September 8, 2000

Mr. M. Wadley
Senior Vice President and
Chief Nuclear Officer
Nuclear Management Company
700 First Street
Hudson, WI 54016

SUBJECT: PRAIRIE ISLAND - NRC EXAMINATION REPORT 50-282/2000302(DRS);

50-306/2000302(DRS)

Dear Mr. Wadley:

A Nuclear Regulatory Commission (NRC) approved licensed operator test (retake) was administered to a Senior Reactor Operator applicant at your Prairie Island Nuclear Generating Plant on August 22, 2000. The applicant had failed the May 2000 initial licensed operator test. The retake test was graded and the results finalized on August 25, 2000. The applicant passed the test and was issued a Senior Reactor Operator license. The enclosed report presents the results of the test.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available **electronically** for public inspection in the NRC Public Document Room **or** from the *Publicly Available Records (PARS) component of NRC's document system (ADAMS)*. ADAMS is accessible from the NRC Web site at http://www.nrc.gov/NRC/ADAMS/index.html (the Public Electronic Reading Room).

We will gladly discuss any questions you have concerning this examination.

Sincerely

/RA/

David E. Hills, Chief Operations Branch Division of Reactor Safety

Docket Nos. 50-282; 50-306 License Nos. DPR-42; DPR-60

Enclosures: 1. Operator Licensing Examination Report

50-282/2000302(DRS); 50-306/2000302(DRS)

2. Simulation Facility Report

See Attached Distribution

cc w/encls: Site General Manager, Prairie Island

Plant Manager, Prairie Island

J. Bernstein, Deputy Commissioner, Minnesota

Department of Public Service

State Liaison Officer, State of Wisconsin

Tribal Council, Prairie Island Dakota Community

J. Jensen, Training Department

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DOCUMENT NAME: G:DRS\PRA2000302DRS.WPD

*See Previous Concurrence-

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OFFICE	RIII *	RIII *	RIII	
NAME	DMcNeil:sd	RLanksbury	DHills	
DATE	09/08/00	09/08/00	09/8/00	

cc w/encls: Site General Manager, Prairie Island

Plant Manager, Prairie Island

J. Bernstein, Deputy Commissioner, Minnesota

Department of Public Service

State Liaison Officer, State of Wisconsin

Tribal Council, Prairie Island Dakota Community

J. Jensen, Training Department

ADAMS Distribution:

CAC

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TJK3 (Project Mgr.)

T. Frye, NRR

A. Madison, NRR

S. Stein, NRR

J. Dyer, RIII

J. Caldwell, RIII

B. Clayton, RIII

SRI Prairie Island

DRP

DRSIII

PLB1

JRK1

BAH3

MAB1

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Nos: 50-282; 50-306 License Nos: DPR-42; DPR-60

Report Nos: 50-282/2000302(DRS); 50-306/2000302(DRS)

Licensee: Nuclear Management Company, LLC

Facility: Prairie Island Nuclear Generating Plant

Location: 1717 Wakonade Dr. East

Welch, MN 55089

Date: August 22, 2000

Examiner: D. McNeil, Chief Examiner

Approved by: David E. Hills, Chief

Operations Branch

Division of Reactor Safety

NRC's REVISED REACTOR OVERSIGHT PROCESS

The federal Nuclear Regulatory Commission (NRC) recently revamped its inspection, assessment, and enforcement programs for commercial nuclear power plants. The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas): reactor safety (avoiding accidents and reducing the consequences of accidents if they occur), radiation safety (protecting plant employees and the public during routine operations), and safeguards (protecting the plant against sabotage or other security threats). The process focuses on licensee performance within each of seven cornerstones of safety in the three areas:

Reactor Safety

Radiation Safety

Safeguards

- Initiating Events
- Mitigating Systems
- Barrier Integrity
- Emergency Preparedness
- Occupational
- Public

Physical Protection

To monitor these seven cornerstones of safety, the NRC uses two processes that generate information about the safety significance of plant operations: inspections and performance indicators. Inspection findings will be evaluated according to their potential significance for safety, using the Significance Determination Process, and assigned colors of GREEN, WHITE, YELLOW or RED. GREEN findings are indicative of issues that, while they may not be desirable, represent very low safety significance. WHITE findings indicate issues that are of low to moderate safety significance. YELLOW findings are issues that are of substantial safety significance. RED findings represent issues that are of high safety significance with a significant reduction in safety margin.

