

Nine Mile Point 2

2Q/2012 Plant Inspection Findings

Initiating Events

Significance:  Jun 30, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Inadequate Corrective Action for Instrument Fill and Vent Activities

A self-revealing NCV of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," was identified for NMPNS' failure to effectively implement corrective actions. Specifically, NMPNS's corrective actions from a differential fill and vent evolution which caused a reactor scram in January 2010 were ineffective in preventing an inadvertent start of the Division I EDG on a Level 1 ECCS initiation (reactor pressure vessel low-low-low level) signal on April 26, 2012, during a similar fill and vent evolution. NMPNS entered this issue into its corrective action program as CR-2012-003778.

The finding is more than minor because it adversely impacted the human performance attribute of the Initiating Events Cornerstone to limit the likelihood of those events that upset plant stability and challenge critical safety functions during shutdown as well as power operations. The finding was determined to be of very low significance (Green) because of the availability of safety systems and procedures pertaining to core heat removal, inventory control, electrical power, and secondary containment. The finding has a cross-cutting aspect in the area of problem identification and resolution, corrective action program, because Nine Mile Point Nuclear Station did not take appropriate corrective actions to address safety issues commensurate with their safety significance and complexity. Specifically, the corrective actions taken as a result of the January 7, 2010 event were ineffective in preventing the April 26, 2012 event. [P.1(d)] (Section 1R19)

Inspection Report# : [2012003](#) (*pdf*)

Significance:  Dec 31, 2011

Identified By: NRC

Item Type: FIN Finding

Failure to Meet Fleet Standards for Preventive Maintenance Templates

The inspectors identified a Green finding for the failure of NMPNS to meet the fleet standard for establishing and implementing preventive maintenance (PM) templates. Specifically, in 2009, NMPNS failed to implement PM templates for critical non-safety related molded case circuit breakers in accordance with the guidance in the new fleet standard. NMPNS entered this issue into their corrective action program as CR-2011-011000 and CR-2011-011045 to evaluate corrective actions needed to address this issue.

The inspectors determined that the finding was more than minor because if left uncorrected, the performance deficiency would have the potential to lead to a more significant safety concern. Specifically, continued failure to perform the "clean and inspect" PM on critical NSR MCCBs could lead to a failure that could cause a plant transient. The inspectors determined that the finding was of very low safety significance (Green) since the finding did not contribute to both the likelihood of a reactor trip and the likelihood that mitigation equipment or functions will not be available. This finding had a cross-cutting aspect in the human performance area, work practices component, in that NMPNS did not implement procedures for conducting preventive maintenance on electrical breakers [H.4.(b)].

Inspection Report# : [2011005](#) (*pdf*)

Significance:  Dec 31, 2011

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to Follow Valve Packing Procedure

A Green self-revealing NCV of technical specification (TS) 5.4.1, "Procedures," was identified for NMPNS' failure to properly implement S-MMP-GEN-201, "Site Valve Packing Procedure," Revision 00600 when maintenance personnel repacked recirculation pump discharge isolation valve 2RCS*MOV18A in August 2011. As a result, on December 9, 2011, the packing for valve 2RCS*MOV18A failed and unidentified reactor coolant system (RCS) leakage increased above the TS limit of a 2 gpm increase per 24 hours forcing a plant shutdown. NMPNS' immediate corrective actions were to repair the valve stem and install a live loaded packing system on the recirculation discharge isolation valves.

This finding is more than minor because it reasonably could be viewed as a precursor to a more significant event and adversely impacted the Initiating Events Cornerstone objective of limiting the likelihood of those events that upset plant stability and challenge critical safety functions during power operations. This finding challenged the availability and reliability of a mechanical RCS pressure boundary. This finding was evaluated using IMC 0609, "Significance Determination Process," Attachment 04, "Phase 1-Initial Screening and Characterization of Findings," Table 4a, and determined to require further evaluation because the as-found leakage exceeded a TS RCS leakage limit. Based on Region I Senior Reactor Analyst (SRA) review, the finding was determined to be of very low safety significance (Green) since the maximum possible leak rate through the valve packing would be compensated by normal operation of the control rod drive system and the condensate/feedwater system. This finding has a cross-cutting aspect in the area of human performance, work control because NMPNS did not define and effectively communicate expectations regarding procedural compliance and personnel did not follow procedures during their inspection of the valve stem [H.4.(b)].

