

D.C. Cook 2

3Q/2013 Plant Inspection Findings

Initiating Events

Significance: G Jul 26, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Evaluate Routing of Fiber Optic Cable in Combustible Exclusion Zone.

The inspectors identified a finding of very low safety significance (Green) and associated NCV of Technical Specification 5.4.1.d, "Procedures," for the failure to control combustibles in accordance with a Fire Protection Program (FPP). Specifically, the licensee failed to obtain the FPP engineering review when they routed a fiber optics cable in a combustible exclusion area which was designated to establish separation between two fire areas required per 10 CFR Part 50, Appendix R. A twenty feet separation space with no intervening combustibles was located between Fire Areas AA36 and AA42 in the Auxiliary Building at 609 foot elevation. The licensee subsequently entered the issue into their Corrective Action Program and performed a preliminary evaluation of this issue and concluded that the cable routing did not affect the requirements of the FPP.

The inspectors determined that this finding was more than minor because if left uncorrected, it would become a more significant safety concern. Specifically, the licensee's failure to perform an engineering evaluation when introducing combustibles in the combustible exclusion zone or safety-related areas could potentially affect the validity of future evaluations. The inspectors determined that the finding screened as having very-low-safety significance in Task 1.3.1 of IMC 0609, Appendix F. This finding has a cross-cutting aspect in the area of human performance, work control, because the licensee failed to coordinate the routing of the fiber optics cable through a combustible exclusion area with the Fire Protection Engineer (FPE).

Inspection Report# : [2013009](#) (*pdf*)

Significance: G Dec 31, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform an Evaluation on Essential Service Water Piping

: The inspectors identified a finding of very low safety significance and associated NCV of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures and Drawings," for the failure to perform an evaluation required to meet 10 CFR Part 50, Appendix B, Criterion XI, on essential service water piping. Specifically, the inspectors identified the licensee failed to perform a required evaluation on a segment of essential service water piping when the results of the pipe wall thickness measurements demonstrated they were below the established minimum wall thickness acceptance criteria. The licensee entered the issue into their corrective action program and based on an engineering evaluation determined the condition did not represent an operability concern and the structural integrity of the piping system was not compromised.

The performance deficiency was determined to be more than minor because if left uncorrected it had the potential to lead to a more significant safety concern. Specifically, by not performing the required evaluation there was a potential to return a system to service that could exceed the design limits prior to the next inspection. The issue impacted the initiating events cornerstone because the degraded wall thickness could lead to a loss of service water and/or internal

flooding initiating event; and the issue adversely affected the attribute of equipment reliability. The finding was screened as very low safety significance (Green) because a subsequent evaluation demonstrated that this issue did not result in the complete or partial loss of operability of the essential service water system. The inspectors determined the finding had a cross cutting aspect in the area of human performance because the licensee did not follow their procedure which required them to generate an action request and perform an evaluation when acceptance criteria were not met.

Inspection Report# : [2012005](#) (*pdf*)

Mitigating Systems

Significance:  Jul 26, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Propped Open Fire Doors Required Manual Actuation of the CO2 System to Close

The inspectors identified a finding of very low safety significance (Green) and associated NCV of the D. C. Cook Nuclear Power Plant facility operating licensee conditions for the Fire Protection Program for the licensee's failure to ensure fire doors that were propped open will automatically close at time of a fire. Specifically, Fire Doors 1-DR-AUX471 and 2-DR-AUX472 were found propped open and held by fusible links and CO2 devices. In the event of a fire in either Fire Area AA40 or Fire Area AA43, the associated door would not automatically close due to the location of the fusible link, and the CO2 pop-off devices would activate when the CO2 System is manually actuated. The licensee subsequently entered the issue into their Corrective Action Program and established fire tours of the affected fire areas.

