses and (2) the impact of IRWST loads on submerged safety-related piping. The staff is discussing possible approaches for the resolution of this issue to minimize the impact on schedule.

Meeting With NEI/NUSMG on Year 2000 Issue

On October 7, 1997, staff from the Instrumentation and Controls Branch (HICB), IRM, AEOD and NMSS met with the Nuclear Energy Institute/Nuclear Utilities Software Management Group (NEI/NUSMG) task force on the Year 2000 issue. The purpose of the meeting was to exchange information on industry efforts to address the problem and staff plans for future action to ensure that all licensees are effectively pursuing the problem. The NEI/NUSMG task force informed the staff that they have developed a guidance manual on establishing a program to address the year 2000 problem including guidance on scoping the issue, implementing corrective actions and documenting activities. The NEI/NUSMG guidance is patterned after programs being pursued by licensees such as Duke Power and Northeast Utilities. Training on the guidance will be provided to licensees in November at the NUSMG

conference in San Francisco, CA. The staff indicated that the guidance appears to provide a good plan for dealing with the issue. The guidance will be finalized and issued for information to all licensees and the NRC staff by the end of October 1997. However, because the NEI/NUSMG task force does not intend to ensure implementation of a Year 2000 program by each licensee, the staff indicated that a generic communication on the issue will likely be forthcoming in the near future. The communication will request licensees to confirm that they have a point of contact and a Year 2000 program comparable to the NEI/NUSMG guidance or equivalent, provide a schedule for completion of their efforts to be Year 2000 ready, and provide a certification of readiness prior to January 1, 2000. The staff indicated that sample inspections of licensee Year 2000 programs may also be conducted.

Crystal River 3 -- Restart Progress

Florida Power Corporation (FPC) continues to maintain its target for completing the corrective actions and restarting the facility by December 1997.

Licensing Submittals

FPC has identified 34 potential licensing submittals that could require NRC approval prior to restart in December 1997. The current status is as follows:

STATUS OF LICENSING SUBMITTALS THAT ARE REQUIRED FOR RESTART

Total Identified:	34
Awaiting initial submittal:	8
Received and review in progress:	16
Completed / Closed:	10

Key Meetings

The next Crystal River Restart Panel meeting is scheduled on October 30-31, 1997 at the Crystal River site. Both the Project Manager and the Project Director will attend the meeting.

Challenges

In the September Restart Panel meeting, the Panel observed that the licensee submittals required for restart, and completion of staff review and approval of them continue to be the critical path for restart. Specifically, staff review of two licensing issues relating to mitigation of a small break loss of coolant accident (LOCA) and prevention of boron precipitation are critical due to the complexity of the issues involved. On September 25, 1997, in the FPC Restart Progress meeting, the staff informed the licensee of these concerns.

The staff has also observed that the licensee's corrective action for addressing weaknesses in the implementation of 10 CFR 50.59 requirements is not fully effective. This is evidenced by the licensee's USQ determinations relating to (1) boron precipitation and (2) evaluation of the effect of dynamic LOCA effects. The licensee used methodologies different from its current licensing basis and concluded that it did not involve an USQ because the methodologies are based on a staff letter to Babcock and Wilcox Owners' group or are consistent with a revised Standard Review Plan. The staff continues to monitor the effectiveness of the licensee's corrective actions in this area.

Salem Nuclear Generating Station, Units 1 and 2

Salem Unit 2 - Power ascension is being held at 40% as the licensee troubleshoots an inoperable steam generator steam flow channel.

Salem Unit 1 - The Salem Assessment Panel will meet at the site on October 17, 1997, to discuss Salem Unit 1. Following the internal meeting, the Panel will meet with the licensee to review items that are needed to support restart of Salem Unit 1.

Site Visit - On October 16, 1997, Mr. Hugh Thompson, Jr., Deputy Executive Director for Regulatory Programs will visit Salem, accompanied by Hub Miller, who will have visited Hope Creek on the previous day.

