

July 23, 1997

SECY 97-157

For: The Commissioners
From: James L. Blaha, Assistant for Operations, Office of
the EDO
Subject: WEEKLY INFORMATION REPORT - WEEK ENDING JULY 18,
1997

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*No input this week.

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Office of Nuclear Reactor Regulation
Items of Interest
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Millstone 2 Continued Operation Without a Startup Rate Trip

A startup rate (SUR) trip was originally provided in the older ABB-Combustion Engineering (CE) plants with analog protection systems (Palisades, Fort Calhoun, Maine Yankee, Calvert Cliffs, and St. Lucie) to provide protection for a startup accident in which control rods are withdrawn at a maximum rate from a subcritical condition. Millstone 2 requested removal of the SUR trip from the plant technical specifications (TS) in 1978. NRC allowed elimination of trip from TS and also required physical removal from the reactor protection system (RPS) to eliminate any possible degradation that could affect other safety related equipment.

All other licensed CE analog plants still incorporate this trip in their RPS, although not necessarily in their TS. CE has recently informed their utilities of the original design intent of this trip and the inadvisability of removing it from the RPS without due consideration of the events against which it provides protection. CE recommended that utilities considering removal of this trip function from the RPS should assure themselves that the power excursion events initiated from subcritical conditions, which are currently protected by the SUR trip, will be afforded similar protection by other trips remaining in the RPS. Based on this, the current fuel vendor for Millstone 2, reanalyzed the uncontrolled control rod bank withdrawal event from a subcritical condition and credited the variable overpower (VOP) trip rather than the SUR trip. The Reactor Systems Branch has reviewed the revised analysis and has concluded that the VOP trip provides acceptable fuel protection and, therefore, continued operation of Millstone 2 without the SUR trip is acceptable.

Axial Power Distribution Anomaly Caused by Crud Buildup

Power distribution and reactivity anomalies have been observed at two Westinghouse plants, Callaway and Comanche Peak. The behavior is characterized by a gradual unexpected power shift toward the bottom of the core which continues until near end of cycle when the power shifts back to the top of the core. The anomaly is believed to be caused by subcooled nucleate boiling enhanced by crud deposition on the upper portion of the fuel.

In addition to the anomalous power distribution, estimated critical control rod position (ECP) deviations between actual and predicted values increased with core burnup due to the unexpected power shift. Therefore, an additional safety concern is the potential adverse impact that the nonconservative ECP deviations

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might have on shutdown margin. Specifically, the licensee for the Callaway plant has indicated that they will need to begin reducing power in mid-August to maintain shutdown margin within the technical specification limits and have increased the frequency that they verify shutdown margin to approximately twice a week.

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NRR and Region IV are following this issue and have requested a meeting with Westinghouse to discuss this anomaly in more detail.

Salem Unit 2 Restart Schedule

Salem, Unit 2, is planning to cool down for approximately four days beginning on July 23, 1997, to perform several corrective maintenance items, including the repair of leaks in both the primary and secondary systems. The licensee is projecting criticality late July - early August 1997. The staff continued to closely monitor the licensee's readiness for restart.

Zion, Units 1 and 2 Operational Readiness Demonstration Program

The staff met with Zion station management on 07/11/97, to discuss its preparations for restart of Unit 2. One of the subjects discussed was the licensee's Operational Readiness Demonstration Program. The purposes of the program are to verify that 1) the operators and shift crews can implement management's expectations for plant operations, 2) management supports operations through day to day plant activities, and 3) communication and directions for shift operations are effective. The licensee identified a series of plant operating tasks which includes routine and infrequent surveillance activities and off normal evolutions. These activities were then combined with the normal scheduled activities (maintenance repairs, preventive maintenance activities, out of services, etc.) to complete a station work and operations schedule. The demonstration program started on 07/07/97, and is expected to run through 08/15/97. The region will observe selected evolutions during the operations demonstration period.

