

# Farley 1

## 2Q/2009 Plant Inspection Findings

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### Initiating Events

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### Mitigating Systems

**Significance:**  Jun 30, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

#### **Inadequate Test Procedures for SG Blowdown Isolation Valves**

Green. An NRC identified NCV of 10 CFR 50, Appendix B, Criterion XI, Test Control, was identified for failure to establish adequate procedures to verify automatic closure of the Unit 1 and Unit 2 Steam Generator blowdown (SGBD) isolation valves. This finding has been entered into the licensee's CAP as condition report (CR) 2009107127.

The failure to establish and implement adequate test procedures including acceptance criteria necessary to verify safety-related equipment is capable of performing its design function is a performance deficiency. This finding is more than minor because if left uncorrected, the condition could result in the failure to recognize safety function testing acceptance criteria specified by plant design had not been met, and would be a more significant safety concern. This finding was assessed using the Phase 1 screening worksheet of the SDP and determined to be of very low safety significance (Green), because it did not result in an actual loss of safety function of a single train for greater than the Technical Specification (TS) allowed outage time, and was not potentially risk-significant due to external events. This finding was assigned a cross-cutting aspect in the Problem Identification and Resolution (PI&R) area (P.1(d)) because the licensee had identified the need for additional testing of contacts related to SGBD isolation in March 2006, but no corrective actions were taken to address this issue in a timely matter commensurate with safety significance and complexity. (Section 1R19)

Inspection Report# : [2009003](#) (*pdf*)

**Significance:**  Jun 30, 2009

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

#### **Inadequate Test Procedures for Verification of Steam Dump Arming From Reactor Trip (P-4) Signal and P-4 Contact Verification**

Green. A self-revealing NCV of TS 5.4.1.a. for failure to implement FNP-1-STP- 33.8, Verification of Steam Dump Arming from Reactor Trip (P-4) Signal and P-4 Contact Verification, was identified when licensee personnel took actions not directed by the procedure. During performance of FNP-1-STP- 33.8, licensee personnel changed the established initial conditions by placing the Solid State Protection System (SSPS) into a different operating mode. This finding has been entered into the licensee's CAP as CR 2009104718.

The failure to follow FNP-1-STP- 33.8 is a performance deficiency. This finding is more than minor because it was associated with the Mitigating Systems cornerstone attribute of Configuration Control and adversely affected a cornerstone objective in that failure to follow the procedure resulted in changing the initial conditions previously established. This finding was determined to be of very low safety significance because the procedure was successfully performed prior to entering a mode that required the SSPS to be operable. This finding has a cross-cutting aspect of Work Control in the area of Human Performance (H.3(b)) in that the licensee did not keep personnel apprised that the SG LO-LO level reactor trip signal in SSPS would not be blocked using the "Normal Method." (Section 1R22)

Inspection Report# : [2009003](#) (*pdf*)

**G****Significance:** Jun 30, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

**Failure to Include Water Tight Doors in the Scope of the Maintenance Rule**

Green. An NRC identified NCV of 10 CFR 50.65 (b) was identified for failure to include the Auxiliary Building water-tight doors within the scope of the maintenance rule (MR) monitoring program. During routine plant inspections, the inspectors noticed degraded door seals (excessive hardening of the rubber seal with small chunks missing) and worn hinges on water tight doors located in the Auxiliary Building. The inspectors discovered one Unit 2 door in which the seal area had cracks completely across. The inspectors determined the sealing function assumed in the licensee internal flooding analysis for these doors was challenged. This finding has been entered into the licensee's CAP as CR 2009106669.

The licensee's failure to include the water-tight doors within the scope of their MR program and the subsequent degraded condition of these doors is a performance deficiency. The performance deficiency is more than minor because it adversely affected the equipment performance attribute of the Mitigating System Cornerstone objective to ensure the availability, reliability, and capability of systems responding to initiating events to prevent undesirable consequences (i.e. core damage). This finding was assessed using Phase 1 of IMC 0609, Appendix A, "Significance Determination of Reactor Inspection Findings for At-Power Situations," and determined the finding was of very low safety significance (Green) because although it did involve degradation of equipment or function specifically designed to mitigate a flooding event (i.e. flooding barriers), it did not result in a loss of safety function. No cross cutting aspects were identified. (Section 40A2)

Inspection Report# : [2009003](#) (*pdf*)**G****Significance:** Feb 27, 2009

Identified By: NRC

Item Type: NCV NonCited Violation

**Inadequate Verification of SW Capability to Concurrently Provide System Design Basis Cooling Requirements and the AFW Alternate Water Source.**

Green: The team identified a finding of very low safety significance involving a non-cited violation (NCV) of 10 CFR 50, Appendix B, Criterion III, Design Control. Specifically, the licensee failed to establish measures to verify the design capability of the service water (SW) system to provide water as a suction source for the auxiliary feedwater (AFW) pumps while maintaining adequate SW flow to other safety-related components. The Technical Specification's (TS) action statement for Condensate Storage Tank (CST) Operability, 3.7.6.a, and the corresponding TS bases credit SW as a water source for AFW pumps upon a loss of normal feedwater supply from the CST.

