

January 15, 2002

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, New York 14649

SUBJECT: R. E. GINNA - NRC INSPECTION REPORT 50-244/01-11

Dear Dr. Mecredy:

On December 29, 2001, the NRC completed an inspection of your R. E. Ginna facility. The enclosed report documents the inspection findings which were discussed on January 11, 2002, with you and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. No findings of significance were identified.

Immediately following the terrorist attacks on the World Trade Center and the Pentagon, the NRC issued an advisory recommending that nuclear power plant licensees go to the highest level of security, and all promptly did so. With continued uncertainty about the possibility of additional terrorist activities, the Nation's nuclear power plants remain at the highest level of security and the NRC continues to monitor the situation. This advisory was followed by additional advisories, and although the specific actions are not releasable to the public, they generally include increased patrols, augmented security forces and capabilities, additional security posts, heightened coordination with law enforcement and military authorities, and more limited access of personnel and vehicles to the sites. The NRC has conducted various audits of your response to these advisories and your ability to respond to terrorist attacks with the capabilities of the current design basis threat (DBT). From these audits, the NRC has concluded that your security program is adequate at this time.

Dr. Robert C. Mecredy

2

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of the NRC's document management system (ADAMS). ADAMS is accessible from the NRC website in the Public Electronic Reading Room, <http://www.nrc.gov/reading-rm.html>.

Sincerely,

/RA/

Michele G. Evans, Chief  
Projects Branch 1  
Division of Reactor Projects

Docket No. 50-244  
License No. DPR-18

Enclosure: Inspection Report 50-244/01-11

Attachment 1: Supplemental Information

cc w/encl: P. Wilkens, Senior Vice President, Generation  
P. Eddy, Electric Division, Department of Public Service, State of New York  
C. Donaldson, Esquire, State of New York, Department of Law  
N. Reynolds, Esquire  
W. Flynn, President, New York State Energy Research  
and Development Authority  
J. Spath, Program Director, New York State Energy Research  
and Development Authority  
D. Stenger, Ballard Spahr Andrews and Ingersoll. LLP  
T. Wideman, Director, Wayne County Emergency Management Office  
M. Meisenzahl, Administrator, Monroe County, Office of  
Emergency Preparedness  
T. Judson, Central New York Citizens Awareness Network

Distribution w/encl: H. Miller, RA/J. Wiggins, DRA (1)  
 T. Bergman, RI EDO Coordinator  
 E. Adensam, NRR (ridsnrrdlpmlpdi)  
 R. Clark, PM, NRR  
 P. Milano, PM, NRR (Backup)  
 R. Fernandes, SRI - Ginna  
 M. Evans, DRP  
 S. Barr, DRP  
 R. Junod, DRP  
 Region I Docket Room (with concurrences)

DOCUMENT NAME: G:\BRANCH1\GINSTUFF\REPORTS\GIN0111.WPD

After declaring this document "An Official Agency Record" it **will** be released to the Public.

**To receive a copy of this document, indicate in the box: "C" = Copy without attachment/enclosure "E" = Copy with attachment/enclosure "N" = No copy**

OFFICE	RI/DRP		RI/DRP	
NAME	Cwelch/MGE for		MEvans/MGE	
DATE	01/15/02		01/15/02	

OFFICIAL RECORD COPY

U.S. NUCLEAR REGULATORY COMMISSION

REGION I

Docket No: 50-244  
License No: DPR-18

Report No: 50-244/01-11

Licensee: Rochester Gas and Electric Corporation (RG&E)

Facility: R. E. Ginna Nuclear Power Plant

Location: 1503 Lake Road  
Ontario, New York 14519

Dates: November 11 through December 29, 2001

Inspectors: R. A. Fernandes, Senior Resident Inspector  
C. R. Welch, Resident Inspector

Approved by: Michele G. Evans, Chief  
Projects Branch 1  
Division of Reactor Projects

## SUMMARY OF FINDINGS

IR 05000244-01-11 on 11/11-12/29/2001; Rochester Gas & Electric; R. E. Ginna Nuclear Power Plant. Resident Operations Report.

The inspection was conducted by resident inspectors. The significance of most findings is indicated by their color (Green, White, Yellow, or Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process (SDP)." Findings for which the SDP does not apply are indicated by "No Color" or by the severity level of the applicable violation. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described at its Reactor Oversight Process website at <http://www.nrc.gov/reactors/operating/oversight.html>.

A. Inspector Identified Findings

None

B. Licensee Identified Violations

None

## Report Details

### SUMMARY OF PLANT STATUS

Genna operated at full power throughout the inspection period.