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Office of Nuclear Material Safety and Safeguards
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Subgroup on International Atomic Energy Agency Safeguards

On July 15, 1997, an Office of Nuclear Material Safety and Safeguards staff member chaired the meeting of the Subgroup on International Atomic Energy Agency (IAEA) Safeguards in the U.S., which was held at Nuclear Regulatory Commission Headquarters in Rockville, Maryland. The meeting included representatives from the Department of State, the Department of Energy, and the Arms Control and Disarmament Agency. A representative from the Department of Defense participated as an observer. Topics of discussion included: potential eligibility of the Research and Test Reactor Fuel Element facility at Babcock and Wilcox (B&W) for IAEA safeguards, preparations for tritium production at an NRC-licensed reactor, finalization of an IAEA Facility Attachment for the tube vault at the Y-12 facility, IAEA inspections of high-enriched uranium downblending at B&W, and other current issues related to IAEA safeguards in the U.S.

Safeguards Workshop and Tour for Chinese Delegation

On July 8-9, 1997, in coordination with the Office of International Programs, Division of Fuel Cycle Safety and Safeguards staff conducted a workshop for representatives of the People's Republic of China National Nuclear Safety Administration. The primary focus of the workshop was on safeguards, but safety was also addressed. Presentations were provided on material control and accounting, licensing and inspection, and physical protection requirements, as well as on Nuclear Regulatory Commission programs for nuclear criticality, radiological, chemical, and fire safety. On July 10-11, 1997, the Chinese delegation was escorted by NRC staff for a tour and discussions at the Siemens Power Corporation fuel fabrication facility in Richland, Washington.

Meeting with the National Mining Association

On July 15, 1997, Division of Waste Management (DWM) staff met with representatives of the National Mining Association (NMA), in one of a continuing series of bimonthly meetings to discuss topics and issues affecting uranium recovery regulation. The DWM staff discussed the status of the uranium recovery portion of the final decommissioning rule, and the NMA presented feedback from the industry on the June 3-5, 1997, uranium recovery workshop. In addition, the DWM staff discussed the status of the Title II Standard Review Plan (SRP) currently under development and, in particular, plans to incorporate the existing Alternate Concentration Limit technical position into the SRP. DWM staff

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requested the NMA to urge licensees to provide the Nuclear Regulatory Commission with approximate submittal dates for future new license applications and amendment requests for planning purposes. Finally, DWM informed the NMA that the September meeting will be the last of the regularly-scheduled meetings with NMA, due to resource limitations, but that meetings to discuss significant issues can be held as needed.

Meeting with Detroit Edison Company (Fermi 1) on Decommissioning Options

On July 15, 1997, Nuclear Regulatory Commission staff met with representatives of the Detroit Edison Company (DECo), the Fermi 1 licensee. DECo briefed the NRC staff on the status of its evaluation of Fermi I decommissioning options. DECo noted that, based on this evaluation, they have determined that the industrial waste, such as sodium and asbestos, pose greater hazards than radiological contamination. DECo expects to make its plans for Fermi 1 decommissioning public within the next week.

Follow Up Safety Inspection of Source Production and Equipment Company, Inc.

On July 9-10, 1997, Spent Fuel Project Office staff conducted a follow up safety inspection at Source Production and Equipment Company, Inc. (SPEC). SPEC is the holder of several Certificates of Compliance for transportation packagings for radioactive material. During an earlier inspection (June 16-19, 1997), the team identified significant deficiencies regarding the test requirements of 10 CFR 71.73 and quality assurance (QA) controls over design and testing activities. Following the June inspection, SPEC committed to stop fabrication and develop corrective actions. The Nuclear Regulatory Commission acknowledged these commitments in a confirmatory action letter (CAL).

Since the June inspection, SPEC had successfully retested its transportation packagings, and implemented improved QA measures. However, the follow up inspection still found deficiencies in SPEC's QA program. In response to the July inspection, on July 14, 1997, SPEC provided the NRC a corrective action plan, committed to update its QA procedures with the help of an independent expert, and to retrain all personnel performing quality-related activities before fabrication activities are resumed.