The inspectors determined that this finding was more than minor because the failure to ensure the propped open fire doors would automatically close in the event of a fire did not ensure that the fire would not spread between the adjacent fire areas separated by the doors and could have potentially compromised the ability to safely shutdown the plant. Based on the Detailed Risk-Evaluation completed by the Region III Senior Reactor Analysts (SRA), the inspectors determined the finding was of very low safety significance (Green) because the resulting change in the Core Damage Frequency (?CDF) was less than 1E-6/yr. The finding did not have a cross-cutting aspect because it was not reflective of current performance.

Inspection Report# : [2013009](#) (*pdf*)

Significance:  Jul 26, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Assure that a Second Fire Pump would Start upon Demand at the Setpoint

The inspectors identified a finding of very low safety significance (Green) and associated NCV of the D. C. Cook Nuclear Power Plant facility operating licensee conditions for the Fire Protection Program for the licensee's failure to ensure fire doors that were propped open will automatically close at time of a fire. Specifically, Fire Doors 1-DR-AUX471 and 2-DR-AUX472 were found propped open and held by fusible links and CO2 devices. In the event of a fire in either Fire Area AA40 or Fire Area AA43, the associated door would not automatically close due to the location of the fusible link, and the CO2 pop-off devices would activate when the CO2 System is manually actuated. The licensee subsequently entered the issue into their Corrective Action Program and established fire tours of the affected fire areas.

The inspectors determined that this finding was more than minor because the failure to ensure the propped open fire doors would automatically close in the event of a fire did not ensure that the fire would not spread between the adjacent fire areas separated by the doors and could have potentially compromised the ability to safely shutdown the plant. Based on the Detailed Risk-Evaluation completed by the Region III Senior Reactor Analysts (SRA), the inspectors determined the finding was of very low safety significance (Green) because the resulting change in the Core Damage Frequency (?CDF) was less than 1E-6/yr. The finding did not have a cross-cutting aspect because it was not reflective of current performance.

Inspection Report# : [2013009](#) (*pdf*)

Significance: G Jul 26, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Incorporate Required Shutdown Guidance into Fire Response Procedure.

The inspectors identified a finding of very low safety significance (Green) and associated NCV of Technical Specification 5.4.1.d, "Procedures," for the licensee's failure to provide adequate guidance required for safe shutdown in the response procedures. Specifically, the licensee failed to provide adequate guidance to reset the associated Emergency Diesel Generator (EDG) lockout relays to support EDG operation, which were required to power safe shutdown components to achieve shutdown in the event of a fire in either Fire Zones 79 or 85 for Units 1 or 2 respectively. The licensee subsequently entered the issue into their Corrective Action Program and added steps into the fire response procedure.

The inspectors determined that this finding was more than minor because the failure to provide adequate procedural guidance to reset the EDG lockout relays could have potentially compromised the ability to safely shutdown the plant in the event of a fire. Based on the Detailed Risk Evaluation completed by the Region III SRA, the inspectors determined the finding was of very low safety significance (Green) because the resulting change in the Core Damage Frequency (?CDF) was equal to 4.17E-9/yr. The finding did not have a cross-cutting aspect because it was not reflective of current performance.

Inspection Report# : [2013009](#) (*pdf*)

Significance: G May 31, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Maintain Emergency Operating Procedures for Mitigating the Consequences of a SGTR per TS Section 5.4.1, "Procedures"

The inspectors identified a finding of very low safety significance, with two associated NCVs of Technical Specification (TS), Section 5.4.1, "Procedures," and TS 3.7.4, "Steam Generator (SG) Power Operated Relief Valves (PORVs)," for the failure to implement design measures which were consistent with the licensing bases for a Steam Generator Tube Rupture (SGTR) concurrent with a Loss of Offsite Power (LOOP) to the station. Specifically, the licensee's emergency operating procedures (EOPs) 1(2) OHP-4023-E-3, "Steam Generator Tube Rupture," failed to provide adequate actions to mitigate the consequences of a SGTR, coincident with a LOOP, in sufficient time to prevent overfilling the ruptured steam generator. Additionally, the licensee failed to declare the affected unit's SG PORVs inoperable and complete the required actions when the non-safety-related control air compressor (CAC) was made unavailable and incapable of providing its required support function. With the unit's CAC unavailable, the SG PORVs would not be capable of being remotely operated from the control room during a SGTR concurrent with the LOOP. The licensee entered this issue into their corrective action program and completed modifications to establish Nitrogen as another motive force to support SG PORV operability.