The finding is more than minor because it is associated with the design control attribute of the Mitigating System Cornerstone and affected the cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Due to the lack of appropriate analysis or testing, the SW design basis accident capability was not assured. The team assessed the finding using the SDP and determined that the finding was of very low safety significance (Green) since it was a design deficiency determined not to have resulted in the loss of safety function. Specifically, the licensee had not operated in a condition for which the design deficiency in question was relied upon for operation. The finding was entered into the licensee's corrective action program. There is no cross cutting aspect to this finding because it does not reflect recent performance in that the original SW system analyses were performed in 1990 and 1999 and the inspectors identified no subsequent opportunity for the licensee to identify this deficiency. (Sections 1R21.2.1)

Inspection Report# : [2009006](#) (*pdf*)**W****Significance:** Aug 01, 2008

Identified By: NRC

Item Type: AV Apparent Violation

**Inadequate Work Instructions Cause 1B EDG Exhaust Header Failure.**

A self-revealing finding and Apparent Violation of Technical Specification 5.4.1 was identified for inadequate work

instructions which resulted in the 1B Emergency Diesel Generator (EDG) exhaust header not being installed in accordance with the vendor's instructions. Subsequently, the 1B EDG exhaust header failed during a surveillance test. No immediate safety concern exists because the exhaust header has been repaired and the 1B EDG was returned to service. In addition, the exhaust header replacement had not been implemented on the remaining EDGs.

The failure to provide adequate work instructions for installing the 1B EDG exhaust header is a performance deficiency. This finding is more than minor because it was associated with the equipment performance attribute of the Mitigating Systems cornerstone objective of ensuring availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. In addition the 1B EDG exhaust header failure potentially affected the ability of the 1B EDG to meet its mission time. This finding was assessed using the applicable SDP and determined to White because there was a calculated risk increase over the base case between 1E-5 and 1E-6. The dominant accident sequence is a series of failures which results in a reactor coolant pump seal loss-of-coolant accident that cannot be mitigated leading to core damage. The exposure time assumed in the attached SDP analysis was one-half the period from February 10 to March 13 plus approximately 60 hours repair time for the exhaust header.

Inspection Report# : [2008011](#) (*pdf*)

Inspection Report# : [2008012](#) (*pdf*)

Inspection Report# : [2008013](#) (*pdf*)

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## Barrier Integrity

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## Emergency Preparedness

**Significance:** **W** May 02, 2009

Identified By: NRC

Item Type: VIO Violation

### **Failure to Establish Effective Means of Providing Early Notification (Inaccurate Tone Alert Radios Addresses)**

An AV of 10 CFR 50.47(b)(5) was identified for a failure to maintain the means to provide alert and notification and clear instruction to all of the population within the plume exposure pathway emergency planning zone (EPZ). Specifically, in February 2008 the licensee determined that they had not provided tone alert radios (TARs) to approximately 171 addresses requiring radios, and failed to ensure the State of Georgia had established the capability for compensatory alerting measures. The licensee's failure to maintain the public alert and notification system to meet the design requirements of the Federal Emergency Management Agency (FEMA) approved Alert and Notification system (ANS) design report and supporting FEMA approval letter resulted in a degradation of a risk significant planning standard. The licensee's failure to provide the means for notification and instruction to the populace within the plume exposure pathway EPZ in the event of a radiological emergency as required by 10 CFR 50.47(b)(5) is a performance deficiency. The licensee's failure to remain in compliance with the FEMA approved ANS design report and supporting FEMA approval letter contributed to the performance deficiency. This finding is more than minor because it is associated with the emergency preparedness cornerstone attribute of facilities and equipment, and affected the cornerstone objective of ensuring that the licensee is capable of implementing adequate measures to protect the health and safety of the public in the event of a radiological emergency. The significance of this finding was determined using Manual Chapter 0609 Appendix B, Emergency Preparedness Significance Determination Process (sheet 1) – Failure To Comply. The NRC preliminarily determined this finding to have low to moderate safety significance (White) in that it resulted in the degradation of a Risk Significant Planning Standard (RSPS) function (10 CFR 50.47(b)(5)).