Green Ticket - On September 30, 1997, David Lochbaum of the Union of Concerned Scientists sent a second letter raising concerns regarding the staff's handling of an exigent amendment on Salem Unit 2. The staff is preparing a response and will brief the Director of NRR on the issue.

Limerick Generating Station, Units 1 and 2

On October 3, 1997, the staff (NRR and RGN-I) held a conference call with the licensee to discuss the status and actions associated with the failure of a single control rod during the half-scram testing on September 25, 1997, for Limerick Unit 2. The staff held a conference call on Saturday, September 27, 1997, and also asked a series of questions on October 2, 1997. There has been a long history of problems with the diaphragms of scram solenoid pilot valves (SSPVs) in BWRs and the staff has been following industry efforts to resolve problems with the degraded scram times caused by both BUNA-N and VITON diaphragms. A new and improved VITON-B diaphragm is now considered as the acceptable design.

PECO had in place a program to replace all of the BUNA-N SSPVs for both units and identified that 58 and 60 BUNA-N SSPVs remained to be replaced for Unit 1 and Unit 2, respectively. The replacements are scheduled to be completed in

December 1997, for Unit 2, and April 1998 for Unit 1. The replacement schedule was tied to the age of the SSPVs. Also, based on inspection results,

failure of the SSPVs was attributed to air leakage through the diaphragms. The licensee has determined that all of the 58 control rods in Unit 1, with the BUNA-N diaphragms, are less than 4 years old (recommended replacement age).

The licensee tested the 60 control rods in Unit 2 with the BUNA-N SSPVs. PECO has found that 4 SSPVs exhibited slower than normal 5% insertion times, but

are within the TS for full insertion, and they have replaced them. Further, the licensee has committed to a weekly inspection of all of the BUNA-N SSPVs for both units, and has advanced the replacement of the remaining BUNA-N SSPVs to November 11, 1997, for Unit 2, and January 1998, for Unit 1. All of the replacements utilize the newer VITON-B diaphragms.

GE will test the following: the failed SSPV; 4 SSPVs identified with slow 5% insertion time; 1 SSPV with identified air leak and no identified 5% insertion time; and a normal SSPV to determine the root cause of the failure to scram. Both NRC and BWROG will evaluate the completed analysis for generic complications. The number of plants currently using BUNA-N diaphragms is believed to be limited. The BWROG is obtaining a count of those currently in use, as well as information on any that have experienced failure. PECO has agreed to document their commitments and provide the results of GE's analysis.

Quad Cities, Units 1 and 2

Inoperable Safe Shutdown Paths and Early Unit 2 Maintenance Outage

On 09/26/97 the licensee entered a 67 day LCO because all safe shutdown (SSD) paths were determined to be inoperable. In the Quad Cities SSD analysis, all cables within a fire area are postulated to fail in the event of a fire, except for those cables provided with passive fire protection. The SSD analysis relies on the opposite unit equipment to help place and maintain the affected unit in a safe shutdown condition. This assumes that the SSD equipment is not affected by the fire and that the breakers for the non-SSD loads on the same bus as the SSD equipment would be opened or shed from the bus. If the non-SSD load breakers are not shed from the bus and the cables for these loads pass through the fire zone, damage to these cables could cause loss of power to the SSD equipment on that bus for the unaffected unit.

Upon review of the Fire Protection Report (FPR), the station discovered two issues with the implementation of the SSD analysis. The first issue discovered was that even though the SSD analysis assumes shedding non-SSD loads, the station procedures do not implement all of the assumed load shedding. The second discovery was that the analysis that addresses the ability to shutdown the unaffected unit after shedding of non-SSD loads could not be located. The licensee did not have assurance that the SSD paths were operable. There is no analysis in place that ensures that equipment used to safely shutdown the unaffected unit will not be affected by fire in the affected unit, therefore the licensee declared all SSD paths inoperable.

