

Watts Bar 1

1Q/2013 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance: G Mar 31, 2013

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to correctly implement procedure for pressurizer pressure channel calibration

A self-revealing (Green) NCV of 10 CFR Part 50, Appendix B, Criterion V, Instructions, Procedures, and Drawings, was identified for failure to properly follow plant procedures to perform a surveillance test for calibration of pressurizer pressure channel II. Specifically, the instrument maintenance surveillance test procedure 1-SI-68-6, 18 Month Channel Calibration Pressurizer Pressure Channel II, Loop 1-LPP-68-334 (P-456), Revision 10, was not adequately followed by maintenance personnel during the calibration test. The licensee's failure to correctly implement procedure 1-SI-68-6, 18 Month Channel Calibration Pressurizer Pressure Channel II, Loop 1-LPP-68-334 (P-456), to configure the test equipment in the correct configuration was a performance deficiency. The finding was more than minor because it was associated with the Initiating Events Cornerstone attribute of Human Performance and adversely affected the cornerstone objective of limiting the likelihood of events that upset plant stability and challenge critical safety functions during shutdown or power operations. Specifically, the failure to adequately implement SI 1-SI-68-6, 18 Month Channel Calibration Pressurizer Pressure Channel II, Loop 1-LPP-68-334 (P-456), resulted in a reactor trip. The inspectors screened this finding in accordance with IMC 0609, Significance Determination Process, Attachment 4, Phase 1 – Initial Screening and Characterization of Findings, and determined the finding was of very low safety significance (Green) since it did not result in the loss of mitigation equipment relied upon to transition the plant from the onset of the trip to a stable shutdown condition. The finding directly involved the cross-cutting area of Human Performance, work practices component under the aspect related to proceeding in the face of uncertainty. (H.4 (a))

Inspection Report# : [2013002](#) (*pdf*)

Significance: W Jan 18, 2013

Identified By: NRC

Item Type: VIO Violation

Inadequate abnormal condition procedure for flood mitigation strategy prior to installation of HESCO barriers

The inspectors identified a Violation (VIO) of Technical Specification (TS) 5.7.1, Procedures, for the licensee's failure to establish an adequate procedure for mitigation of external events, specifically flooding prior to the installation of HESCO barriers in 2009. The inspectors determined that the licensee's failure to comply with TS 5.7.1, "Procedures," was a performance deficiency. Specifically, AOI-7.1, Maximum Probable Flood, was not adequate to prevent the loss of critical safety functions (e.g., emergency power) during a PMF event prior to the installation of the HESCO barriers and other compensatory measures. This procedure, in part, is used to maintain the established license basis for compliance with 10 CFR 50, Appendix A, General Design Criteria 2, "Design Bases for Protection Against Natural Phenomena." Failure to establish adequate procedures for flood mitigation results in a failure to maintain

adequate protection against natural phenomena in accordance with the licensing basis of the plant.

This performance deficiency was considered more than minor because it was associated with the protection against external factors attribute of the Reactor Safety/ Mitigating Systems Cornerstone and adversely affected the cornerstone objective of ensuring availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). Specifically, prior to the installation of the HESCO barriers and other compensatory measures, Abnormal Conditions procedure, AOI 7.1, "Maximum Probable Flood," was not adequate to prevent the loss of Emergency Power during a PMF event. Loss of emergency power would lead to core and/or spent fuel pool inventory damage. The combination of the event frequencies and types of rainfall events which would over-top earthen dams leading to the loss of emergency power resulting in core damage has an impact of low to moderate safety significance. The NRC concluded that the significance of the finding is preliminarily of low to moderate safety significance (White). The inspectors determined that no cross-cutting aspect was applicable.