Inspection Report# : [2013010](#) (*pdf*)

Significance: G May 31, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Enter the Limiting Condition for Operations and Perform Required Actions per TS 3.7.4, “SG PORVs.”

This performance deficiency was more than minor because it was associated with the equipment performance attribute of the Mitigating Systems Cornerstone and affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. The finding was evaluated using the SDP in accordance with IMC 0609, Appendix A, “The Significance Determination Process for Findings at Power.” Based on the Detailed Risk Evaluation required, the inspectors determined the finding was of very low safety significance (Green) because the resulting change in the Core Damage Frequency (CDF) was equal to $2.4E-8$ /yr. The inspectors determined the cause of this finding involved the crosscutting area of human performance, the component of decision making, and the aspect of conservative assumptions, H.1(b) in that the licensee did not adopt a requirement to demonstrate that the proposed action is safe in order to proceed rather than a requirement that it is unsafe in order to disapprove the action. Specifically, the licensee incorrectly assumed the unaffected unit’s plant air system (not backed by the emergency diesel generators) would be available during the SGTR scenario to supply motive power to the affected unit’s SG PORVs. This assumption failed to take into account the licensing basis requirement of considering a SGTR and a loss of offsite power to the station (both units).

Inspection Report# : [2013010](#) (*pdf*)

Significance: G Mar 31, 2013

Identified By: NRC

Item Type: FIN Finding

Operability Evaluation Relied on Alternate Methods Not Demonstrated to be Technically Appropriate

The inspectors identified a finding of very low safety significance for the failure to follow operability evaluation procedural guidance. Specifically, an evaluation was conducted for past operability of the residual heat removal and containment spray systems due to the discovery of a void in the containment recirculation sump suction piping. However, the evaluation relied on computer software that has not been benchmarked to demonstrate its applicability to the type of analyses being conducted. This finding was entered into the licensee’s Corrective Action Program (CAP) to revise the affected evaluation of past operability. Reanalysis using other appropriate methods determined the piping was operable.

The performance deficiency was more than minor because it was associated with the Mitigating System Cornerstone attribute of equipment performance and adversely affected the cornerstone objective of ensuring the capability of systems that respond to initiating events to prevent undesirable consequences. In addition, the performance deficiency was more than minor because it was associated with the Barrier Integrity cornerstone attribute of structure, system, component, and barrier performance and adversely affected the cornerstone objective of providing reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. The finding screened as of very low safety significance (Green) because it was a design deficiency confirmed not to result in loss of operability. Specifically, the licensee performed an alternate operability determination which reasonably concluded the residual heat removal system was operable. In addition, it did not represent an actual open pathway in the physical integrity of reactor containment or involve an actual reduction in function of hydrogen igniters in the reactor containment. This finding did not involve enforcement action because no violation of regulatory requirements was identified. The inspectors did not identify a cross cutting aspect associated with this finding because it was not confirmed to reflect current performance.

Inspection Report# : [2013002](#) (*pdf*)

Significance: G Dec 31, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Non-conservative Condensate Storage Tank (CST) Cross-Tie NPSH Calculation

The inspectors identified a finding of very low safety significance and associated Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the failure to ensure sufficient water volume in the condensate storage tank when both units' auxiliary feedwater (AFW) pumps are aligned to a single condensate storage tank (CST.) Specifically, the licensee failed to perform a calculation to demonstrate sufficient volume and level to prevent net positive suction head and vortex issues when a single CST is providing water to all six AFW pumps as allowed by procedures. The licensee's corrective action included performing a formal calculation and increasing the available water volume in the CST when both units' AFW pumps are cross-tied.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating System Cornerstone attribute of equipment performance and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. The finding screened as of very low safety significance (Green) because it was a design deficiency confirmed not to result in loss of operability. Specifically, the licensee performed an operability determination which concluded the actual useable tank level during the previous 12 months had been sufficient. The inspectors determined the cause of this finding did not represent current licensee performance and, thus, no cross-cutting aspect was assigned.

