



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM MUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23TB5
ATLANTA, GEORGIA 30303-8931

October 21, 2004

Duke Energy Corporation
ATTN: Mr. R. A. Jones
Site Vice President
Oconee Nuclear Station
7800 Rochester Highway
Seneca, SC 29672

SUBJECT: ASSESSMENT FOLLOWUP LETTER - OCONEE NUCLEAR STATION

Dear Mr. Jones:

In a letter dated September 24, 2004, the NRC informed you of the final significance determination of a White inspection finding in the mitigating systems cornerstone for Oconee Units 1, 2, and 3. This finding involved fire response procedures that were not consistent with the licensing basis with regard to the criteria for manning the standby shutdown facility (SSF). Additionally, you were informed in the letter that, as a result of this White finding, plant performance had been determined to be in the Degraded Cornerstone Column for all three Oconee units, because of a previously identified White finding in the mitigating systems cornerstone involving pressurizer ambient heat losses in all three Oconee units that exceeded the capacity of those pressurizer heaters powered from the SSF (EA-03-145). That letter also informed you that, because Oconee Units 1, 2, and 3 plant performance had been determined to be in the Degraded Cornerstone Column, we would use the NRC Action Matrix to determine the most appropriate NRC response for the finding, and notify you by separate correspondence of our determination.

The purpose of this letter is to inform you that we plan to conduct a Supplemental Inspection at Oconee Nuclear Station during the February/March 2005 time frame. This time frame was confirmed in a phone call with Mr. G. Davenport, Compliance Manager, of your staff on October 20, 2004. In accordance with NRC Inspection Manual Chapter 0305, Operating Reactor Assessment Program, the inspection will be conducted using NRC Inspection Procedure 95002, Inspection for One Degraded Cornerstone or Any Three White Inputs in a Strategic Performance Area. This inspection procedure is conducted to provide assurance that the root and contributing causes for the individual and collective risk significant performance issues are understood, to independently assess the extent of condition, and to provide assurance that the corrective actions are sufficient to prevent recurrence.

DEC

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In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter will be available electronically for public inspection in the NRC Public Document Room (PDR) or from the NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

If you have any questions, please contact me at (404) 562-4669.

Sincerely,

\RA

D. Charles Payne, Chief
Engineering Branch 2
Division of Reactor Safety

Docket Nos: 50-269, 50-270, 50-287
License Nos: DPR-38, DPR-47, DPR-55

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