Inspection Report# : [2012009](#) (*pdf*)

Inspection Report# : [2013009](#) (*pdf*)

Significance: N/A Jan 18, 2013

Identified By: NRC

Item Type: VIO Violation

Failure to report unanalyzed condition related to external flooding

The inspectors identified a VIO of 10 CFR 50.72(b)(3)(ii)(B), "Immediate Notification Requirements for Operating Nuclear Reactors," for failure to report within eight hours an unanalyzed condition that significantly degraded plant safety. Specifically, the licensee failed to notify the NRC upon discovery that a postulated PMF would result in the overtopping of earthen dams not previously assumed in the plant design. The failure to report this unanalyzed condition resulted in the NRC not being made aware of a condition which would have resulted in additional NRC review. Specifically, the failure to notify the NRC within eight hours of discovery of an unanalyzed condition that significantly degraded plant safety and resulted in an unacceptable change to the facility or procedures. The inspectors determined an evaluation for cross-cutting aspect was not applicable because this is a traditional enforcement violation.

Inspection Report# : [2012009](#) (*pdf*)

Inspection Report# : [2013009](#) (*pdf*)

Significance: Y Jan 18, 2013

Identified By: NRC

Item Type: VIO Violation

Failure to maintain an adequate abnormal condition procedure to implement the flood mitigation strategy

The inspectors identified an violation (VIO) of Technical Specification 5.7.1, Procedures, for the licensee's inability to demonstrate that the required Stage I and Stage II activities could be performed within 27 hours as required by AOI-7.1, Maximum Probable Flood. The licensee's failure to adequately demonstrate the ability to realign plant systems into their flood mode configuration using AOI-7.1, Maximum Probable Flood, within the time frame required by TRM 3.7.2 and Watts Bar UFSAR Section 2.4, which could directly lead to the inability to remove decay heat from the reactor core resulting in core damage, was a performance deficiency. This performance deficiency was considered more than minor because it was associated with the Protection Against External Factors attribute of the Reactor Safety/ Mitigating Systems Cornerstone and adversely affected the cornerstone objective of ensuring availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). Specifically, the inability to realign plant systems into their flood mode configuration within the required time frame could directly lead to the inability to remove decay heat. The combination of the seismic and rainfall event frequencies and types of rainfall events which would lead to flooding above site grade and the inability to realign plant systems into their flood mode configuration within the 27-hour required time frame could directly lead to the inability to remove decay heat from the reactor core resulting in core damage which has an impact of substantial

safety significance. The NRC concluded that the significance of the finding is of substantial safety significance (Yellow).

The cause of the finding had a cross-cutting component of Resources in the area of Human Performance with an aspect of ensuring that personnel, equipment, procedures, and other resources were available and adequate to assure nuclear safety. Specifically, inadequacies in those procedures, equipment, and personnel training necessary to realign plant systems within the required time frame to cope with all anticipated external flooding events. (H.2 (d)) (Section 1R01.3)

Inspection Report# : [2012009](#) (*pdf*)

Inspection Report# : [2013009](#) (*pdf*)

Significance: G Jan 18, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adequately protect safety-related equipment during flood mode preparation

The inspectors identified a non-cited violation (NCV) of 10 CFR 50, Appendix B, Criterion III, “Design Control,” for the licensee’s failure to adequately protect safety-related equipment during flood mode preparation. The licensee’s failure to adequately protect safety-related equipment during flood mode preparation as implemented by AOI-7.1, Maximum Probable Flood, was a performance deficiency. This performance deficiency was considered more than minor because it was associated with the Protection Against External Factors attribute of the Reactor Safety/Mitigating Systems Cornerstone and adversely affected the cornerstone objective of ensuring availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). Specifically, the TACF was incapable of preventing water damage, during a PMF event, to both trains of important equipment, specifically the thermal barrier booster pumps (TBBPs), necessary for coping with the PMF impact on Unit 1. Without the TBBPs and with reactor coolant pump (RCP) seal injection lost, there is no engineering assurance that RCP seal damage would not occur, leading to an RCP seal loss of coolant accident (LOCA). The licensee concluded that a potential loss of RCP seal cooling during Mode 4 conditions with the RCP pumps stopped would be bounded by the probability of a small break loss of coolant accident (1.5E-3 per NUREG/CR-6928, “Industry-Average Performance for Components and Initiating Events at U.S. Commercial Nuclear Power Plants”). The rationale for the lower RCP seal failure probability was based on: 1) lower RCS pressure affecting seal ring extrusion, 2) lower RCS temperature affecting popping seal failure, and 3) lower RCS temperature affecting seal ring binding. This technical issue did not reduce the risk of the AOI-7.01 issue (AV 05000390/2012009-03) because under a hypothetical sustained loss of core cooling, core damage ultimately would occur regardless of whether the RCP seals failed or not. Following the Regulatory Conference, the analysts reviewed the vendor information and agreed with TVA’s RCP seal failure analysis insights that the original probabilities of RCP seal integrity loss in the NRC risk analysis could be assumed as conservative. As a result, the staff determined that it was appropriate to apply an order of magnitude reduction to the RCP seal failure probability used in the original Phase 3 analysis, and the finding is appropriately characterized as Green.