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Office of Nuclear Regulatory Research
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Completion of First Phase of Tube Removals from Retired McGuire Unit 1 Steam Generators

The first phase of tube removals from the retired McGuire Unit 1 steam generators has been completed. Twenty previously plugged tubes were identified as candidates for removal based on the past inservice inspections. Plugs from those tubes were removed, and the tubes were reinspected to confirm the previous inspection results and to provide the basis for selecting the specific tubes to be pulled. Sections of 11 of the previously plugged tubes (106 feet total length), which included 26 tube support plate intersections and 23 indications identified by eddy current testing, were removed. The eddy current testing performed after the plugs were removed confirmed the presence of the indications detected in service that caused the tubes to be plugged and, in addition, identified from one to three new indications in each tube. Interestingly, 3 of the 20 tubes had new circumferential crack indications on the ID of the plugged tube near the top of the tube sheet, suggesting either that the previous inspection missed these small indications or that there was sufficient moisture inside the plugged tube to permit further degradation after the plug had been installed.

The next phase of the tube removal will involve removal of two tube sheet sections containing a total of 20 "undisturbed" tubes, with 8 of those tubes containing circumferential crack indications, and the removal of tube support plate sections containing 40 to 45 tubes, with 12 tubes containing crack indications. Cutting of these tubes will begin on July 23, 1997, and the entire activity is scheduled to be completed by August 8, 1997. Progress on this project has stayed close to the contractor's schedule, and the cumulative personnel dose incurred through this phase is approximately 2 person-rem below the target exposure. The entire project is expected to be completed below the total exposure limit of 35 person-rem that was established by the licensee.

MELCOR Code, Version 1.8.4

Development and validation of the latest version of the MELCOR code, version 1.8.4, is now complete, and the code is ready for release. This version of the code contains many new and important improvements reflecting, in part, recommendations from the MELCOR Cooperative Assessment Program (MCAP). MCAP involves participation by MELCOR users from research and regulatory organizations around the world. The code improvements include incorporation of the Larson-Miller creep rupture failure model

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for the bottom head, modeling of hygroscopic aerosol behavior in containments, and modeling for passive containment cooling systems. Many models, including the ability to model flow blockages in the core, downward (reverse) flow in the core, enhanced boron-carbide reactions with steam, and modeling of an zirconium oxide shell to support fuel, have been enhanced. These changes have resulted in a code that runs faster and is more robust and more versatile. Calculations have been performed to validate the new code version. These calculations include plant calculations (BWR and PWR) for different sequences (e.g., station blackout and induced steam generator tube rupture) and comparisons to experimental data (e.g., VANAM MB, CORA 13, and Phebus B9+). A letter formally announcing the availability of MELCOR 1.8.4 to the international members of the Cooperative Severe Accident Research Program and to domestic users will be issued soon.

Decommissioning Decision-Process Workshop

On July 15-16, 1997, members of the RES and NMSS staff met with contracting staff from Sandia National Laboratories to discuss implementation issues related to developing guidance on releasing sites under the new site cleanup standard. The meeting focused on a review of the spectrum of expected cases-- from a simple case where a screening methodology would be sufficient to a complex case, such as those being studied in the radionuclide behavior and transport program, where site-specific parameters would need to be developed. The information exchanged will assist the NRC staff in developing the implementation guidance for the rule. Public workshops addressing this guidance are planned for the fall of 1997.