Update: As of 10/08/97 the licensee says that the risk of core damage is less with Unit 1 operating at 100% power and Unit 2 shutdown and with no SSD paths available then with both units operating with all SSD paths available. The licensee has extended their 10 day commitment for procedure completion from 10/06/97 to 10/20/97 to correct additional problems that have been identified. Quad Cities is presently in day 12 of the 67 day administrative LCO. Unit 2 will not start up from Maintenance Outage Q2P01 until the procedures for Unit 2 have been reviewed/revised as necessary. The Region and NRR have had several conference calls with the licensee on this problem and are following this issue closely to ensure the appropriateness of the licensee's actions.

Unit 2 Maintenance Outage Q2P01

The licensee has completed fuel sipping and removal of the suspect leaking fuel assembly. Three other fuel assemblies were shuffled and one assembly was removed from the pool and inserted into the core. Reactor reassembly is in progress. During this outage the 250v battery was given a discharge test because of previous changes to the battery load profile. The load test was successful. The licensee also completed several TS surveillances that are required every 15 months that required a plant shutdown because the station fuel cycles are now 24 months long. Other activities included a PORV replacement and snubber testing in the containment. Critical path is the resolution of the fire protection issues addressed earlier.

Office of Nuclear Material Safety and Safeguards
Items of Interest
Week Ending October 10, 1997

Presentation at Local Nuclear Medicine Conference

On October 3, 1997, a member of the Division of Industrial and Medical Nuclear Safety staff delivered a presentation on release criteria for patients undergoing nuclear medicine procedures at the Nuclear Medicine Update 1998 Conference held at the Washington Adventist Hospital, Takoma Park, Maryland. The presentation covered the latest revision to 10 CFR 35.75, which became effective May 29, 1997, and associated Regulatory Guide 8.39, "Release of Patients Administered Radioactive Material." The conference, attended by about 110 medical professionals, provided participants with the latest information on cardiology, oncology, and regulatory issues in nuclear medicine.

Year 2000 Team Activities

In order to determine the potential significance of the Year 2000 problem for materials and fuel cycle licensees, the Office of Nuclear Material Safety and Safeguards (NMSS) developed an action plan to assess the scope of the Year 2000 problem for Nuclear Regulatory Commission materials licensees. A team consisting of representatives from the four NMSS technical divisions and a representative from the Office of Nuclear Regulatory Research

developed a questionnaire to be used in discussions with up to nine licensees to gather information about potential Year 2000 issues.

On October 8, 1997, the NRC team visited the National Institute of Standards Technology (NIST) and The Georgetown University for the initial meetings with the regulated community. Neither licensee reported any safety significant systems that are not Year 2000 ready. Both institutions reported that management was aware of the Year 2000 problem, and both have programs to resolve Year 2000 issues. NIST has a Year 2000 Workgroup which evaluates systems for Year 2000 compatibility. At Georgetown University, the University Information Service and computer specialists from the Office of the Dean of Research evaluate systems for Year 2000 compatibility. Both licensees have contingency plans for systems which may fail.

Meeting with the United Plant Guard Workers of America

On October 8, 1997, Nuclear Regulatory Commission staff members met with members of the United Plant Guard Workers of America union from the Portsmouth Gaseous Diffusion Plant. Topics discussed at the meeting included NRC security and safeguards inspections at the Gaseous Diffusion Plants (GDPs), application of overtime restrictions to guards, anticipated reductions in the guard force, and NRC regulations and requirements that apply to security at the GDPs.

Orientation Meeting with Department of Energy Regulatory Unit

From September 23-October 2, 1997, Division of Fuel Cycle Safety and Safeguards (FCSS) staff were in Richland, Washington, to participate in an orientation meeting with Department of Energy Regulatory Unit (DOE RU) staff and their consultants, and to initiate review of the British Nuclear Fuels, Ltd. (BNFL) Standards Approval Package (SAP) for the Hanford Tank Waste Remediation System (TWRS). BNFL is one of the privatization contractors for Hanford. The BNFL SAP, delivered on September 26, 1997, was accompanied by an all-day BNFL staff presentation to the DOE RU and FCSS staff.

On September 25, 1997, the FCSS staff had attended a public meeting hosted by DOE RU in Richland. At the public meeting, both BNFL and Lockheed Martin Advanced Environmental Systems, the other privatization contractor, presented the non-proprietary portions of their standards approval packages.

