

Hope Creek 1 3Q/2012 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance:  Jun 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Average Power Range Monitor Flow Unit Summers out of Tech Spec Tolerance

The inspectors identified an NCV of very low safety significance of TSs 3.3.1 and 6.8.1 because PSEG's written procedure (HC.IC-CC.SE-0032) was not adequately established and implemented for performing the weekly channel test and calibration of the flow biased APRMs that input into the simulated thermal power upscale RPS trip. Specifically, the procedure provided inadequate instructions for calculating total reactor recirculation drive flow while in single loop operation (SLO). PSEG's corrective actions included revision of the appropriate procedures and development of a schedule template (including required surveillances) for entry into and return from SLO. The violation was entered into the CAP as notification 20549760.

The performance deficiency was more than minor because it is associated with the procedure quality attribute of the Mitigating Systems and affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Specifically, incorrect calibration of the APRM flow units resulted in the APRM flow biased setpoint being non-conservative and exceeding the associated TS limiting safety system setpoint (LSSS) allowable value for a period of time that was considered a condition prohibited by TS. The inspectors performed a Phase I screening of the finding using IMC 0609, Attachment 0609.04, Table 4a, Mitigating Systems cornerstone, and determined the issue was of very low safety significance (Green) because the finding was not a design or qualification deficiency, did not result in an actual loss of safety function, and was not potentially risk significant for external events. The finding had a cross-cutting aspect in the area of human performance, resources component, because PSEG did not ensure that a TS-required RPS calibration procedure was complete, accurate, and adequate to assure nuclear safety. Specifically, the formula provided in the APRM flow unit summer procedure that calculated the drive flow was incorrect. The formula provided in the procedure was for dual loop operation, not for SLO.

Inspection Report# : [2012003](#) (*pdf*)

Barrier Integrity

Significance:  Jun 30, 2012

Identified By: NRC

Item Type: NCV NonCited Violation

Preconditioning of the Reactor Building to Torus Vacuum Relief Valves

The inspectors identified a NCV of very low safety significance of 10 CFR 50, Appendix B, Criterion XI, “Test Control,” because PSEG conducted unacceptable preconditioning of the reactor building to torus vacuum relief valve. Specifically, PSEG’s surveillance test procedure for these valves cycled the valve (H1GS-1GSPSV-5032) prior to recording the as-found opening setpoint required to meet Technical Specification (TS) Surveillance Requirement (SR) 4.6.4.2.b.2.a. PSEG’s immediate corrective actions included revising the surveillance test procedure to record the as-found setpoint before cycling the valve manually. The violation was entered into the CAP as notification 20554080.

The performance deficiency was more than minor because it was associated with the procedure quality attribute of the Barrier Integrity Cornerstone and affected the cornerstone objective of providing reasonable assurance that physical design barriers (containment) protect the public from radionuclide releases caused by accidents or events. Specifically, preconditioning of the reactor building to torus vacuum relief opening setpoint could mask its actual as-found condition and result in an inability to verify its operability and potentially make it difficult to determine whether the vacuum breaker would perform its intended safety function during an event. The inspectors evaluated the finding using IMC 0609, Attachment 4, “Initial Screening and Characterization of Findings,” and determined the finding was of very low safety significance (Green) because it was not a degradation of the radiological barrier function provided for the control room, auxiliary building, spent fuel pool, or standby gas treatment system, did not represent a degradation of the barrier function of the control room against smoke or toxic atmosphere, did not represent an actual open pathway in the physical integrity of reactor containment and heat removal components, and did not involve an actual reduction in function of hydrogen igniters in the reactor containment. The finding had a cross-cutting aspect in the area of problem identification and resolution, corrective action component, because PSEG did not thoroughly evaluate a prior problem such that the problem resolution addressed the extent of condition. Specifically, PSEG’s extent of condition for notification 20370021, Potential Preconditioning BJHV-F004, did not go beyond operations’ procedures and review maintenance procedures for unacceptable preconditioning. Therefore, PSEG did not identify the unacceptable preconditioning of the reactor building to torus vacuum relief valve opening setpoint because the surveillance test was in a maintenance procedure.

Inspection Report# : [2012003](#) (*pdf*)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Security

Although the Security Cornerstone is included in the Reactor Oversight Process assessment program, the Commission has decided that specific information related to findings and performance indicators pertaining to the Security Cornerstone will not be publicly available to ensure that security information is not provided to a possible adversary. Other than the fact that a finding or performance indicator is Green or Greater-Than-Green, security related

information will not be displayed on the public web page. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

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