Inspection Report# : [2012007](#) (*pdf*)

Significance: G Dec 31, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to Maintain High Energy Line Break and Fire Barriers

A self-revealed finding of very low safety significance (Green) and associated NCV occurred based on two violations of 10 CFR 50 Appendix B, Criterion V in the fourth quarter of 2012. The violations occurred due to failure of licensee personnel to secure High Energy Line Break (HELB)/Fire barrier doors following use of the door as required by procedure. These examples resulted in inoperability of Auxiliary Feedwater (AFW) pumps during the two periods when the doors were no longer in use for personnel transit. Upon discovery, the licensee restored the doors to an operable condition. The licensee entered the issues into the corrective action program (CAP).

The performance deficiency was determined to be more than minor because it is associated with the mitigating system cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences; and the issue adversely affected the attribute of equipment reliability. The inspectors determined that the finding was of very low safety significance because inoperability of the safety related auxiliary AFW system was less than allowed outage times in technical specifications. However, due to the multiple mitigative functions performed by the doors, the inspectors requested a review by the regional senior risk analyst (SRA). The doors protect the mitigating AFW systems from fire and high energy line breaks. The SRA concurred with the inspectors that the finding was of very low safety significance (Green) based on a bounding Probabilistic Risk Analysis using the Standardized Plant Analysis Risk (SPAR) model. The inspectors determined the finding had a cross cutting aspect in the area of human performance because the licensee did not use human error prevention techniques.

Inspection Report# : [2012005](#) (*pdf*)

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Security

Although the Security Cornerstone is included in the Reactor Oversight Process assessment program, the Commission has decided that specific information related to findings and performance indicators pertaining to the Security Cornerstone will not be publicly available to ensure that security information is not provided to a possible adversary. Other than the fact that a finding or performance indicator is Green or Greater-Than-Green, security related information will not be displayed on the public web page. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Significance: N/A Dec 31, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Not Reporting the Use of CPAP Devices by Licensed Operators

An NCV of 10 CFR 50.9, "Completeness and Accuracy of Information," was identified due to the submittal of inaccurate medical information for a licensed operator. The submittal to the NRC was inaccurate because it certified that the operator had been medically examined and had met all medical qualifications, when in fact, a Senior Reactor Operator (SRO) did not disclose that he had been prescribed a therapeutic device to treat sleep apnea. The licensee entered the issues into the corrective action program (CAP). The licensee's corrective actions included amending the SRO licensee to include the restriction related to use of a medical device.

The SRO was unaware that being prescribed a therapeutic device for treatment of sleep apnea in March 2010 was a condition requiring reporting. The licensee submitted medical information associated with relicensing the SRO in March 2012 that was incomplete and incorrect for the SRO. Because violations of 10 CFR 50.9 are considered to be violations that potentially impede or impact the regulatory process, they are dispositioned using the traditional enforcement process. In accordance with the Enforcement Policy, the inspectors concluded that the violation was a Severity Level IV because the SRO met ANSI/ANS 3.4 criteria but failed to report a condition that required an amended license. The licensee's failure to provide complete and accurate information to the NRC impacted the regulatory process because it resulted in an incorrect licensing action and is a performance deficiency. This is a minor

ROP issue since the non disclosure of a medical condition for a licensed operator did not result in an adverse impact on plant operation. Since there is no ROP Finding, there is no cross-cutting aspect associated with this violation.

Inspection Report# : [2012005](#) (*pdf*)

Last modified : December 03, 2013