This finding had a cross cutting aspect of Human Performance because the licensee did not adequately ensure supervisory and management oversight of work activities, including the

electrical utilities providing connect and disconnect information regarding addresses within the emergency planning zone, such that nuclear safety was supported (H.4.c).

Update:  
NOTICE OF VIOLATION  
Southern Nuclear Operating Company, Inc. Docket Nos. 50-348, 50-364  
Joseph M. Farley Nuclear Plant License Nos. NPF-2, NPF-8  
Units 1 and 2 EA-09-103  
During an inspection completed by the NRC on May 18, 2009, a violation of NRC requirements was identified. In accordance with the NRC Enforcement Policy, the violation is set forth below: 10 CFR 50.54(q) states, in part, that a licensee authorized to possess and operate a nuclear power reactor shall follow and maintain in effect emergency plans which meet standards in 10 CFR 50.47(b).  
10 CFR 50.47(b)(5) requires, in part, that the licensee establish a means to provide early notification and clear instruction to the populace within the plume exposure pathway Emergency Planning Zone (EPZ).  
The Farley emergency plan identifies both tone alert radios (TARs) and sirens as the means by which it provides alert and notification to the populace within the plume exposure pathway.  
Contrary to the above, the licensee failed to maintain an effective means of providing early notification and clear instruction to the populace within the plume exposure pathway EPZ. Specifically, in January 2008, the licensee identified that approximately 109 TARs had not been provided to residences that were outside the limits of the sirens but within the 10 mile EPZ of Farley Nuclear Plant. The licensee's subsequent review identified additional residences within the 10 mile EPZ which were required to have TARs in accordance with the Farley emergency plan, but were not provided TARs.  
This violation is associated with a White Significance Determination Finding.  
Inspection Report# : [2009502](#) (*pdf*)  
Inspection Report# : [2009503](#) (*pdf*)

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## Occupational Radiation Safety

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## Public Radiation Safety

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## Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

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## Miscellaneous

**Significance:** N/A Dec 19, 2008  
**Identified By:** NRC  
**Item Type:** FIN Finding  
**PI&R Summary**

The inspectors identified that the licensee was effective at identifying problems and putting them into the corrective action program (CAP). The licensee's effectiveness at problem identification was evidenced by a CR generation rate of approximately 1000 per month. However, the inspectors identified two examples of delayed identification. The licensee effectively used risk in prioritizing the extent to which individual problems would be evaluated and in establishing schedules for implementing corrective actions. However, the inspectors identified two examples where errors were made in risk evaluation. Licensee assessments were found to be effective. Assessment results adequately identified problems.

Operating experience usage was found to be effective. Operating experience had been integrated into the licensee's processes for managing work and plant operations. However, the licensee had not been periodically reviewing the Part 21 Notices provided on the NRC public web site. On the basis of interviews conducted during the inspection, workers at the site felt free to input safety findings into the CAP.

The corrective actions implemented and planned, to address the issues identified during the 2008 supplemental IP 95002 and IP 95001 inspections were appropriately targeted. The licensee's response to pipe wall thinning and valve replacement in the Service Water System has been commensurate with safety significance.

Inspection Report# : [2008007](#) (*pdf*)

**Significance:** N/A Aug 24, 2007

Identified By: NRC

Item Type: FIN Finding

### **Biennial Identification and Resolution of Problems Inspection Results**

One finding of very low safety significance (Green) was identified. The licensee was generally effective in identifying problems at a low threshold and entering them into the corrective action program. The licensee properly prioritized issues entered into the corrective action program (CAP) and routinely performed evaluations that were technically accurate and of sufficient depth to address the issue documented in the condition reports (CRs). Overall, corrective actions were effective; however, minor examples of inadequate condition report broadness reviews and documentation issues related to the closure of action items were identified. In general, operating experience was found to be used both proactively and reactively by personnel involved in the corrective action program; however, an example of industry operating experience was identified in which the licensee did not completely develop interim compensatory measures for a condition to which Farley was vulnerable. The licensee's programmatic self-assessments and audits were generally effective in identifying weaknesses in the corrective action program; however, a missed opportunity in the trending of issues which could result in adverse effects on safety-related plant components was identified. The inspectors also concluded that the workers at Farley felt free to report safety concerns.

Inspection Report# : [2007006](#) (*pdf*)

Last modified : August 31, 2009