This finding has a cross-cutting aspect in the Work Practices component of the Human Performance area because it was directly related to the licensee not ensuring adequate supervisory and management oversight of engineering design work activities associated with a plant design change to protect the TBBPs during certain flood events. (H.4 (c)). (Section 1R01.4)

Inspection Report# : [2012009](#) (*pdf*)

Inspection Report# : [2013009](#) (*pdf*)

Significance: G Jan 18, 2013

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to correct conditions adverse to quality related to IPS CKV-040-0604 and IPS 3A and 3B sump pumps

The inspectors identified two examples of an NCV of the 10 CFR 50 Criteria XVI, "Corrective Action," for failure to correct conditions adverse to quality for the intake pumping station (IPS) CKV-040-0604, pump 3B, discharge check valve which resulted in it being non-functional for an extended period of time, and both the IPS 3A and 3B sump pumps, which resulted in the pumps remaining in a degraded condition for an extended period of time. The licensee's failure to maintain these components in accordance with the requirements of the augmented in-service testing program and WB-DC-40-29, Flood Protection Provisions, were performance deficiencies. The performance deficiencies were determined to be more than minor because, if left uncorrected, would lead to a more significant safety concern. Specifically, internal flooding of the IPS mechanical equipment room housing the train A essential raw cooling water (ERCW) strainers could occur. The inspectors performed a Phase 1 evaluation per IMC 0609, Attachment 4, and determined that the finding was potentially risk significant because it involved the degradation of equipment specifically designed to mitigate a seismic, flooding, or severe weather initiating event (e.g., seismic snubbers, flooding barriers, tornado doors). Consequently a Phase 3 analysis was performed by a Senior Reactor Analyst. The analyst determined the finding was of very low safety significance, Green. These findings directly involved the cross-cutting area of Human Performance under the Work Practices component, in that, the licensee failed to provide adequate supervisory and management oversight to ensure corrective actions were taken to maintain the functionality of IPS equipment for extended periods of time. (H.4 (c)) (Section 1R01.5)

Inspection Report# : [2012009](#) (*pdf*)

G

Significance: Dec 31, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adequately develop and implement ice condenser ice basket repairs

A NCV of 10 CFR 50 Appendix B, Criterion III, Design Control, for the licensee's failure to adequately develop and implement ice condenser ice basket repairs in accordance with approved engineering and maintenance documents.

Specifically, the inspectors observed that repairs to six

damaged ice condenser ice baskets, previously signed off as complete in the work order (WO) by the installers and following Quality Control inspection and acceptance were not in accordance with the design and maintenance WO documents. The licensee initiated Problem Evaluation Reports (PERs) 623040 and 626983 to address the inspector-identified deficiencies.

The licensee's failure to adequately develop and implement ice condenser ice basket repairs in accordance with approved engineering and maintenance documents was a performance deficiency. The inspectors reviewed Inspection Manual Chapter (IMC) 0612 and determined that the finding was more than minor because the deficiencies were not identified by the licensee and would have remained unidentified at least for the duration of the upcoming fuel cycle. Without the specified repairs being properly implemented on the damaged ice baskets, there was no reasonable assurance they were capable of performing their design function, and there was also potential for damage to adjacent ice baskets obstructing open flow paths, in the event the ice condenser was required to perform its design function. Using the Initial Characterization of Findings guidance of IMC 0609, the inspectors determined that the finding was of very low safety significance (Green) because no actual loss of safety function occurred. The cause of the finding had a cross-cutting aspect in the area of effective supervisory/management oversight in the Work Practices component. It was directly related to the licensee not ensuring adequate supervisory and management oversight of work activities, including the licensee engineering personnel that prepared and reviewed the ECP, the contractors that performed the repair work and the Quality Control personnel that performed the repair inspection and acceptance. (H.4 (c)). (See Section 1R18)