#### 1. REACTOR SAFETY

##### Initiating Events, Mitigating Systems, and Barrier Integrity [Reactor - R]

#### R01 Adverse Weather Protection

##### a. Inspection Scope

The inspectors reviewed RG&E's cold weather protective measures to ensure that systems required for safe operation would remain functional when challenged by adverse weather conditions. The inspectors performed walkdowns of plant areas susceptible to freezing conditions, to verify adequate space heating. In addition, heat trace circuits were verified to be energized, the intake structure heaters were verified available, and external ventilation openings were verified secured for the winter as required. Procedures M-1306.1, "Genna Station, Maintenance Department Winterizing Inspection Program," and A-54.4.1, "Cold Weather Walkdown Procedure," were used as references.

##### b. Findings

No findings of significance were identified.

#### R04 Equipment Alignment

##### a. Inspection Scope

The inspectors performed partial walkdowns of the following system trains while their redundant train(s) were out of service for maintenance.

- Safety Injection Trains A, B, and C.
- Emergency Diesel Generator B.

These inspections reviewed alignment of system valves and electrical circuit breakers to ensure proper in-service or standby configurations, as described in plant procedures and drawings. During the walkdowns, the inspectors also evaluated material conditions and general housekeeping of the systems and adjacent spaces.

A complete walkdown was performed of the accessible portions of the control room ventilation system, while operating in the emergency re-circulation mode (MODE F). The material condition of the system, damper alignment, and general housekeeping of the adjacent spaces were evaluated. The updated final safety analysis report, technical specifications, system diagrams, and station surveillance procedures were reviewed to determine the proper system alignment and operating requirements. Current and future modifications (see R17) to the system were reviewed and/or discussed with the system

engineer. RG&E's immediate corrective action for ACTION Report 2001-2113, which documented the failure to verify that stroke times for the redundant control room isolation dampers were within the assumptions of the control room dose calculation analysis, was reviewed.

b. Findings

No findings of significance were identified.

R05 Fire Protection

a. Inspection Scope

The inspectors toured the following plant areas to assess RG&E's control of combustible materials and ignition sources, and the physical condition of installed fire suppression and detection systems. In addition, the inspectors observed a planned fire drill conducted in the All-volatile treatment (AVT) room during the inspection period.

- Turbine building basement
- Contaminated storage building
- All-volatile treatment (AVT) room

b. Findings

No findings of significance were identified.

R12 Maintenance Rule Implementation

a. Inspection Scope

The inspectors reviewed RG&E's maintenance rule implementation for the following performance problems. This inspection evaluated system scoping, performance criteria/goal monitoring, and problem classification.

- AR 2001-1757, Service water pump B tripped.
- AR 2000-1268, Loss of service air header pressure.
- AR 2001-1426, Trip of service air compressor due to high ambient temperatures.
- AR 2001-1028, Trip of service air compressor due to high intercooler temperatures.

b. Findings

No findings of significance were identified.

R13 Maintenance Risk Assessments and Emergent Work Control

a. Inspection Scope

The inspectors evaluated the effectiveness of RG&E's maintenance risk assessments required by paragraph a(4) of 10 CFR 50.65. This inspection included discussions with control room operators and scheduling department personnel regarding the use of RG&E's online risk monitoring software. The inspectors reviewed equipment tracking documentation, daily work schedules, and performed plant tours to gain reasonable assurance that actual plant configuration matched the assessed configuration. Additionally, the inspectors verified that RG&E's risk management actions, for both planned and/or emergent work, were consistent with those described in procedure IP-PSH-2, "Integrated Work Schedule Risk Management." Risk assessments for the following out of service systems, structures, and/or components were reviewed.

- November 26, B emergency diesel generator and B motor driven auxiliary feedwater train.
- December 17, B residual heat removal train, A control room toxic gas monitor train, and containment penetrations P205 and P 206.
- December 20, emergent work to replace MQ-400E, TWINCO fast response AC regulator.

b. Findings

No findings of significance were identified.

R15 Operability Evaluations

a. Inspection Scope

The inspectors reviewed the following operability evaluations to determine if system operability is properly justified.

- AR 2001-1460, main steam isolation bypass valves (MS-3614/3615) seismic qualification.
- AR 2001-1889, Internal Corrosion of differential pressure indicators and switches.
- AR 2001-0447, NaOH tank outlet valve (836B) fluctuations.

This inspection included discussion with plant personnel and reviews of applicable technical specifications and design bases information.

b. Findings

No findings of significance were identified.

R17 Permanent Plant Modifications

a. Inspection Scope

The inspectors reviewed plant change record (PCR) 99-004 Revision 0, "Control Room Radiation Monitor Replacement." The PCR installed redundant radiation detectors into the control room ventilation intake duct along with the required instrumentation and associated conduit and cabling within the control room. The PCR also modified the control room auxiliary bench board, installing redundant trains of control room isolation logic for improved system reliability and flexibility. The inspectors reviewed design documents, associated 50.59 evaluations, various wiring diagrams, the post modification testing procedure, the updated final safety analysis report, and current technical specifications (T.S.). The inspector verified that the modification did not affect proper operation of the existing control room radiation monitoring system nor prevent it from performing its associated isolation function, as required for compliance with existing technical specification requirements.

b. Findings

No findings of significance were identified.

