

Fermi 2

1Q/2014 Plant Inspection Findings

Initiating Events

Significance: G Jun 30, 2013

Identified By: NRC

Item Type: FIN Finding

Not Following AQP and NPOA for Switchyard Modifications

The inspectors identified a finding of very low safety significance for the licensee's failure to follow the augmented quality program (AQP), nuclear plant operating agreement (NPOA), and Updated Final Safety Analysis Report (UFSAR) for plant modifications installed in the 345-kilovolt (kV) and 120-kV switchyards by the International Transmission Company (ITC) around September 2011. Specifically, the ITC liaison did not notify his counterpart at Fermi of the planned installation of new equipment in the switchyards, but no condition assessment resolution document (CARD) was issued or other communication made to Fermi 2 plant support engineering to conduct the required evaluation of proposed design modifications. In addition, no 10 CFR 50.59 review was performed of proposed changes to a modification.

The finding was determined to be more than minor because the inspectors did not see a similar example in IMC-0612, Appendix E, "Examples of minor issues." Further, because the licensee (nor ITC) had performed any design evaluation to assure the proposed activity would not have an adverse impact on the plant, the inspectors concluded that if left uncorrected this failure to perform a systematic design process in accordance with the AQP, NPOA, and UFSAR could lead to more significant safety concerns. Therefore, the issue screened as being more than minor. The inspectors evaluated the significance of the finding using Inspection Manual Chapter (IMC) 0609, Appendix A, "The Significance Determination Process (SDP) for Findings at-Power," "Exhibit 1 – Initiating Events Screening Questions," and answered "no" to the "Transient Initiators" question, "Does the finding contribute to both the likelihood of a reactor trip AND the likelihood that mitigation equipment or functions will not be available?" Therefore, the issue screened as having very low safety significance (Green). The finding has a cross-cutting aspect in the area of human performance, work control, because the licensee did not properly coordinate with ITC on the switchyard work to ensure the requirements of the AQP, NPOA, and UFSAR were met (H.3(b)).

Inspection Report# : [2013003](#) (*pdf*)

Significance: G Jun 30, 2013

Identified By: NRC

Item Type: FIN Finding

Failure to Maintain Configuration Control during Plant Operation

The inspectors identified a finding of very low safety significance for the licensee's failure to maintain configuration control during plant operations. Specifically, the inspectors identified multiple instances concerning the improper storage of equipment and control of scaffolding from January 1 through June 30, 2013. These instances did not meet the requirements of several licensee programs and management expectations.

The multiple instances constitute a programmatic issue with configuration control. This issue is more than minor because if left uncorrected would lead to a more significant safety concern and is similar to Inspection Manual Chapter (IMC) 0612, Appendix E, Section 4, Example a, in that the licensee routinely failed to perform procedurally-

required engineering evaluations on similar issues. Specifically, multiple examples were identified where the licensee placed items in the plant without proper engineering evaluation. The inspectors evaluated the significance of the finding using IMC 0609, Appendix A, “The Significance Determination Process (SDP) for Findings at-Power,” “Exhibit 1 – Initiating Events Screening Questions,” and answered “no” to the “Transient Initiators” question, “Does the finding contribute to both the likelihood of a reactor trip AND the likelihood that mitigation equipment or functions will not be available?” Therefore, the issue screened as having very low safety significance (Green). The finding has a cross-cutting aspect in the area of human performance, work practices, because the licensee either failed to follow established procedures or removed the controls from applicable procedures (H.4(b)).

Inspection Report# : [2013003](#) (pdf)

Significance: G Jun 14, 2013
 Identified By: Self-Revealing
 Item Type: FIN Finding

Failure to Implement Foreign Material Exclusion Procedure Requirements Adversely Affected the Reliability of the Main Turbine Generator and Caused a Reactor Scram

A finding of very low safety significance was self-revealed from an event that resulted in a reactor scram. The licensee failed to correctly implement its foreign material exclusion procedure following a reactor scram on September 30, 2009. The scram was caused by a turbine trip which was caused by the presence of a very small metallic particle (foreign material) that had bored into a main generator stator bar over time and created a hole that allowed hydrogen cooling gas to leak into the stator cooling water system. The ineffective corrective actions resulted in a second reactor scram for the same cause on November 7, 2012. Because the main turbine generator is not safety-related, no violation of regulatory requirements was identified. The licensee implemented appropriate mitigation actions until a permanent corrective action involving replacement of the generator or a modification to the existing stator design can be implemented.

