

Vogtle 2

4Q/2008 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance:  Mar 30, 2008

Identified By: NRC

Item Type: NCV NonCited Violation

Violation of 10CFR50, Appendix B, Criterion XI for Failure To Establish Adequate Test Control Measures For TS SR 3.7.5.2

Green. The inspectors identified a Green NCV of 10 CFR 50, Appendix B, Criterion XI, Test Control, for failure to consider the effects of instrument uncertainty, water temperature, or system orifice resistance during auxiliary feedwater (AFW) pump technical specification required surveillance testing. This finding was entered into the licensee's corrective action program as condition reports CR 2007105436, CR 2007105713, CR 2007105870, and CR 2007105895. Planned corrective actions included revision of the AFW pump test procedures to correct the non-conservative acceptance criteria.

This finding is more than minor because it affects the Mitigating Systems Cornerstone objective of ensuring the availability, reliability, and operability of the AFW pumps to perform the intended safety function during a design basis event and the cornerstone attribute of Procedure Quality, i.e. maintenance and testing procedures. The inspectors assessed the finding using the SDP and determined that the finding was of very low safety significance (Green) because the deficiency did not result in any AFW pumps being inoperable based upon a recent review of AFW surveillance testing results. (Section 4OA5)

Inspection Report# : [2008002](#) (*pdf*)

Significance:  Mar 30, 2008

Identified By: NRC

Item Type: NCV NonCited Violation

Capability of Auxiliary Feedwater System to Meet Design and Licensing Requirements

The inspectors identified a Green NCV of 10 CFR 50, Appendix B, Criterion III, Design Control, for failure to include the cumulative effects of the replacement of the 1A motor driven auxiliary feedwater (AFW) pump rotating element, accuracy of AFW system resistance values, safety relief valve setpoint tolerances, and turbine driven AFW pump speed settings on evaluation of the performance of the AFW system. This finding was entered into the licensee's corrective action program as condition report CR 2007105979. Planned corrective actions included revision of the AFW system flow calculations to incorporate the most limiting design inputs.

This finding is more than minor because it affects the Mitigating Systems Cornerstone objective of ensuring the availability, reliability, and operability of the AFW system to perform the intended safety function during a design basis event and the cornerstone attribute of Design Control, i.e. initial design. The inspectors assessed the finding using the SDP and determined that the finding was of very low safety significance (Green) because the deficiencies did not result in the AFW system being inoperable based upon additional analysis that showed that the AFW system had sufficient flow performance margin to accommodate pump performance and the increased system flow resistance when applying appropriate resistance values and steam generator backpressures.

Inspection Report# : [2008002](#) (*pdf*)

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Significance:  Dec 31, 2008

Identified By: NRC

Item Type: NCV NonCited Violation

Unauthorized Entries Into High Radiation Areas

Two examples of a self-revealing non-cited violation of Technical Specification 5.7.1, High Radiation Area, [were] identified for unauthorized entry into High Radiation Areas (HRAs). Inadequate communication between workers and Health Physics department resulted in licensee personnel breaching HRA boundaries without prior knowledge of the radiological conditions. The licensee had entered these issues into the corrective action program as Condition Reports 2007105476 and 2007108830.

This finding is greater than minor because it is associated with the Occupational Radiation Safety Cornerstone attribute of Human Performance and adversely affects the cornerstone objective of ensuring adequate protection of worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation. The finding was evaluated using the Occupational Radiation Safety Significance Determination Process and was determined to be of very low safety significance because it not related to As Low As Reasonably Achievable (ALARA) planning, did not involve an overexposure or substantial potential for overexposure, and the ability to assess dose was not compromised. This finding involved the cross-cutting aspect of Human Performance, Work Practices [H.4.a] because the HRA events were a direct result of poor communications during pre-job briefings and a willingness on the part of licensee personnel to proceed in the face of uncertainty.

Inspection Report# : [2008005](#) (*pdf*)

Public Radiation Safety

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

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