Russian-American Hydrogeology Seminar

On July 8-9, 1997, RES representatives, attended the Joint Russian-American Hydrogeology Seminar sponsored by DOE, EPA, and RES/NRC at Lawrence Berkeley National Laboratory (LBNL). The seminar's objective was to bring together the leading scientists from Russian organizations working on radionuclide transport with their American counterparts to discuss field studies, data, and remediation efforts associated with radionuclide migration by means of ground-water transport. The seminar presentations focused on the environmental problems at the Mayak site dealing with ground-water plumes emanating from the disposal site at Lake Karachai toward the Mishelyak River; deep-well injection of liquid radioactive waste in Russia; and transport analyses at United States sites. Presentations and discussions in the seminar provided the RES staff with direct information on the Russian field studies and analyses of the large-scale radionuclide plume migrations and on LBNL's cooperative state-of-the-science analyses. The information will assist in evaluating

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potential radionuclide transport in ground- and surface-water systems at decommissioned sites in complex natural environments.

Status of Thermal-Hydraulic Test Facility (THTF)

To assess available facilities, the staff explored the possibility of whether the THTF test facility at Oak Ridge can be refurbished and operated to provide the data necessary for the thermal-hydraulic code improvement. In a telephone conversation with Oak Ridge National Laboratory, the staff discussed the state of THTF. The THTF program was sponsored by RES, and funding totaled approximately 15 to 20 million dollars over its course (in 1970 dollars). THTF was conceived and constructed in the mid-1970s. Its purpose was to study blowdown heat transfer in a rod bundle representative of the core. The focus of the tests was heat transfer under dispersed and film boiling conditions. It was capable of full-pressure, full-power conditions. The rod bundle

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consisted of 64 rods in an 8 x 8 array. The bundle was heavily instrumented with rod internal thermocouples. Void profile was measured with differential pressure gauges. The facility has been idle for approximately 15 years.

RELAP5 Maintenance

The closeout of RELAP5 code maintenance at INEEL has begun. RELAP5 code maintenance is being performed by SAIC.

The first task orders have been prepared to initiate work under the umbrella contract entitled "Thermal-Hydraulic Research." The work will entail modularization of TRAC-P to facilitate the consolidation effort; interfacial area transport experimentation and modeling; development of numerical techniques for developmental assessment (determining the fidelity of code comparison to data); and experimental investigation of entrainment in off-take pipes.

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Office for Analysis and Evaluation of Operational Data
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Incident Response Division (IRD) Emergency Response Coordinators Workshop

Representatives from AEOD/IRD conducted the 29th Emergency Response Coordinators Workshop in Region II on July 14-16, 1997. In attendance were the Emergency Response Coordinators, Assistants, and selected management from all NRC Regions. Areas of discussion included status of program FTE and new initiatives in the area of incident response. Also present were representatives from NMSS who briefed the group on developments related to incident response regarding materials.

Preliminary Notifications (PNs)

- a. PNO-I-97-041, Babcock & Wilcox, PERSONNEL CONTAMINATION
- b. PNO-I-97-042, This is a test for PN using WordPerfect 6.1 for Windows.
- c. PNO-I-97-042A, Ambric Engineering, Inc., PORTABLE GAUGE SOURCE BROKEN FROM SOURCE ROD AND BELIEVED LEFT AT JOB SITE
- d. PNO-I-97-043, Duquesne Light Co. (Beaver Valley 2), BEAVER VALLEY UNIT 2 SHUTDOWN GREATER THAN 72 HOURS
- e. PNO-II-97-037, Duke Power Co. (McGuire 2), SHUTDOWN IN EXCESS OF 72 HOURS FOR FAILED REACTOR COOLANT PUMP MOTOR STATOR
- f. PNO-II-97-038, Bluefield Medical Center, STRONTIUM-90 (SR-90) APPLICATOR MEDICAL MISADMINISTRATIONS
- g. PNO-III-97-058, Mallinckrodt Nuclear Medicine, TWO MOLYBDENUM-99/TECHNETIUM GENERATORS EXCEED DOT REQUIREMENTS
- h. PNO-IV-97-040, Agreement State, DERAILMENT OF RAIL CAR CONTAINING RADIOPHARMACEUTICAL GENERATORS
- i. PNO-IV-97-041, Agreement State Licensee (A. G. Wassenaar), THEFT OF PORTABLE GAUGE FROM STATE OF COLORADO LICENSEE
- j. PNO-IV-97-042, Arkansas Agreement State (Porocel Corporation), LOSS OF PROCESS LEVEL GAUGE

