

McGuire 2

1Q/2008 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance:  Mar 31, 2008

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Establish and Maintain Abnormal Procedures for Loss of Nuclear Service Water (Section 1R11)

The inspectors identified a non-cited violation (NCV) of Technical Specification (TS) 5.4.1.a. for failure to adequately establish and maintain procedures required by Regulatory Guide 1.33, Appendix A, Section 5, Procedures for Abnormal Conditions. Specifically, loss of nuclear service water (RN) procedures were not established and maintained with an adequate safety analysis for the sharing of nuclear service water between units.

This finding is more than minor because it affects the availability, reliability, and capability of the RN system (ultimate heat sink) and is related to the design control and procedure quality attributes of the mitigating systems cornerstone. In addition, this finding could be reasonably viewed as a precursor to a significant event (i.e. loss of RN on both units). The issue was determined to be of very low safety significance in IMC 0609 SDP Phase 1 screening based on the fact that this finding did not represent an actual loss of system safety function nor a loss of a single train of RN for greater than its Technical Specification allowed outage time, because these procedural steps had never been used. This finding has a cross-cutting aspect of resources in the area of human performance [H.2.c] because the licensee failed to ensure that procedures had complete, accurate and up-to-date design documentation to assure nuclear safety. (Section 1R11)

Inspection Report# : [2008002](#) (*pdf*)

Significance:  Mar 31, 2008

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to Implement Foreign Material Exclusion Control Procedures (Section 1R13)

A self-revealing NCV of TS 5.4.1.a, for failure to adequately implement procedures required by Regulatory Guide 1.33, Appendix A, Section 9, Procedures for Performing Maintenance, was identified. Specifically, foreign material exclusion control procedures as described in work orders were not implemented.

This finding is more than minor because it affects the availability, reliability, and capability of one train of the RN system (ultimate heat sink) and is related to the human performance and procedure quality attributes of the mitigating systems cornerstone. This finding was considered self-revealing because the foreign material (i.e., diver's knife) was discovered in the 2A RN strainer as a result of the loss of safety equipment functionality. Data related to the frequency of high RN strainer differential pressure alarms was reviewed by the NRC staff for the seasonal macro-fouling periods of 2006 and 2007 to determine the total actual exposure time that macro-fouling occurred. Based on the data, a collective period of less than 30 days was selected as a conservative, bounding exposure number to determine the significance of the collective seasonal macro-fouling for the period from 2006 until January 28, 2008. The issue is of very low safety significance based on review IMC 0609 Appendix A pre-solved risk tables for loss of one train of nuclear service water for less than 30 days. This finding has a cross-cutting aspect of decision making in the area of human performance [H.1.b] because the licensee failed to use conservative assumptions in decision making when deciding not to implement foreign material procedures. (Section 1R13)

Inspection Report# : [2008002](#) (*pdf*)

Significance: **G** Mar 31, 2008

Identified By: NRC

Item Type: NCV NonCited Violation

Nuclear Service Water System Flow Analysis Deficiencies (Section 40A5.2)

The inspectors identified a NCV of 10 CFR 50, Appendix B, Criterion III, Design Control, for failure to establish measures to verify the design capability of the RN pumps. Specifically, the licensee did not perform system hydraulic analyses or use other means to demonstrate that RN pumps 1A and 1B could perform their safety function under the most limiting design basis conditions.

This finding is more than minor because it affected the design control attribute of the mitigating systems cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. The inspectors assessed the finding using the SDP and determined that the finding was of very low safety significance because subsequent engineering analysis, completed in March 2008, demonstrated there was no loss of RN system safety function capability when the worst case design basis accident (DBA) limiting values were input into the RN system flow analysis. (Section 40A5.2)

Inspection Report# : [2008002](#) (pdf)

Significance: **G** Nov 02, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Take Adequate Corrective Action For A Nonconformance Associated With ECCS Throttle Valves.