Meeting with the International Atomic Energy Agency

On October 8, 1997, representatives from the Arms Control and Disarmament Agency and the Nuclear Regulatory Commission briefed the Nuclear Energy Institute (NEI) on the status and potential impact of the International Atomic Energy Agency's Strengthened Safeguards System measures. Meeting participants included representatives from fuel fabricators, the U.S. Enrichment Corporation, and NEI. Additional meetings will be scheduled with NEI to further address the potential impacts on fuel fabricators and enrichers, as well as on reactors, and nuclear equipment and materials suppliers.

Completion of Groundwater Reclamation at Spook Mill

On October 8, 1997, Division of Waste Management staff concurred on the Department of Energy's (DOE's) Groundwater Compliance Action Plan (GCAP) for the Spook Uranium Mill Tailings Remedial Action Project site at Spook, Wyoming. The groundwater restoration phase of the Uranium Mill Tailings Remedial Action (UMTRA) Project was initiated by DOE's final Programmatic Environmental Impact Statement (PEIS). As set forth in the GCAP, DOE's approach requires no remedial action to the groundwater at the Spook site since there is no apparent risk to human health or the environment because there is no known exposure pathway. The Nuclear Regulatory Commission staff determined that the "no action" approach for this site satisfies the requirements set forth in the Uranium Mill Tailings Radiation Control Act of 1978 (UMTRCA), and the standards in 40 CFR 192, Subparts B and C for the cleanup of groundwater contamination resulting from the processing of ores for the extraction of uranium. NRC had previously concurred on the surface cleanup of the Spook site and licensed it for long-term care. With the NRC concurrence on the GCAP, remedial action for this site as required under the UMTRCA is considered complete. Spook is the first uranium mill tailings site where NRC has concurred on all of the actions required of DOE for both surface and groundwater remediation since the DOE PEIS was issued.

Office of Administration
Items of Interest
Week Ending October 10, 1997

United States Enrichment Corporation

The Chief, Information Security Branch (C/INFOSEC) participated in a Region III predecisional enforcement conference on October 9, 1997, with USEC regarding an apparent violation of NRC regulations at the Paducah Gaseous Diffusion Plant. The apparent violation involves USEC's discovery of uncontrolled classified matter in areas outside the Paducah Plant Control Access Area. The information from the conference will enable NRC to gain a common understanding of the facts, root causes, missed opportunities, etc., and make an enforcement decision. The C/INFOSEC attended the conference based on a request from the Region, and represented ADM by providing security and classification advice and assistance based on requirements contained in 10 CFR Part 95 and Paducah's commitments made in their NRC approved Security Plan for the Protection of Classified Matter.

Fire Prevention Week

Fire Prevention Week, which was inspired by the Great Chicago Fire of October 9, 1871, will be observed October 5-11. Since 1925, when President Calvin Coolidge first declared Fire Prevention Week an official national observance, the week containing the October 9th Chicago Fire anniversary has been the focus of an annual fire safety awareness campaign sponsored by the National Fire Protection Association (NFPA). Fire Prevention Week posters and information have been posted on bulletin boards in the White Flint Complex and a fire drill will be scheduled during the month of October.

PEPCO Electric Deregulation Meeting

DFS representatives attended a September 25, 1997, meeting, hosted by PEPCO, that addressed the deregulation of the electric industry and its effect on large government and private sector customers. New legislation deregulating the electric industry will allow full and open competition between companies to generate electricity. Deregulation will be phased in over four years with full competition by the Spring of 2001.

Restack Project

On October 6, 1997 the 16th floor phase of the Restack project began. After the workstations are removed, DFS will begin to perform the actual refurbishment work on the floor, to include new finishes, improved conference rooms and air circulation between the elevator lobby and the 'A' and 'B' corridors. All work is scheduled to be completed in the beginning of January, 1998. The floor will then be occupied by: the staff of the Executive Director for Operations; the Director, Office of International Programs; the Office of the Secretary; and the staff of the Office of Congressional Affairs.

Rulemaking Activities for the Week Ending October 10, 1997.