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Office of Administration
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U. S. Enrichment Corporation (USEC)

The Division of Facilities and Security (DFS) is coordinating NRC and DOE comments on USEC's preliminary information package that informs prospective bidders of the restrictions on foreign ownership, control or influence. NRC's security inspection team composed of representatives from DFS and IRM is scheduled to conduct an inspection of the Portsmouth gaseous diffusion plant (GDP) during the week of July 28, 1997. The plant will be inspected against the security requirements of 10 CFR Part 95 and the approved Portsmouth GDP security plan for the protection of classified matter.

Suspicious Package/Explosives Detection/X-Ray Training

Representatives of the U. S. Capitol Police will lead a three day training program (July 21-23, 1997) in the identification of suspicious packages, explosives detection and use of the x-ray screening equipment owned and used by NRC. Participants include NRC staff from Headquarters, the Technical Training Center, Region III, and the Walnut Creek Field Office; NRC contractors from the Headquarters mail room; and special agents in training from the FBI. The schedule includes training by NRC's Radiation Safety Officer and a trip to Quantico, VA for an explosives demonstration.

Chief Information Officer
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Significant FOIA/PA Requests Received during the 5-Day Period of
July 11-17, 1997:

Waste Management, Inc., referral of record from DOJ responsive to FOIA. (FOIA/PA-97-256)

Materials licensees in state of New Jersey and New York City area. (FOIA/PA-97-257)

Contracts \$100,000 and over, names, addresses and phone numbers for specified contracts. (FOIA/PA-97-258)

Commercial Facilities Management Services, Contract No. NRC-10-94-141. (FOIA/PA-97-259)

Neely Research Reactor, Georgia Institute of Technology, license revocation. (FOIA/PA-97-260)

Sequoyah Power Plant, Unit 1, 100% core thermal power rating. (FOIA/PA-97-261)

Unpaid Treasury Checks, Limited Payability Cancellation lists for calendar year 1996, \$1,000 and over, and non-individuals. (FOIA/PA-97-262)

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Office of Human Resources
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Arrivals

GORDON, Dennis
(REACTOR) (PFT)
NRR

SECURITY SPECIALIST

ORDAZ, Vonna

REACTOR SYSTEMS ENGR (PFT)
NRR

Retirements

CAMPBELL, Nancy
NRR

REACTOR ENGINEER (PFT)

Departures

HSU, Cynthia
ACRS

CLERK (OPFT)

Office of State Programs
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Environmental Justice Subcommittee Meeting

A representative from the Office of State Programs attended the July 17, 1997 meeting of the Interagency Working Group (IWG) on Environmental Justice (EJ) Subcommittee on Policy and Coordination held in Washington, DC. The status of the several outstanding items was provided, as follows: the Report to the President on Environmental Justice remains forthcoming; the Council on Environmental Quality (CEQ) continues to incorporate comments on the March 26, 1997 Revised Draft CEQ Guidance for Considering Environmental Justice under the National Environmental Policy Act (NEPA); a number of Subcommittee members continue to express the need for a memo from the President to all agency heads asking for continued support and commitment to promoting EJ throughout the Federal Government.

One of the various agenda items of interest to NRC was the Subcommittee's discussion of the relationship of Executive Order (EO) 12898, "Federal Actions to Address Environmental Justice in Minority Populations and Low-Income Populations" to EO 13045, "Protection of Children from Environmental Health Risks and Safety Risks." The Administrator for the Environmental Protection Agency (EPA) and the Secretary of Health and Human Services (HHS) co-Chair the Task Force on Environmental Health Risks and Safety Risks to Children. The Subcommittee Chair stated that Federal agencies are expected to support EO 13045, noting lead poisoning was the subject of an early EJ case under EO 12898.