Inspection Report# : [2011005](#) (pdf)

Significance:  Sep 30, 2011

Identified By: Self-Revealing

Item Type: FIN Finding

Inadequate Actions to Prevent Vibration Induced Failure on a Socket Weld for a Vent Line on the 'A' FWP Minimum Flow Line

A Green self revealing finding was identified for inadequate implementation of corrective actions regarding vibration induced failures of socket welds. This finding resulted in an August 11, 2011, Nine Mile Unit 2 scram due to a failed socket weld on the vent line for the 'A' feedwater pump (FWP) minimum flow line. NMPNS did not properly consider the impact of high vibration levels on a vent line attached to the 'A' FWP mini-flow recirculation line. NMPNS corrective actions included upgrading the socket weld to the requirements outlined in industry operating experience (OE).

The inspectors determined that the finding was of very low safety significance (Green) through performance of a Phase 1 SDP in accordance with IMC 0609.04, Table 4a, "Characterization Worksheet for Initiating Events, Mitigating Systems (MS) and Barrier Integrity Cornerstones." Specifically, the finding did not contribute to both the likelihood of a reactor trip and the likelihood that mitigation equipment or functions will not be available. This finding has a cross-cutting aspect in the area of problem identification and resolution in that NMPNS did not implement and institutionalize OE through changes to station processes, procedures, equipment and training programs. Specifically in 1998 and again in 2010, NMPNS did not institutionalize external and internal OE to reduce the probability of a socket weld failure.

Inspection Report# : [2011004](#) (pdf)

Mitigating Systems

Significance:  Mar 30, 2010

Identified By: NRC

Item Type: FIN Finding

Inadequate Maintenance Procedure Results in Loss of Loads for Non-Vital UPS

A self-revealing finding of very low safety significance was identified for inadequate coordination during concurrent execution of a maintenance procedure and an operating procedure, which resulted in a loss of power to the loads supplied by Unit 2 uninterruptible power supply (UPS) 2VBB-UPS1A. The loss of operational capabilities, and alarm and display functions, complicated normal plant operations and impacted an "anticipated transient without scram" (ATWS) mitigation strategy. As immediate corrective action, maintenance on UPS1A was stopped pending causal evaluation of the event. The issue was entered into the corrective action program (CAP) as condition report (CR) 2009-8928.

The finding was more than minor because it was associated with the procedure quality attribute of the Mitigating Systems cornerstone and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Additionally, the finding was significant because it would have impacted Nine Mile Point Nuclear Station's (NMPNS's) ability to execute emergency operating procedure N2-EOP-C5, "Failure to Scram," in that the reactor manual control system was not available for use in accordance with N2-EOP-6, Attachment 14, "Alternate Control Rod Insertions." The finding was of very low safety significance because it was not a design or qualification deficiency, did not represent a loss of a system/train safety function, and did not screen as potentially risk significant due to external events. The finding had a cross-cutting aspect in the area of human performance, work control, because NMPNS did not address the impact of changes to the work activity on the plant and human performance.

Inspection Report# : [2010002](#) (*pdf*)

Barrier Integrity

Significance:  Jun 30, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Loss of Spent Fuel Pool Cooling due to Inadequate Procedure

A self-revealing NCV of TS 5.4.1, "Procedures," was identified for NMPNS' failure to provide an adequate procedure for spent fuel pool (SFP) cooling system restoration. Specifically, procedure N2-PM-082, "RPV Floodup/Draindown," Revision 00600, did not contain adequate guidance for restoring the spent fuel cooling (SFC) system from single to two skimmer surge tank operation during a medium risk configuration for decay heat removal. As a result, on May 17, 2012 SFP cooling was lost when the 'A' SFP pump tripped on low suction pressure. NMPNS restored SFP cooling within 30 minutes and entered this issue into its corrective action program as CR 2012-004850.

This finding is more than minor because it affects the Barrier Integrity Cornerstone attribute of maintaining the functionality of the SFC system to provide reasonable assurance that physical design barriers protect the public from radiological releases caused by accidents or events. The finding was determined to be of very low safety significance (Green), due to it not being associated with a loss of cooling to the SFP that would have precluded restoration prior to boiling, a fuel handling error, or loss of SFP inventory. Specifically, control room operators had a contingency plan in place to restore the loss of SFC if it were to occur.