Information Collection Requirements: Statutory and Technical Amendments (10 CFR Chapter I) A final rule that amends NRC regulations implementing the Paperwork reduction Act was published in the Federal Register on October 6, 1997 (62 FR 52184). The final rule makes changes required by statute and technical correcting amendments. The final rule became effective October 6, 1997.

Documents Submitted for Publication

The following documents have been forwarded to the Office of the Secretary for signature and transmission to the Office of the Federal Register for publication:

A document that withdraws an advance notice of proposed rulemaking amendment entitled "Acceptability of Plant Performance for Severe Accidents; Scope of Consideration in Safety Regulations" (Part 50)

A direct final rule and the accompanying proposed rule entitled "Codes and Standards; IEEE National Consensus Standard" (Part 50)

Chief Information Officer
Items of Interest
Week Ending October 10, 1997

FOIA Requests Received during the 5-Day Period of October 2-9, 1997:

Pool leakage and pool water losses, non-power reactors. (FOIA/PA-97-382)

Nuclear material spills, reported for 1991 to Oct. 3, 1997, on disk. (FOIA/PA-97-383)

Senator Lauch Faircloth, NC, correspondence from 1992 to Oct. 6, 1997. (FOIA/PA-97-384)

MLTS database for Puerto Rico and U.S. Virgin Islands. (FOIA/PA-97-385)

SECY-86-318, Enclosure C (Conservatism in App. K & 50.46). (FOIA/PA-97-386)

Beaver Valley Plant, Duquesne Light, 1993 quality control allegations, RI-93-A-0027. (FOIA/PA-97-387)

V/R Wesson Division/Fansteel, Lake County Lake Bluff/Fansteel, Metals Division, Johnson Fire Proof Door Co. (FOIA/PA-97-388)

Monetary penalties against power plants since 1995. (FOIA/PA-97-389)

IMPAC list with purchase limitations. (FOIA/PA-97-390)

Financial compensation for damages and attorney fees, policies and procedures. (FOIA/PA-97-391)

Office of Human Resources
Items of Interest
Week Ending October 10, 1997

1997 Combined Federal Campaign (CFC) Kickoff Held

On October 8, 1997, the NRC's 1997 CFC Kickoff was held in the OWFN Commissioners' Conference Room. The kickoff included remarks by Chairman Jackson; L. Joseph Callan, EDO; Dale Yeilding, National Treasury Employees Union representative; and Dan Carney, CFC Loaned Executive. The Chairman and EDO also presented awards to the outstanding offices from last year's campaign. The guest speaker was Greg Wims, who represented the Boys and Girls Clubs of Greater Washington.

National Depression Screening Day Observed

On October 8, 1997, the NRC observed National Depression Screening Day by presenting a lunch-time workshop on depression at NRC Headquarters. The program was sponsored by the Employee Assistance Program (EAP), and EAP counselor Carolyn Jensen was the speaker. The workshop focused on the signs, symptoms, and treatments for depression. Participants were offered the opportunity to schedule a confidential depression screening with Ms. Jensen. Agency-wide employees were advised of the free, anonymous Depression Screening telephone service, provided by Federal Occupational Health (FOH), which has been extended through December 31, 1997. By calling 1-800-698-8267, employees access an interactive screening questionnaire that will provide them with immediate feedback and results and initial information on depression.

Flu Shots for Headquarters Employees

On October 7, 1997, flu shots were given to approximately 600 employees. The Health Center had additional staff on hand to administer the shots, which are especially recommended for any person who, because of age or underlying medical condition, is at increased risk for complications of influenza. For those who missed the initial program, flu vaccine will be available at the Health Center on Tuesday, Wednesday, and Thursday afternoons throughout October.