The EPA announced its on-line EJ home page, which links to other Federal agencies, including HUD, DOE, HHS, DOJ, DOT. The home page has an interactive EJ calendar of events and all users are encouraged to add any appropriate EJ events to the calendar. The EPA home page address is es.inel.gov/oeca/oejbut.html (Note: There is no www:).

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Office of Public Affairs
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Media Interest

Chairman Jackson was interviewed by Channel 32, Howard University's television station.

FOX-TV Philadelphia aired a story last week about the proposed Salem restart.

Press Releases

Headquarters:

97-105 Note To Editors: ACRS Reports

Regions:

I-97-82 NRC Staff to Meet on July 17 With Northeast Utilities, Nuclear Energy Advisory Council Regarding Millstone Plant

I-97-83 NRC, Haddam Neck Officials to Meet on Radiation Control Problems

I-97-84 Radioactive Source Recovered at Delaware Job Site

I-97-85 NRC, Grand View Hospital Officials to Meet to Discuss Apparent Violations

I-97-86 NRC Maine Yankee Officials to Discuss Cable Separation Issues

I-97-87 NRC Rates Peach Bottom as "Superior" in Three Areas, "Good" in Fourth, in Agency's Latest Assessment of Plant's Performance

II-97-54 NRC Staff to Hold Conference With Centers for Disease Control to Discuss Apparent Violations

II-97-55 NRC Staff to Meet With FPC Officials in Washington on July 22 to Discuss Progress Toward Restart of Crystal River Nuclear Plant

III-97-66 NRC Staff Proposes \$50,000 Fine Against Wisconsin Public Service for Testing Violations at Kewaunee Nuclear Plant

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III-97-67

**NRC Staff to Meet With OSU Officials on Apparent
Violations of Requirements for Handling and
Storing Radioactive Materials**

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- IV-97-40** **Comanche Peak Rated 'Superior' in all Areas in Latest NRC Assessment Report**
- IV-97-41** **NRC to Meet With Power Resources, Inc. to Discuss Apparent Violations**

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Office of the Secretary
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Commission Decision Documents Released to the Public

<u>Document</u>	<u>Date</u>	<u>Subject</u>
1. SECY- 97- 110	5/29/97	Status Report on Implementation of Dam Safety Program
- SRM on 97- 110	7/11/97	(same)
- Commission Voting Record on 97- 110	7/11/97	(same)
2. SECY- 97- 117	6/3/97	Final Policy Statement on Restructuring and Economic Deregulation of the Electric Utility Industry
- SRM on 97- 117	7/10/97	(same)
- Commission Voting Record on 97- 117	7/10/97	(same)

Negative Consent Papers Released to the Public

1. SECY- 97- 120	6/16/97	Rulemaking Plan for Emergency Planning Requirements for Permanently Shutdown Nuclear
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Power
Plant
Sites 10
CFR Part
50.54(q)
and (t);
10 CFR
50.47; and
Appendix E
to 10 CFR
Part 50

- SRM on 97-120 7/10/97 (same)

Information Papers Released to the Public

- | | | | |
|----|-------------|---------|--|
| 1. | SECY-97-135 | 7/2/97 | Weekly
Information Report -
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Ending
June 27,
1997 |
| 2. | SECY-97-142 | 7/10/97 | Weekly
Information Report -
Week
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1997 |

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Federal Register Notices Released to the Public

1. Barnett Industrial X-Ray, Inc.; Docket No. 30-30691-CivP;
Establishment of Atomic Safety and Licensing Board
2. Radiological Criteria for License Termination, Final Rule:
10 CFR Parts 20, 30, 40, 50, 51, 70 and 72
3. Radiological Criteria for License Termination, Uranium
Recovery Facilities: Request for additional comment on
uranium recovery facilities: 10 CFR Parts 20 and 40

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Region I
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Reactive Inspection at Babcox & Wilcox