This finding has a cross-cutting aspect in the area of human performance, resources, because NMPNS did not ensure that procedures were adequate to ensure nuclear safety. Specifically, procedure N2-PM-082 did not provide adequate instructions to establish a valve lineup that would ensure that the SFP cooling pump had sufficient suction pressure. [H.2(c)] (Section 1R13)

Inspection Report# : [2012003](#) (*pdf*)

Significance:  Jun 30, 2012

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Inadequate Special Operating Procedure for Loss of Spent Fuel Pool Cooling

A self-revealing Green NCV of TS 5.4.1, "Procedures," was identified for NMPNS' failure to provide an adequate special operating procedure for loss of SFP cooling. On May 25, 2012, following a loss of SFP cooling, the inadequate procedural guidance delayed restoration of SFP cooling for over two hours during which SFP temperature rose three degrees. NMPNS entered this issue into its corrective action program as CR 2012-004850 to track resolution of this issue.

The finding is more than minor because it was associated with the procedure quality attribute of the Barrier Integrity cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radiological release by maintaining functionality of the SFP cooling system. The finding was determined to be of very low safety significance (Green), due to it not being associated with a loss of cooling to the SFP that would have precluded restoration prior to boiling, a fuel handling error, or loss of SFP inventory.

This finding has a cross-cutting aspect in the area of human performance, resources, because NMPNS did not ensure that procedures were adequate to ensure nuclear safety. Specifically, the special operating procedure for loss of SFP cooling did not contain adequate guidance either for restoration of a two skimmer surge tank lineup or for starting a SFP cooling pump on a single surge tank. [H.2(c)] (Section 40A3.1)

Inspection Report# : [2012003](#) (*pdf*)

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Significance: Dec 31, 2011

Identified By: NRC

Item Type: FIN Finding

Troubleshooting Approach Not Consistent With Technical Specification Bases

The inspectors identified a Green finding for the failure of NMPNS to follow the technical specifications (TS) bases associated with limiting condition for operation (LCO) 3.0.2. Specifically on October 24, 2011, on three separate occasions, NMPNS entered TS 3.3.6.1 Condition B for operational convenience to conduct troubleshooting of a reactor water cleanup (RWCU) system differential flow high channel. The inspectors determined this action was contrary to the bases of TS LCO 3.0.2 which states, in part, intentional entry into actions should not be made for operational convenience and must not compromise safety. NMPNS immediate corrective actions included coaching the control room personnel involved in the troubleshooting process and entered the issue into the corrective action program as CR 2011-009767.

This finding is more than minor because it impacted the configuration control aspect of the Barrier Integrity Cornerstone and adversely affected the Cornerstone objective to maintain functionality of containment. Specifically, as part of a planned troubleshooting activity, a protective isolation feature was removed from service on multiple occasions that collectively exceeded the allowed LCO time for the system. The inspectors determined that the finding was of very low safety significance (Green) since the finding did not represent an actual open pathway in the physical integrity of the reactor containment. This finding has a cross-cutting aspect in the area of human performance in that NMPNS did not use conservative assumptions in decision making when performing multiple entries into TS [H.1.(b)].
Inspection Report# : [2011005](#) (*pdf*)

Emergency Preparedness

Occupational Radiation Safety

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Significance: Sep 30, 2010

Identified By: Self-Revealing

Item Type: FIN Finding

Failure to Maintain Radiation Exposure ALARA During RHR System Modification

A self-revealing finding of very low safety significance was identified due to Nine Mile Point Nuclear Station (NMPNS) having unplanned, unintended occupational collective dose resulting from deficiencies in "as low as is reasonably achievable" (ALARA) planning and work control while performing the removal of steam condensing mode piping and components associated with the Unit 2 residual heat removal (RHR) system. Specifically, NMPNS failed to properly plan and coordinate outage work, and failed to perform welding activities correctly. This resulted in expansion of the collective exposure for this work from 8.557 person-rem to 17.968 person-rem. NMPNS entered this issue into their corrective action program (CAP) as condition report (CR) 2010-8443.