Arrivals		
ADJODHA, Michael	CHEMICAL ENGINEER (PFT)	NMSS
LARIZZA, Giuseppe	REACTOR ENGINEER (PFT)	RIII
MCCOY Gerald	PROJECT ENGINEER (PFT)	RII
Retirements		
ALSTON, Kevin	COMPUTER ASSISTANT (PFT)	IRM
BELTRACCHI, Leo	SR HUMAN FACTORS ENGR (PFT)	RES
KAMMERER, Carlton	DIRECTOR, DAS (PFT)	ADM
Departures		
GONECONTO, Gregory	CRIMINAL INVESTIGATOR (PFT)	OIG

Office of Public Affairs
Items of Interest
Week Ending October 10, 1997

Media Interest

Region I reported media interest in the latest fine levied against TMI - the largest since the accident in 1979. There was also media interest in the fire at Limerick and the Maine Yankee decommissioning meeting.

The San Francisco Examiner will be publishing an article on spent fuel shipments through the Bay area over the last fifteen years.

School Volunteers Program

Karen Cotton, NRR, was "shadowed" by an aspiring 9th grade nuclear engineer from Magruder H.S.

Press Releases

Headquarters:

97-148	NRC Staff Issues Demand for Information to Sierra Nuclear Corporation
97-149	NRC Advisory Committee on Nuclear Waste to Meet October 21-23 in Potomac and Rockville, MD
97-150	Note to Editors: ACRS meeting, Subcommittee on Reliability and Probabilistic Risk Assessment
97-151	Note to Editors: ACRS meeting, Subcommittee on Planning and Procedures
97-152	Note to Editors: ACRS meeting, Subcomittee on Safety Research Program
97-153	NRC to Hold Workshops on Revisions to Regulations on Medical Uses of Radioactive Material

Regions:

I-97-133	NRC Proposes $110,000 Civil Penalty Against Consolidated Edison for Violations at Indian Point 2 Nuclear Power Plant in New York
I-97-134	NRC Staff Proposes $210,000 Civil Penalty Against GPU Nuclear for Violations at Three Mile Island Nuclear Power Plant
II-97-72	NRC Staff Plans Performance Review Meeting at NFS Facility in Erwin, Tennessee
II-97-73	NRC Names New Resident Inspector at Brunswick Nuclear Power Plant
III-97-90	NRC Staff Proposes Two $55,000 Fines for Similar Violations at Braidwood and Byron Nuclear Power Stations

Office of International Programs
Items of Interest
Week Ending October 10, 1997

Ronald Hauber, OIP Director for Nonproliferation, Exports and Safeguards, participated in the meeting of the OECD Steering Committee for Nuclear Energy in Paris, France, on Thursday and Friday. Budget, membership, and interagency relationship issues were included on the agenda. Professor Adolf Birkhofer gave an oral status report on the High-Level Advisory Group on the Future Role of the OECD Nuclear Energy Agency (NEA). Mr. Lars Hoegberg, Chairman of the NEA Committee on Nuclear Regulatory Activities CNRA) and Sweden's representative to the Steering Committee, gave an oral report on CNRA issues and activities, and described a CNRA self-assessment carried out over the past year. Activities of the Radioactive Waste Management Committee and the Radiation Protection Committee were also presented at the meeting.

Office of the Secretary
Items of Interest
Week Ending October 10, 1997

Decision Documents Released to the Public			
Document		***Date***	***Subject***
1.	SECY-97-155	7/21/97	Staff's Action Regarding Exemptions from 10 CFR 70.24 for Commercial Nuclear Power Plants
	- SRM on 97-155	8/19/97	(same)
	- Voting Record on 97-155	8/19/97	(same)
2.	SECY-97-205	9/10/97	Integration and Evaluation of Results from Recent Lessons-Learned Reviews
	- SRM on 97-205	10/2/97	(same)
3.	SECY-97-162	7/25/97	Proposed Agreement for Cooperation Between the U.S. and Brazil
	- SRM on 979-162	9/18/97	(same)
	- Voting Record on 97-162	9/18/97	(same)
Negative Consent Documents Released to the Public			
1.	SECY-97-112	5/30/97	Staff Requirements - COMSECY-97-027 - Strategic Assessment Issue Paper: Staffing and Core Capabilities (DSI-18)
	- SRM on 97-112	10/2/97	(same)
Information Papers Released to the Public			
1.	SECY-97-221	9/30/97	Acceptance Guidelines and Consensus Standards for Use in Risk-Informed Regulation
2.	SECY-97-222	10/2/97	Weekly Information Report - Week Ending September 26, 1997
3.	SECY-97-218	9/29/97	Special Provisions for Transport of Large Quantities of Plutonium (Response to Staff Requirements Memorandum - SECY-97-215)
4.	SECY-97-219	9/30/97	Order Modifying License to Convert from High-Enriched to Low-Enriched Uranium Fuel (University of Massachusetts Lowell)