The finding was of more than minor significance because this issue was associated with the Equipment Performance attribute of the Initiating Events Cornerstone and adversely affected the cornerstone objective of limiting the likelihood of events that upset plant stability and challenge critical safety functions during power operations. Specifically, inadequate foreign material exclusion controls coupled with a stator design that allows magnetized particles to be trapped in between the stator bars resulted in a reactor scram following development of a hydrogen leak through a stator bar. The finding was of very low safety significance because the issue: (1) did not involve a loss-of-coolant accident initiator; (2) did not cause a reactor trip AND the loss of mitigation equipment; (3) did not involve the complete or partial loss of a support system that contributes to the likelihood of, or cause, an initiating event AND affect mitigation equipment; and (4) did not increase the frequency of a fire or internal flooding initiating event. The inspector did not identify a cross-cutting aspect related to this finding.

Inspection Report# : [2013009](#) (pdf)

Mitigating Systems

Significance: G Mar 31, 2014
 Identified By: NRC
 Item Type: NCV NonCited Violation

Failure to Control the Work Hours of a Covered Worker

The inspectors identified a finding of very low safety significance with an associated non-cited violation of 10 CFR 26.205(c) and (d) for the licensee’s failure to schedule and control the work hours of a covered worker directing and

performing surveillance testing on a safety-related check valve during the refueling outage. Specifically, an engineer performing inservice testing was scheduled successive 12 hour shifts and was inappropriately excluded from the work hour limits specified in 10 CFR 26.205(d)(1) and (d)(2). As part of its corrective action, the licensee removed the engineer from covered work activities for the remainder of the refueling outage and reviewed the work activities of other engineers to ensure that any engineer performing covered work appropriately met work hour limits.

The finding was of more-than-minor significance since the failure to schedule and control the work hours of a worker performing covered work, if left uncorrected, would become a more significant safety concern because it could reasonably result in human performance errors that could affect the function of safety-related structures, systems, and components. Since the issue involved inservice testing on a safety-related emergency core cooling system check valve, the inspectors concluded this issue was associated with the Mitigating Systems Cornerstone. The finding was a licensee performance deficiency of very low safety significance because it: (1) was not a design or qualification deficiency; (2) did not represent an actual loss of function of a system; (3) did not represent an actual loss of function of a single train or two separate trains for greater than its Technical Specification (TS) allowed outage time; (4) did not represent an actual loss of function of one or more non TS trains of equipment designated as high safety significant; and (5) did not screen as potentially risk significant due to a seismic, flooding, or severe weather initiating event. The inspectors concluded this finding affected the cross cutting area of human performance and the cross-cutting aspect of resources (H.1). Specifically, the engineer did not meet expectations regarding the performance of covered work activities because he did not challenge his role in directing the work activity and he assisted the maintenance craftsman while the craftsman attempted to exercise the check valve. In addition, licensee management inappropriately assigned the engineer responsibility for the test activity without ensuring he was in compliance with the 10 CFR 26.205 work hour requirements.

Inspection Report# : [2014002](#) (pdf)

Significance:  Dec 31, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

RHR Pump Seal Cooler Test Was Not Adequately Implemented

The inspectors identified a finding of very low safety significance with an associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion XI, "Test Control," for the failure to demonstrate the cooling capability of the residual heat removal pump seal coolers. Specifically, on December 4, 2013, the inspectors noted examples of missed and late inspections, and examples of as-found conditions not evaluated. This finding was entered into the licensee's corrective action program, in part, to provide additional guidance in the preventive maintenance program database to ensure the Generic Letter 89-13 Program inspection requirements were implemented for these heat exchangers.

The performance deficiency was determined to be of more than minor safety significance because it was associated with the Mitigating Systems Cornerstone attribute of Equipment Performance and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of the residual heat removal pumps to respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance because it did not result in the loss of operability or functionality. Specifically, the licensee reviewed the maintenance history of the coolers and determined it provided reasonable assurance of acceptable heat transfer. The inspectors did not identify a cross-cutting aspect associated with this finding because it was confirmed to not reflect current performance due to the age of the performance deficiency.

Inspection Report# : [2013005](#) (pdf)

Significance:  Dec 31, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Acceptance Criteria for UHS Level and Temperature Did Not Consider Instrument Uncertainties

The inspectors identified a finding of very low safety significance with an associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings," for the failure to include appropriate acceptance criteria for ultimate heat sink level and temperature in surveillance procedures. Specifically, as of December 5, 2013, the inspectors identified that these acceptance criteria did not account for instrument uncertainties. This finding was entered into the licensee's corrective action program, in part, to revise the acceptance criteria included in the associated surveillance procedure to account for instrument uncertainties.