The finding was more than minor because it was associated with the program and process attribute of the Occupational Radiation Safety cornerstone and affected the cornerstone objective to ensure the adequate protection of the worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation. Additionally, the finding was similar to example 6.i in Appendix E of Inspection Manual Chapter (IMC) 0612, in that it resulted in collective exposure of greater than 5 person-rem and exceeded the outage goal by greater than 50 percent. The finding was evaluated in accordance with IMC 0609, Appendix C, "Occupational Radiation Safety Significance Determination Process," and was determined to be of very low safety significance because NMPNS's current three year rolling average collective dose is 144.781 person-rem, less than 240 person-rem per unit. The finding had a cross-cutting aspect in the area of human performance, work control, in that the outage plan did not adequately incorporate actions to address the impact of work on different job activities.

Inspection Report# : [2010004](#) (*pdf*)

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Significance: Sep 30, 2010

Identified By: Self-Revealing

Item Type: FIN Finding

Failure to Maintain Radiation Exposure ALARA During Refueling Floor Activities

A self-revealing finding of very low safety significance was identified due to Nine Mile Point Nuclear Station (NMPNS) having unplanned, unintended occupational collective dose resulting from deficiencies in "as low as is reasonably achievable" (ALARA) planning and work control while performing refueling floor activities at Unit 2. Specifically, the failure to have cleaned up a crud burst that had occurred late in the previous refueling outage, the decision to flood up the refueling cavity while refueling water activity remained four times higher than planned, incorrect calculations during reactor vessel (RV) head stud tensioning that resulted in having to remove the RV head insulation package and re-tension the RV head, and the inability to control work crew size on the refueling floor, resulted in expansion of the collective exposure for this work from 19.810 person-rem to 38.222 person-rem. NMPNS entered this issue into their corrective action program (CAP) as condition report (CR) 2010-8444.

The finding was more than minor because it was associated with the program and process attribute of the Occupational Radiation Safety cornerstone and affected the cornerstone objective to ensure the adequate protection of the worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation. Additionally, the finding was similar to example 6.i in Appendix E of Inspection Manual Chapter (IMC) 0612, in that it resulted in collective exposure of greater than 5 person-rem and exceeded the outage goal by greater than 50 percent. The finding was evaluated in accordance with IMC 0609, Appendix C, "Occupational Radiation Safety Significance Determination Process," and was determined to be of very low safety significance because NMPNS's current three year rolling average collective dose is 144.781 person-rem, less than 240 person-rem per unit. The finding had a cross-cutting aspect in the area of human performance, work control, in that the job site conditions which impacted human

performance were not adequately incorporated into the outage plan.
Inspection Report# : [2010004](#) (*pdf*)

Public Radiation Safety

Security

Although the Security Cornerstone is included in the Reactor Oversight Process assessment program, the Commission has decided that specific information related to findings and performance indicators pertaining to the Security Cornerstone will not be publicly available to ensure that security information is not provided to a possible adversary. Other than the fact that a finding or performance indicator is Green or Greater-Than-Green, security related information will not be displayed on the public web page. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Significance: N/A Oct 21, 2011

Identified By: NRC

Item Type: FIN Finding

PI&R Team Report Summary

The inspectors concluded that Constellation was generally effective in identifying, evaluating, and resolving problems. Constellation personnel identified problems, entered them into the corrective action program at a low threshold, and prioritized issues commensurate with their safety significance. In most cases, Constellation appropriately screened issues for operability and reportability, and performed causal analyses that appropriately considered extent of condition, generic issues, and previous occurrences. The inspectors also determined that Constellation typically implemented corrective actions to address the problems identified in the corrective action program in a timely manner.

The inspectors concluded that, in general, Constellation adequately identified, reviewed, and applied relevant industry operating experience to Nine Mile Point operations. In addition, based on those items selected for review, the inspectors determined that Constellation's self-assessments and audits were thorough.

Based on the interviews the inspectors conducted over the course of the inspection, observations of plant activities, and reviews of individual corrective action program and employee concerns program issues, the inspectors did not identify any indications that site personnel were unwilling to raise safety issues nor did they identify any conditions that could have had a negative impact on the site's safety conscious work environment.

Inspection Report# : [2011008](#) (*pdf*)

Last modified : September 12, 2012