R19 Post Maintenance Testing

a. Inspection Scope

The inspectors reviewed the post maintenance tests for the following work orders (WO) to verify that RG&E appropriately demonstrated the components' ability to perform their intended safety function.

- W.O. 20102263, replace main control board control switch for the B emergency diesel generator supply breaker to bus 17.
- W.O. 19900065, plant change record (PCR) 99-004, Control room radiation monitor replacement.
- W.O. 20103612, replace diaphragms in reactor make-up water system manual valves

b. Findings

No findings of significance were identified.

**R22** Surveillance Testing**a.** Inspection Scope

The inspectors witnessed the performance and/or reviewed test data for the following activities to verify that the tests demonstrated the associated system's functional capability and operational readiness.

- PTT-23.52, "Containment Isolation Valve Leak Rate Testing Fire Service Water Pen 307." (Performed for Check valve 9229 only.)
- PT-32A Reactor trip Breaker Testing- "A" Train
- PT-13 Fire Pump Operation and system alignment, rev 85

**b.** Findings

No findings of significance were identified.

**R23** Temporary Plant Modifications**a.** Inspection Scope

The inspectors reviewed Temporary Modification (TM) 2001-0012, "Safety Injection (SI) Accumulator Makeup Pump," and walked down the installed system to verify that the TM did not adversely affect the reliability or functional capability of the safety system. The updated final safety analysis report, technical specifications, and system drawings were referenced.

**b.** Findings

No findings of significance were identified.

**Emergency Preparedness [EP]****EP6** Drill Evaluation**a.** Inspection Scope

The inspectors observed two simulator training scenarios and the subsequent critique to evaluate RG&E's performance in emergency preparedness drill/training. This inspection focused on the risk significant activity of making timely emergency action level (EAL) classifications and notifications. The inspectors reviewed NEI 99-02 to verify that the training exercise was of appropriate scope to be included in the performance indicator (PI) statistics.

b. Findings

No findings of significance were identified.

**4. OTHER ACTIVITIES [OA]**

OA1 Performance Indicator Verification

a. Inspection Scope

The inspectors verified the accuracy and completeness of the safety system unavailability (SSU) performance indicators (PI) below, for the second and third quarter of 2001. Data reviewed included the control room operator and equipment logs, action reports, and A-52.4 tracking documentation.

- High pressure injection system.
- Heat removal (auxiliary feedwater) system.

b. Findings

No Findings of significance were identified.

OA6 Meetings

a. Exit Meeting Summary

On January 11, 2002, the inspectors presented their overall findings to members of RG&E management led by Dr. R. Mecredy. RG&E management acknowledged the findings presented. No proprietary information was identified.

## Attachment 1

## Supplemental Information

a. Key Points of ContactRG&E

P. Bamford	Primary Systems and Reactor Engineering Manager
R. Biedenbach	Safety/Fire Coordinator
M. Flaherty	Nuclear Safety & Licensing Manager
T. Harding	Nuclear Safety & Licensing
R. Marchionda	Nuclear Assessment Department Manager
F. Mis	Acting Radiation Protection and Chemistry Manager
R. Ploof	Balance of Plant Systems Engineering Manager
R. Popp	Production Superintendent
M. Ruby	Nuclear Safety & Licensing
J. Smith	Maintenance Superintendent
P. Swift	System Engineering
R. Teed	Nuclear Security Supervisor
J. Wayland	I&C/Electrical Maintenance Manager
T. White	Operations Manager
J. Widay	VP, Plant Manager
G. Wrobel	Nuclear Safety & Licensing Manager
J. Zulawski	Performance Monitoring

b. List of Items Opened, Closed, and Discussed

None

c. List of Documents Reviewed

Action report 97-0141 (partial)  
 PT-17.4, Control Room Radiation Monitor R-36, R-37, R-38 Operability Test  
 PT-17.7A, Control Room Toxic Gas Monitor Operability Test - A Train  
 PT-17.7B, Control Room Toxic Gas Monitor Operability Test - B Train  
 Change Impact Evaluation PCR 99-004  
 DA-EE-2001-009 Rev 1, PCR 99-004 Electrical Factors Analysis  
 Electrical I&C Analysis Impact Form PCR 99-004  
 DA-EE-2000-009 Rev 0, Instrument Loop Number RMS 45/46  
 DA-CE-2001-056 Rev 0, Deviations from Standard Seismic Conduit Supports and Conduit Shield  
 PCR 99-004 Rev 0, Control Room Radiation Monitor Replacement  
 SM-99-004.1 Rev 1, Control Room Radiation Monitor Replacement  
 EE-171 Rev 1, Control Room Radiation Monitor Specification

d. List of Acronyms Used

AVT	All-volatile treatment
EAL	Emergency Action Level
IMC	Inspection Manual Chapter
NRC	Nuclear Regulatory Commission
RG&E	Rochester Gas and Electric Corporation
SDP	Significance Determination Process
SSU	Safety System Unavailability