The performance deficiency was determined to be of more than minor safety significance because it was associated with the Mitigating Systems Cornerstone attribute of Equipment Performance and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of the ultimate heat sink to respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance because it did not result in the loss of operability or functionality. Specifically, a historic review did not find an example where the Technical Specification limits were exceeded when accounting for instrument uncertainties. The inspectors did not identify a cross-cutting aspect associated with this finding because it was confirmed to not reflect current performance due to the age of the performance deficiency.

Inspection Report# : [2013005](#) (pdf)

Significance:  Dec 31, 2013

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Nonconforming Materials Used in EDG Air Coolant Piping System

A finding of very low safety significance with an associated Non-Cited Violation of 10 CFR 50, Appendix B, Criterion XV, "Nonconforming Materials, Parts, or Components," was self-revealed on August 9, 2013, when operators had to manually shut down emergency diesel generator (EDG) 14 due to high air coolant system inlet temperature during a 24-hour surveillance test run. The high temperature condition occurred due to the licensee's failure to adequately control the installation of the EDG 14 air coolant system control air pipe fitting between the relief valve and pressure regulator to prevent the use of materials that did not conform to design requirements. The licensee completed repairs to the EDG 14 air coolant system and returned the EDG to an operable status. The issue was entered into the licensee's corrective action program for evaluation and additional corrective actions.

The finding was of more than minor safety significance since it was associated with the Design Control attribute and adversely affected the Mitigating Systems Cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). Specifically, the use of nonconforming materials led to failure of the EDG 14 air coolant system control air pipe fitting, which rendered the EDG inoperable. Although the finding involved an actual loss of function of a single train for greater than its Technical Specification allowed outage time, it was determined to be of very low safety significance during a detailed quantitative Significance Determination Process review since the delta core damage frequency was determined to be less than $1E-7$ /year using the NRC Standardized Plant Analysis Risk model. The inspectors concluded that because the nonconforming control air pipe fitting was installed in the EDG 14 air coolant system in 1988 and the most recent missed opportunity to correct the problem occurred in 2005 or 2006, this issue would not be reflective of current licensee performance and no cross-cutting aspect was identified.

Inspection Report# : [2013005](#) (pdf)

Significance:  Sep 06, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Battery Rack Configuration Not in Accordance with Design Basis

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the failure to assure the battery rack end rail configuration for battery 2P-29 was in accordance with the design basis Seismic Category I qualification. Specifically, the licensee failed to install battery 2P-29 with the battery rack end rails within 1/8 inch from the battery. The inspectors found one end rail gap for battery 2P-29 greater than 1/8 inch and up to approximately 1/4 inch. The licensee entered this concern into its Corrective Action Program, provided reasonable assurance the installed oversized battery rack end rail gap did not result in a loss of battery 2P-29 battery rack system functionality, completed corrective actions to install a battery rack end rail shim and readjust the battery rack end rail gap within 1/8 inch.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems cornerstone attribute of design control and affected the cornerstone objective of ensuring the capability and reliability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance (Green), because the finding was a design deficiency that did not result in a loss of battery functionality. The inspectors did not identify a cross-cutting aspect associated with this finding, because the finding was not representative of current performance.

Inspection Report# : [2013008](#) (pdf)

Significance:  Sep 06, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Design Loads for ECCS Suction Strainer Modification Not in Conformance with Design Basis Plant Unique Analysis Report

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the failure to establish measures to assure the design basis was correctly translated into specifications, drawings, procedures, and instructions. These measures shall include provisions to assure appropriate quality standards are specified and deviations from such standards are controlled. Specifically, the design loads used in a calculation that supported the emergency-core-cooling system suction strainer modification deviated from the loads used in the original design analysis of torus attached piping without providing sufficient justification the design load changes were in conformance with the original torus attached piping design and licensing basis.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems cornerstone attribute of design control and affected the cornerstone objective of ensuring the capability and reliability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance (Green) because the finding was a design deficiency that did not result in a loss of piping system functionality. The inspectors did not identify a cross-cutting aspect associated with this finding because the finding was not representative of current performance.

Inspection Report# : [2013008](#) (pdf)

Significance:  Sep 06, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Correct Non-Conservative Technical Specification

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," for the failure to ensure a non-conservative battery technical

specification (TS) was corrected in a timely manner. Specifically, the licensee failed to apply for a license amendment to correct the maximum allowed 150 micro-ohm resistance values for the battery cell-to-cell and terminal connections to an acceptable value.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems cornerstone attribute of equipment performance and affected the cornerstone objective of ensuring capability and reliability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance (Green), because the licensee provided test results that indicated the measured resistance values had never approached the TS allowed values. The inspectors determined this finding has a cross-cutting aspect in the area of human performance associated with decision making – systematic processes, because the licensee did not make safety significant or risk significant decisions using a systematic process.