From July 9-11, 1997, Region I staff performed a reactive inspection at the Babcock & Wilcox facility in Parks Township, Pennsylvania. The inspection was an investigation of a June 26 incident involving three workers who were decontaminating a wall of one of the buildings at the facility. The room previously had been the site of a plutonium scrap recovery operation. Dust from the decontamination process resulted in significant plutonium contamination of the worker's protective clothing and resulted in a high breathing zone air sample result. Bioassay results were below detectable levels for all three individuals, and work has been suspended until an internal investigation is completed and corrective actions are implemented. Staff will review data gathered during the inspection as well as additional information to be provided by the licensee, and will conduct a telephone exit conference on Monday, July 21.

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Region II
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General Electric Company

On July 16, Region II participated in the General Electric fuel fabrication plant emergency exercise. Region II's participation included a Base Team and two site participants. The exercise was fully successful with all objectives being met and no exercise weaknesses.

Centers for Disease Control and Prevention (CDC)

The CDC met with Regional staff for a pre-decisional Enforcement Conference on July 17. The apparent violations discussed included failure to adequately secure licensed material, failure to conduct inventories of licensed material, failure to calibrate survey instruments (repeat) and posting issues. The licensee outlined corrective actions and indicated their plans to have an independent review of the radiation safety program performed.

Region III
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Prairie Island Dakota Community Inspection Observation

The Prairie Island Dakota Community participated in its first observation of an NRC inspection July 14-18, 1997, at the Prairie Island Nuclear Power Station. This inspection observation was the result of the recent Commission decision to afford the Prairie Island Community the same opportunity as adjacent states under the Commission's Policy Statement on Cooperation with States at Commercial Nuclear Power and Other Production and Utilization Facilities. The inspection which was observed by a representative of the Prairie Island Community covered radiation protection and effluents.

Predecisional Enforcement Conference with Ohio State University

The Deputy Regional Administrator and other members of the regional and headquarters staffs met with Ohio State University, Columbus, Ohio, officials on July 17, 1997, in the regional office for a predecisional enforcement conference. The conference addressed apparent violations identified in NRC inspections conducted between June 1996 and April 1997. These apparent violations included exceeding time limits for storage of radioactive waste, failing to make periodic inventories of radiation sources, failing to report lost or missing radioactive material or not reporting the loss promptly, and failing to properly secure packages containing radioactive material and laboratories where the material was used. Some of the apparent violations were similar to violations noted in earlier NRC inspections.

Region IV
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Pre-Decisional Enforcement Conference with Omaha Public Power District

On Monday, July 14th, the Regional Administrator and members of the Region IV and NRR staffs met with representatives of the Omaha Public Power District to conduct a pre-decisional enforcement conference. The purpose of the pre-decisional enforcement conference was to discuss several apparent violations regarding the Fort Calhoun Station fire protection program.

Meeting on River Bend Station (Entergy)

On July 17, 1997, Region IV held a public meeting with Entergy at the Region IV office. The purpose of the meeting was to discuss licensee implementation of its Strategic Plan for the River Bend Station. The meeting focused on transition from River Bend's Long Term Performance Improvement Plan to the new Strategic Plan. The meeting was attended by the Regional Administrator and the Director, Walnut Creek Field Office.

Enforcement Conference

On Friday, July 18, Region IV conducted a predecisional enforcement conference with a former San Onofre Nuclear Generating Station operations department supervisor. The conference was based on the former supervisor's apparent deliberate failure to report two arrests to his employer. The conference was conducted by conferencing between the regional office in Arlington, Texas and IRS offices in Laguna Niguel, California.

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**Office of Congressional Affairs
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CONGRESSIONAL HEARING SCHEDULE, <u>No. 27</u>					
OCA CONTACT	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Combs	07/22 366 DSOB	9:00	Dept. of Interior, GAO	S. 964, Ward Valley Land Transfer Act	Senators Murkowski/Bumpers Energy & Natural Resources

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