Performance indicator data will be compared to established criteria for measuring licensee performance in terms of potential safety. Based on prescribed thresholds, the indicators will be classified by color representing varying levels of performance and incremental degradation in safety: GREEN, WHITE, YELLOW, and RED. GREEN indicators represent performance at a level requiring no additional NRC oversight beyond the baseline inspections. WHITE corresponds to performance that may result in increased NRC oversight. YELLOW represents performance that minimally reduces safety margin and requires even more NRC oversight. And RED indicates performance that represents a significant reduction in safety margin but still provides adequate protection to public health and safety.

The assessment process integrates performance indicators and inspection so the agency can reach objective conclusions regarding overall plant performance. The agency will use an Action Matrix to determine in a systematic, predictable manner which regulatory actions should be taken based on a licensee's performance. The NRC's actions in response to the significance (as represented by the color) of issues will be the same for performance indicators as for inspection findings. As a licensee's safety performance degrades, the NRC will take more and increasingly significant action, which can include shutting down a plant, as described in the Action Matrix.

More information can be found at: http://www.nrc.gov/NRR/OVERSIGHT/index.html.

SUMMARY OF FINDINGS

ER 50-282/2000302(DRS); 50-306/2000302(DRS), on 08/21-22/2000; Nuclear Management Company, Prairie Island Nuclear Generating Station, Units 1 & 2, Initial Operator Licensing Activities. A retake initial license test was administered to a senior reactor operator license applicant who failed the May 2000, initial license test. The test was administered in accordance with NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," Revision 8. This examination implemented the operator licensing requirements of 10 CFR §55.45.

- The applicant passed the retake test and was issued a senior reactor operator license. (Section 4OA5.1)
- The retake examination submitted by the licensee was outside the acceptable quality range expected by the NRC. Significant modification or replacement was required for two of the five job performance measures. (Section 4OA5.1)

Report Details

4. OTHER ACTIVITIES (OA)

4OA5 Other

.1 <u>Initial Licensing Examinations</u>

a. Inspection Scope

An NRC examiner conducted an announced operator licensing initial retake test on August 22, 2000. The Prairie Island Nuclear Generating Station (PINGS) training department developed the retake operating test. One senior reactor operator (upgrade) applicant was reexamined.

b. <u>Findings</u>

The examiner reviewed the applicant's remedial training program and found the program addressed the applicant's weaknesses identified during the May 2000 test. The applicant's performance during the test indicated the program was effective. The applicant passed all of the administrative job performance measures (JPMs) with no observed weaknesses.

The licensee provided the test outline and proposed test on schedule. However, the submitted retake examination was outside the acceptable quality range expected by the NRC. Significant modification was required for two of the five submitted JPMs. One JPM was replaced due to duplication from the applicant's audit test. The other JPM was modified because failure to perform the JPM would have raised questions regarding adequate justification for denying a license. Although the sample size (five JPMs) was small due to the limited scope of the retest, the examiner noted that the licensee has a history of submitting examinations that require considerable modification prior to administration. In particular, four of the five licensee submitted examinations (1997 to present) had quality problems sufficient to deem recognition in the respective examination report. The licensee would be expected to incorporate lessons learned from previous efforts.

The licensee did not submit any post-examination comments.

The examiners did not identify any significant security concerns associated with the development or administration of the test.

4OA6 Management Meetings

Exit Meeting Summary

The inspector presented preliminary examination observations to members of licensee management on August 22, 2000. The licensee acknowledged the issues presented. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

<u>Licensee</u>

- M. Jones, Nuclear Training Department Instructor J. Kempkes, Nuclear Training Department Instructor
- D. Schuelke, Plant Manager
 T. Silverberg, General Superintendent Operations
- D. Westphal, Operations Training Superintendent

<u>NRC</u>

S. Ray, Senior Resident Inspector

	ITEMS OPENED, CLOSED AND DISCUSSED
<u>Opened</u>	
None	
Closed	
None	
<u>Discussed</u>	
None	

SIMULATION FACILITY REPORT

Facility Licensee: Prairie Island Nuclear Generating Station

Facility Licensee Docket Nos: 50-282/50-306

Operating Test Administered: August 22, 2000

The following documents observations made by the NRC examination team during the initial operator license examination. These observations do not constitute audit or inspection findings and are not, without further verification and review, indicative of non-compliance with 10 CFR 55.45(b). These observations do not affect NRC certification or approval of the simulation facility other than to provide information which may be used in future evaluations. No licensee action is required in response to these observations.

During the conduct of the simulator portion of the operating tests, the following items were observed:

ITEM DESCRIPTION

1. None