Inspection Report# : [2012005](#) (*pdf*)

Significance:  Dec 31, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to follow procedure resulted in failing to remove jumpers inhibiting proper operation of the steam generator blowdown system

A self-revealing NCV of Technical Specifications (TS) 5.7.1, Procedures, was identified for failing to adhere to OPDP-1, Conduct of Operations, Section 5.1, Procedure Adherence. The licensee failed to ensure a jumper was removed prior to placing the steam generator blowdown system into service per System Operating Instruction 90.01, Rev. 29, Liquid Process Radiation Monitors, step 5.5 [10]. This was a performance deficiency and a finding. The finding was more than minor because, if left uncorrected, it could lead to a more significant safety issue, a radioactive release, and was associated with the Mitigating Systems Cornerstone attribute of equipment performance (reliability) and adversely affected the cornerstone objective. The finding was evaluated using the SDP Phase I and was determined to be a finding of very low safety significance because actual high contamination levels did not occur within the steam generators during the period that the jumper was installed. The licensee entered this issue into the corrective action program as PER 637279. The finding directly involved the cross-cutting area of Human Performance under the procedural compliance aspect of the work practices component; in that the procedural requirements of System Operating Instruction 90.01 were not met. (H.4(b)) (Section 40A3)

Inspection Report# : [2012005](#) (*pdf*)

Significance:  Sep 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to follow scaffold procedure threatens ERCW pump operability

The inspectors identified a NCV of Technical Specification 5.7.1, Procedures, for the licensee's failure to properly implement Maintenance Procedure MMTP-102, Erection of Scaffolds/Temporary Work Platforms and Ladders, Revision 7. Specifically, a temporary scaffold erected in close proximity to an essential raw cooling water (ERCW) pump was not adequately restrained to prevent interaction with the pump motor during a seismic event. The licensee entered the issue into the corrective action program as Problem Evaluation Report (PER) 588895, removed the subject scaffold, and implemented corrective actions to inspect all scaffolding in Seismic Category I areas for similar conditions.

The licensee's failure to erect the scaffold in accordance with procedures in the vicinity of safety-related equipment was a performance deficiency. The inspectors reviewed IMC 0612 and determined that the finding was more than minor because, if left uncorrected, scaffold interaction with the pump motor during a seismic event could render the pump inoperable. The finding was associated with the Mitigating Systems Cornerstone. Using the Phase I screening worksheet of IMC 0609, the inspectors determined that the finding was of very low safety significance (Green) because no actual loss of safety function occurred and the finding did not screen as potentially risk significant due to external events. The cause of the finding had a cross-cutting aspect in the area of effective supervisory/management oversight in the Work Practices component. It was directly related to the licensee not ensuring adequate supervisory and management oversight of work activities, including contractors that erected the scaffold and licensee engineering personnel that reviewed and approved the deficient scaffold installation that could adversely affect nuclear safety. (H.4 (c)). (See Section 1R01).

Inspection Report# : [2012004](#) (*pdf*)

Significance:  Sep 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate corrective actions for the C ERCW pump breaker

The inspectors identified a NCV of 10 Code of Federal Regulations (CFR) 50, Appendix B, Criterion XVI, “Corrective Action,” for the licensee’s failure to correct an identified deficiency in the C-A ERCW pump breaker on July 25, 2012. This uncorrected deficiency led to the inability of the breaker to trip and is a performance deficiency. The inspectors reviewed IMC 0612 and determined that the finding was more than minor because, if left uncorrected, it would have the potential to lead to a more significant safety concern; specifically the failure of the C-A ERCW pump to load shed on a loss of offsite power. Additionally, the finding was associated with the equipment performance attribute of the Mitigating Systems Cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences (i.e., core damage). Using the Phase I screening worksheet of IMC 0609, the inspectors determined that the finding was of very low safety significance (Green) because the associated shutdown board is a Unit 2 board and is lightly loaded. Additionally, the failure of the C-A ERCW pump breaker to trip and thus be immediately loaded onto 2A emergency diesel generator is within the transient capability of the emergency diesel generator. The cause of the finding was directly related to the cross-cutting aspect for appropriate corrective actions to address safety issues in a timely manner commensurate with their safety significance and complexity in the corrective action program component of the cross-cutting area of Problem Identification and Resolution, in that the licensee failed to take adequate corrective actions to repair the C-A ERCW breaker when the initial deficiency was discovered on July 25, 2012. (P.1(d)). (See Section 40A2)