Commission Correspondence Released to the Public

1. Letter to Mr. Carl Lischeske, California Department of Health Services, dtd 10/2/97 concerns the Ward Valley low-level waste disposal facility

2. Letter to Mr. Alvin Alm, DOE, dtd 9/29/97 concerns placement of DOE employees issued Reduction in Force notices

3. Letter to Dr. P. Rama Rao, Indian Atomic Energy Regulatory Board, 9/26/97, concerns the implementation of three NRC/AERB nuclear safety projects

Federal Register Notices Issued

1. 10 CFR Part 50; Acceptability of Plant Performance for Severe Accidents; Scope of Consideration in Safety Regulations; Advance Notice of Proposed Rulemaking: Withdrawal

2. Advisory Committee on Reactor Safeguards; Meeting of the ACRS Subcommittee on Reliability and Probabilistic Risk Assessment; Notice of Meeting on October 21 and 22, 1997

3. Evaluation of Agreement State Radiation Control Programs; Implementation of the Integrated Materials Performance Evaluation Program and Rescission of a Final General Statement of Policy

4. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Safety Research Program; Notice of Meeting on November 4-5, 1997

5. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Planning and Procedures; Notice of Meeting on November 5, 1997

6. International Uranium (USA) Corporation; Docket No. 40-8681-MLA-2; Designation of Presiding Officer

7. 10 CFR Part 50; Codes and Standards; IEEE National Consensus Standard; Proposed Rule

8. 10 CFR Part 50; Codes and Standards; IEEE National Consensus Standard; Direct Final Rule

9. Advisory Committee on Reactor Safeguards; Meeting Notice for November 6-7, 1997

Region I
Items of Interest
Week Ending October 10, 1997

New Regional State Agreements Officer in Region I

Effective October 1, 1997, A. Duncan White will serve as Regional State Agreements Officer in Region I. Mr. White has served as a Senior Health Physicist in Region I's Division of Nuclear Materials Safety (DNMS), where he was involved in materials licensing, inspection and decommissioning activities. Prior to joining NRC, Mr. White served on the staff of the Radiation Protection Programs, New Jersey Department of Environmental Protection. He replaces Craig Gordon, who will transfer to Region I's Decommissioning and Laboratory Branch, DNMS.

Maine Yankee

A meeting was held with the public on Tuesday evening in Wiscasset, Maine, to discuss the decommissioning process for Maine Yankee. Maine State Senator Marge Kilkelly chaired the meeting, which about 60 people attended. Representatives from NRR, Region I, and Public Affairs provided information regarding the decommissioning process and answered questions from the public. Two television stations and four newspapers covered the two-hour long meeting. The NRC will meet with the public again on November 6, to receive comments on the licensee's Post Shutdown Decommissioning Activities Report.

Haddam Neck

Region I and NRR are implementing actions related to recent testimony filed by the Connecticut Department of Public Utilities in a rate case before the Federal Energy Regulatory Commission (FERC). On the issue of responsibility for decommissioning funding, the state's technical consultant made various assertions regarding prior licensee mismanagement warranting NRC staff followup. This week, Region I inspectors reviewed selected issues; and a task force prepared for a site visit next week. The results of the review are expected by late November 1997.

Susquehanna

On October 9, 1997, NRC held a SALP Management Meeting with PP&L in Berwick, PA. During a two day visit to the Susquehanna facility, Region I and NRR managers conducted plant tours and personnel interviews in preparation for the management meeting. The SALP report which was issued by Region I on September 26, 1997, noted a decline in licensee performance from Category 1 to Category 2 in three of the four SALP rated areas.