Inspection Report# : [2013008](#) (*pdf*)

Significance:  Sep 06, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Untimely Resolution of Non-Conservative Battery Technical Specification

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, “Design Control,” for the failure to correctly translate the requirements of IEEE 450-1972 into TS surveillance requirements. Specifically, TS surveillance requirements (SR) 3.8.4.8 required verifying battery capacity every “18 months when the battery shows degradation or has reached 85 percent of expected life,” contrary to the requirements for annual capacity tests in the IEEE standard, which the licensee was committed to follow.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems cornerstone attribute of design control and affected the cornerstone objective of ensuring capability and reliability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance (Green), because the finding was a design deficiency that did not result in a loss of battery functionality. The inspectors did not identify a cross cutting aspect associated with this finding because the finding was not representative of current performance.

Inspection Report# : [2013008](#) (*pdf*)

Significance:  Sep 06, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Battery Testing Not in Conformance with Design Standard

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, “Design Control,” for the failure to translate the design standard requirements for battery test periodicity into procedures and instructions. Specifically, after calculations determined battery 2B-1 had an expected life of less than 20 years, the licensee failed to adjust the capacity test periodicity and 85 percent life point in accordance with the battery design standard or provide for battery replacement at 97.1 percent of rated capacity.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems cornerstone attribute of design control and affected the cornerstone objective of ensuring capability and reliability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as very low safety significance (Green), because the finding was a design deficiency that did not result in a loss of battery functionality. The inspectors identified the finding had a cross-cutting aspect in the area of problem

identification and resolution, Corrective Action Program because the licensee failed to ensure issues potentially impacting nuclear safety are promptly identified and fully evaluated such that the resolutions address causes and extent of conditions, as necessary.

Inspection Report# : [2013008](#) (*pdf*)

Barrier Integrity

Significance:  Mar 31, 2014

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to Perform Appropriate Preventive Maintenance on Reactor Building Heating, Ventilation, and Air Conditioning Components

A finding of very low safety significance with an associated non-cited violation of 10 CFR 50.65(a)(2) was self-revealed on November 24, 2013, for the licensee's failure to demonstrate that the performance of the temperature switches, steam traps, and drains of the reactor building heating, ventilation, and air conditioning (RBHVAC) system were effectively controlled through appropriate preventive maintenance or monitored as specified in 10 CFR 50.65(a)(1), such that the RBHVAC system remained capable of performing its intended function. The lack of preventive maintenance on these components for the RBHVAC system led to its failure and resulted in a loss of the safety function of secondary containment. Corrective actions included the creation of work orders to replace the remaining steam traps and reclassification of the steam traps and drains as Non-Critical in the licensee's preventive maintenance program with annual preventive maintenance activities for cleaning scheduled prior to the heating season.

The finding was of more-than-minor significance since it was associated with the Structures, Systems, and Components and Barrier Performance attribute of the Barrier Integrity Cornerstone and adversely affected the cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. Specifically, inadequate preventive maintenance of the RBHVAC system components resulted in a trip of the system. Therefore, this performance deficiency had a direct effect on the licensee's ability to maintain the safety function of secondary containment. The finding was a licensee performance deficiency of very low safety significance because it represented only a degradation of the radiological barrier function provided for the Reactor Building. This finding affected the cross-cutting area of problem identification and resolution and the cross-cutting aspect of trending (P.4). Specifically, over the past several years there were multiple trips of the RBHVAC system documented in the licensee's corrective action program from failures of temperature switches, steam traps, and drains, including an event from January 22, 2013, that also resulted in a loss of the secondary containment function. The licensee failed to analyze this information in the aggregate to identify and correct the issue.

Inspection Report# : [2014002](#) (*pdf*)

Significance:  Mar 31, 2014

Identified By: NRC

Item Type: NCV NonCited Violation

Unacceptable Preconditioning of High Pressure Coolant Injection System Air Operated Valve Prior to Stroke Time Test Measurement

The inspectors identified a finding of very low safety significance with an associated non-cited violation of 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings." The licensee failed to establish an adequate

procedure to perform required stroke time testing for high pressure coolant injection turbine supply drain pot to main condenser drain line isolation valve E4100-F028. Specifically, the surveillance test procedure resulted in unacceptable preconditioning of the valve prior to the stroke time test measurement. The licensee entered this issue into its corrective action program for evaluation and initiated a corrective action to revise the test procedure.