Inspection Report# : [2012004](#) (*pdf*)

Significance: G Jun 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to maintain steam generator blowdown isolation valves in the environmental qualification program

The team identified a Green non-cited violation (NCV) of 10 Code of Federal Regulations (CFR) Part 50, Appendix B, Criterion III, “Design Control”, for the licensee’s failure to maintain steam generator blowdown (SGBD) isolation valves 1-FCV-1-181, 182, 183, and 184 in the environmental qualification (EQ) program. Removing the valves from the EQ database resulted in internal components (lower bottom gasket and reed switch) in the SGBD valves exceeding their qualified life and replacement intervals as stated in the licensee’s existing EQ and revised EQ calculations. The licensee entered this issue into their corrective action program as problem evaluation report (PER) 495239 and service request (SR) 562298, and performed additional analyses and evaluations to provide reasonable assurance of operability of components.

The team determined that the failure to maintain SGBD isolation valves 1-FCV-1-181, 182, 183, and 184 in the EQ program, which resulted in two subcomponents in these valves exceeding their qualified life and replacement interval, is a performance deficiency. In addition, the licensee failed to perform an adequate functional evaluation to confirm operability of these valves after the NRC identified that the reed switch was not included in the original functional evaluation. The revised EQ calculation performed by the licensee to address the lower bottom gasket indicated the reed switch had exceeded its qualified life of 13.5 years; however, this was not addressed in the licensee’s functional evaluation until identified by the NRC. This performance deficiency was more than minor because it affected the Mitigating System Cornerstone attribute of design control to ensure the availability, reliability, and capability of safety systems that respond to initiating events to prevent undesirable consequences. In addition, this performance deficiency also closely parallels Inspection Manual Chapter 0612, Appendix E, example 3.j because the error resulted in a condition where there was a reasonable doubt of the operability of safety related components as a result of the revised EQ calculation. The team screened this finding in accordance with NRC IMC 0609, “Initial Screening and Characterization of Findings,” Attachment 4, Phase 1, and determined the finding was of very low safety significance (Green). The team determined that no cross-cutting aspect was applicable because this finding was not indicative of current licensee performance. (Section 1R17)

Inspection Report# : [2012003](#) (*pdf*)

Significance:  Jun 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to identify degraded auxiliary charging pump and initiate corrective actions

The inspectors identified a Green NCV of 10CFR50 Appendix B Criterion XVI for failure to identify that the 1A auxiliary charging pump (ACP) was degraded based on previous questionable testing results. The inspectors determined that no acceptable testing had been performed which verified the functionality of 1A and 1B ACP until March 23, 2012. During subsequent testing, only the 1B ACP met its acceptance criteria. This system relies on the capability of these pumps to support Technical Requirement 3.7.2, Flood Mode Protection Plan.

The performance deficiency was determined to be more than minor because it was associated with the Mitigating Systems Cornerstone attribute of Equipment Performance, and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Specifically, the 1A ACP was unable to perform its function in the event of severe external flooding for > 1 year. The inspectors performed a Phase 1 evaluation per Inspection Manual Chapter 0609, Attachment 4 and determined that the finding was potentially risk significant due the degradation of equipment specifically designed to mitigate external events (e.g., flooding mitigation). Consequently a Phase 3 analysis was performed by a Senior Reactor Analyst. The analyst determined that the risk significance of the issue was very low (i.e., $P_{CDF} < 1.0E-7$). The dominant sequence was a significant flooding event which would require the licensee to implement their Flood Mode Mitigation strategy, with the subsequent failure of a single train of ACP pumps for the system. The finding directly involved the cross-cutting area of human performance under the supervisory and management oversight of work activities component, in that, the failures of the ACPs were left unresolved for an extended period of time over a number of failed tests. (H.4(c)). (Section 40A2)