Region II
Items of Interest
Week Ending October 10, 1997

Carolina Power and Light Company - Harris

On October 7, Region II participated in an Emergency Exercise at the Shearon Harris facility. Other participants included the licensee, State and local emergency response organizations. NRC participation included Regional and Headquarters teams during the evening hours on the first day and with the State using ingestion pathway techniques on the second day.

Quarterly Press Conference

On October 7, the Regional Administrator held a Quarterly Press Conference in Raleigh, North Carolina to brief news media and interested public representatives. A key subject was a brief on safety-related hurricane preparations at commercial nuclear power plants and fuel manufacturing facilities in North Carolina and the rest of the nation.

Duke Energy Corporation - Oconee

On October 9, representatives from the Duke Energy Corporation's Oconee Nuclear Station attended a closed management meeting in the Region II Office.

The purpose of the meeting was to discuss the results of a recent initial operator examination. The examination, as originally prepared by the licensee, did not meet all NRC examiner standard requirements. This meeting served to improve communications of those requirements between the NRC and licensee.

Tennessee Valley Authority - Watts Bar

On October 14, Watts Bar Unit 1 was in the final stages to restart following its first refueling outage. During the outage, fuel assemblies with tritium-producing, burnable absorber rods were loaded into the reactor core. These rods are part of a project to assist the Department of Energy in testing tritium production in this type reactor. The licensee also resolved two problems during the outage. One problem was with debris in reactor coolant system (pieces of Flexitalic gasket 1 - 3 inch long). The licensee checked suspected sources such as RHR system body/bonnet gaskets and was unable to determine the source of the debris. The licensee performed an analysis and determined the acceptability of restart considering 2-4 pieces of debris remaining under the core plate and the possibility of similar debris yet to be flushed out.

A second problem was with cracked blowdown piping inside of steam generators (SG) 2 & 3. Because stress calculations conducted by the licensee and Westinghouse predict very little movement of the broken piping during normal and faulted conditions, and the fact that preservice inspections indicate that the piping was most likely broken prior to placing the SG's in service, the licensee plans to leave the blow-down piping in service without repair. The NRC reviewed the licensee's analysis, and determined that the licensee's actions are supported by the analysis.

Region IV
Items of Interest
Week Ending October 10, 1997

Omaha Public Power District (OPPD) Predecisional Enforcement Conference

On October 7, 1997, the Regional Administrator, members of the Regional and Headquarters staffs met with OPPD representatives for a predecisional enforcement conference at the Region IV office. The conference was held to discuss apparent violations involving the containment spray system at the Fort Calhoun Station being inadvertently rendered inoperable by operations personnel.

Office of Congressional Affairs
Items of Interest
Week Ending October 10, 1997

CONGRESSIONAL HEARING SCHEDULE, No. 35

OCA CONTACT	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Combs	TBA 406 DSOB	TBA	Markup	S. 8, Superfund Reauthorization	Sen. Chafee/Baucus Environment & Public Works
Rathbun	10/17 CA	4:00	Mikhail Gorbachev	Year 2000 Computer Problem	Reps. Horn/Maloney Gov't Mgmnt, Info & Technology Government Reform & Oversight

Gerke	10/21 2322 RHOB	1:30	TBA	Utility Deregulation: H.R. 655, H.R. 338, H.R. 1230, H.R. 1359, H.R. 1960	Reps. Schaefer/Hall Energy and Power Commerce
Gerke	10/22 2123 RHOB	10:30	TBA	Utility Deregulation: H.R. 655, H.R. 338, H.R. 1230, H.R. 1359, H.R. 1960	Reps. Schaefer/Hall Energy and Power Commerce
Keeling	10/23 366 DSOB	9:30	TBA	U.S.-China Nuclear Cooperation Agreement	Senators Murkowski/Bumpers Energy & Natural Resources
Keeling	10/27 342 DSOB	2:00	TBA	Nuclear Stockpile Safety	Sen. Cochran/Levin Intl Security, Proliferation & Federal Services Governmental Affairs