The finding was of more-than-minor significance since it was associated with the Procedure Quality attribute of the Barrier Integrity Cornerstone and adversely affected the cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. Because the preconditioning altered the as-found condition of the air-operated valve, the data collected through the performance of the surveillance test were not fully indicative of the true valve performance trend. Therefore, this performance deficiency had a direct effect on the licensee's ability to trend as-found data for the purpose of assessing the reliability of the valve. The finding was a licensee performance deficiency of very low safety significance because it did not involve an actual open pathway in the physical integrity of the Auxiliary Building. The inspectors concluded that because the valve testing sequence that unacceptably preconditioned E4100-F028 had existed in the surveillance test procedure for greater than three years and no opportunity reasonably existed during that time to identify and correct it, this issue would not be reflective of current licensee performance and no cross cutting aspect was identified.

Inspection Report# : [2014002](#) (*pdf*)

Significance:  Mar 31, 2014

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform Inservice Testing of High Pressure Coolant Injection and Reactor Core Isolation Cooling System Valves

The inspectors identified a finding of very low safety significance with an associated non-cited violation of 10 CFR 50.55a. The licensee failed to perform required inservice testing of high pressure coolant injection and reactor core isolation cooling turbine supply drain pot to main condenser drain line isolation valves E4100-F029, E5150-F025, and E5150-F026. The licensee entered this issue into its corrective action program for evaluation, completed an immediate operability determination, and initiated a corrective action to revise applicable test procedures to incorporate inservice testing of the valves.

The finding was of more-than-minor significance since it was associated with the Structures, Systems, and Components and Barrier Performance attribute of the Barrier Integrity Cornerstone and adversely affected the cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. Specifically, the licensee's failure to perform required inservice testing had a direct effect on its ability to trend as-found performance data for the purpose of assessing the reliability of the three isolation valves, which are required by design to isolate seismically qualified portions of the piping systems from non-seismically qualified portions. The finding was a licensee performance deficiency of very low safety significance because it did not involve an actual open pathway in the physical integrity of the Reactor and Auxiliary Buildings. The inspectors concluded that because the engineering evaluation that excluded the valves from inservice testing was completed in 1999 and no recent opportunity reasonably existed to identify and correct the error, this issue would not be reflective of current licensee performance and no cross cutting aspect was identified.

Inspection Report# : [2014002](#) (*pdf*)

Significance:  Dec 31, 2013

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to Correctly Connect Thermocouple Wiring During Maintenance Resulted in Inoperable Reactor Core Isolation Cooling System Isolation Instrumentation

A finding of very low safety significance with an associated Non-Cited Violation of Technical Specification (TS)

5.4.1.a was self-revealed on August 30, 2013, when the Division 1 Reactor Core Isolation Cooling (RCIC) Equipment Room temperature input to the associated steam line isolation logic was discovered inoperable during a scheduled surveillance test. Maintenance craftsmen had failed to correctly terminate thermocouple wiring as specified by the work instructions during maintenance to replace terminal block knife switches two weeks earlier. As a result, the Division 1 RCIC Equipment Room temperature input to the associated steam line isolation logic for RCIC steam supply primary containment outboard isolation valve 1E51-F008 was rendered inoperable for greater than the TS 3.3.6.1 completion time. The licensee promptly corrected the wiring discrepancy and restored the Division 1 RCIC system steam line isolation logic to an operable status. The issue was entered into the licensee's corrective action program for evaluation and additional corrective actions.

The finding was of more than minor safety significance since it was associated with the Human Performance attribute and adversely affected the Barrier Integrity Cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. Specifically, the Division 1 RCIC system steam line isolation logic was rendered inoperable for greater than the TS 3.3.6.1 completion time because maintenance craftsmen failed to correctly terminate thermocouple wiring as specified by the procedure when replacing terminal block knife switches. The finding was a licensee performance deficiency of very low safety significance because it only represented a degradation of the radiological barrier function provided for the Reactor Building and was not a complete loss of the barrier function provided by the RCIC system steam line isolation instrumentation since the Division 2 RCIC system steam line isolation logic remained operable. The inspectors concluded that this finding affected the cross-cutting area of human performance since adequate licensee personnel work practices did not support successful human performance. Specifically, human error prevention techniques, such as self and peer checking, were not adequately used to ensure the thermocouple wiring was correctly terminated upon replacing the terminal block knife switches (H.4(a)).

Inspection Report# : [2013005](#) (*pdf*)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Security

Although the Security Cornerstone is included in the Reactor Oversight Process assessment program, the Commission has decided that specific information related to findings and performance indicators pertaining to the Security Cornerstone will not be publicly available to ensure that security information is not provided to a possible adversary. Other than the fact that a finding or performance indicator is Green or Greater-Than-Green, security related information will not be displayed on the public web page. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

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