Inspection Report# : [2012003](#) (*pdf*)

Significance:  May 15, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Establish Test Procedures to Assure Satisfactory auxiliary control air subsystem (ACAS) Performance during Design Basis Accidents

The team identified a non-cited violation of 10 CFR Part 50, Appendix B, Criterion XI, "Test Control," for failure to perform capacity (volumetric flow) testing on the safety-related auxiliary control air subsystem (ACAS). The licensee had documented that, for worst case environmental conditions, the air compressor capacity had little margin when compared to required air demand, even for single unit operation. This issue was entered into the licensee's corrective action program as problem evaluation report 501941 for further evaluation of corrective actions.

Inspection Report# : [2012008](#) (*pdf*)

Significance:  May 15, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Adequately Test the AFW Discharge Check Valves

The team identified a non-cited violation of 10 CFR Part 50, Appendix B, Criterion XI, "Test Control," for the licensee's failure to establish a test program that demonstrated the adequacy of the auxiliary feedwater (AFW) discharge check valves. Specifically, the licensee failed to develop a test program that would provide assurance that back leakage through the AFW discharge check valves would not prevent the system from providing design flowrates to the steam generators. This issue was entered into the licensee's corrective action program as problem evaluation report 499950. The licensee performed a functional evaluation and determined that the AFW system was operable

based on the pumps not currently being degraded to the design limits, and the existence of additional conservatisms in the licensee's design basis hydraulic analysis. (Green NCV).

Inspection Report# : [2012008](#) (pdf)

Significance:  May 15, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Acceptance Criteria in Maintenance and Surveillance Procedures (5 Examples)

The team identified five examples of a non-cited violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the licensee's failure to correctly translate vendor specifications and design calculations into maintenance and surveillance procedures. The five examples were entered into the licensee's corrective action program. (Green NCV)

Inspection Report# : [2012008](#) (pdf)

Barrier Integrity

Emergency Preparedness

Significance:  Dec 31, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Late state notification of unusual event

A self-revealing non-cited violation (NCV) of 10 Code of Federal Regulations (CFR) 50.54q(2) for failure to follow the approved emergency plan. Specifically, on August 10, 2012, state officials were not notified within 15 minutes of the declaration of an Unusual Event. State notification is a risk-significant planning standard requirement required by 10 CFR 50.47(b)(5), 10 CFR 50 Appendix E, Section IV.D.3 and Section 5.2.1, of the licensee's Radiological Emergency Plan.

The issue was greater than minor because it was associated with the Emergency Planning cornerstone attribute of Emergency Response Organization performance during an actual event. The finding affected the cornerstone objective in that timely notification is critical to ensuring that the licensee is capable of implementing adequate measures to protect the health and safety of the public in the event of a radiological emergency. The inspectors reviewed this finding using IMC 0609, Appendix B, Emergency Preparedness Significance Determination Process, Attachment 1, Failure to Implement (Actual Event) Significance Logic. The finding was determined to be of very low safety significance because it was a failure to implement during an Unusual Event. The finding had a cross-cutting aspect in the area of Human Performance, Decision-Making, because the unit supervisor, in the absence of the shift manager, did not effectively fulfill his responsibility to direct or perform required state communications within the required 15 minute time period as required by the Radiological Emergency Plan. (H.1(a). (Section 1EP5)

Inspection Report# : [2012005](#) (pdf)

Occupational Radiation Safety

Public Radiation Safety

Security

Although the Security Cornerstone is included in the Reactor Oversight Process assessment program, the Commission has decided that specific information related to findings and performance indicators pertaining to the Security Cornerstone will not be publicly available to ensure that security information is not provided to a possible adversary. Other than the fact that a finding or performance indicator is Green or Greater-Than-Green, security related information will not be displayed on the public web page. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

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