The purpose of this letter is to provide you with the Nuclear Regulatory Commission's (NRC's) final significance determination for a finding at Duke Power Company's (Duke) McGuire Nuclear Station (MNS) involving the failure to take adequate corrective actions for an identified nonconformance. This nonconformance involved the discovery that the emergency core cooling system (ECCS) cold leg injection throttle valves had the potential for clogging during high pressure recirculation because their narrow plug-to-seat clearances were smaller than the ECCS sump screen openings. More specifically, Duke's corrective action failed to adequately implement credited inspections of the inside of the ECCS sump. This was evidenced by the 2006 discovery, during an unrelated inspection, of a significant amount of aged yellow duct tape inside the Unit 2 ECCS sump around the suction and guard pipe of both ECCS trains. As documented in our Choice Letter dated September 10, 2007, this finding was assessed under the significance determination process as a preliminary greater than Green issue (i.e., an issue of at least low to moderate safety significance), as well as identified as an apparent violation (AV 05000370/ 2007008-01) of 10 CFR Part 50, Appendix B, Criterion XVI, Corrective Action.

The NRC has determined that it is not likely that the two HHI throttle valves which are 2.75 turns open would clog to the point of precluding sufficient decay heat removal. As such, it has been concluded that the inspection finding is appropriately characterized in the mitigating systems cornerstone as having very low safety significance (Green). This final significance determination should not be construed as minimizing the importance of maintaining ECCS sump foreign material exclusion. Rather, it reflects how fortuitous it was that the foreign material consisted entirely of soft debris and that MNS has robust ECCS pump and ND heat exchanger designs, as well as the redundancy of both an IHI and HHI system (the latter of which has two of its four throttle valves approximately 2.75 turns open).

Additionally, the finding was also determined to be a violation of NRC requirements, as delineated in the Choice Letter and presented during the regulatory conference (see Enclosure 2). As previously addressed in the Choice Letter, this finding has a cross-cutting aspect of appropriate corrective actions in the area of problem identification and resolution (Inspection Manual Chapter 0305, Section 06.07, P.1.(d)), and is reflective of the importance in properly implementing established engineering processes to ensure plant licensing and design bases are maintained when dispositioning conditions adverse to quality.

Inspection Report# : [2007010](#) (pdf)

Significance: SL-IV Sep 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform a Written Safety Evaluation for a Change to the Facility

The inspectors identified a non-cited violation of 10 CFR 50.59 for removing the approved seismic qualification methodology (WCAP-8110, supplement 9) from the Updated Final Safety Analysis Report (UFSAR) without performing a written safety evaluation. This issue is in the licensee's corrective action program as PIP M-07-5016.

The failure to perform a written safety evaluation for changes made to the facility as described in the UFSAR is more than minor because there was a reasonable likelihood that the change requiring a 10 CFR 50.59 written safety evaluation would require Commission review and approval prior to implementation in accordance with 10 CFR 50.59 (c)(2). This likelihood is based on the November 21, 1974, NRC Safety Evaluation Report for WCAP-8110

Supplement 9, which stated the WCAP is considered an accepted methodology to demonstrate the continued adequacy of ice retention characteristics of the ice baskets when used as a reference for license applications. Removal of this approved methodology from the licensing basis would constitute a change in methodology and would require NRC review and approval. This issue was treated as traditional enforcement because it had the potential for impacting the NRC's ability to perform its regulatory function. It was characterized as a severity level IV violation because it was evaluated as not having greater than very low safety significance. (Section 1R15)

Inspection Report# : [2007004](#) (pdf)

Barrier Integrity

Significance:  Dec 31, 2007

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Failure to take adequate corrective action for ensuring containment spray isolation valve thrust capacity.

The inspectors identified a self-revealing NCV of 10 CFR 50, Appendix B, Criterion XVI, for inadequate corrective action associated with the prevention of exceeding the thrust capacities of containment spray (NS) isolation valves (due to differential pressure), which could have prevented the NS system from performing its intended safety function.

This issue is more than minor because it affects the availability, reliability, and capability of the NS system and is related to the equipment performance and procedure quality attributes of the mitigating systems cornerstone. This finding was considered self-revealing because a temporary gauge installed to detect cross-train pressurization during NS pump runs revealed the unexpected existence of significant cross-system leakage from the residual heat removal (ND) system. The issue is of very low safety significance based on review IMC 0609 Appendix H, which indicates that containment spray does not impact large early release frequency for pressurized water reactor plants. This finding has a cross-cutting aspect of decision making in the area of human performance (H.1.b). (Section 1R22)

Inspection Report# : [2007005](#) (pdf)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Significance: SL-IV Sep 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Promptly Correct a Condition Adverse to Quality

The inspectors identified a non-cited violation of 10 CFR 72.172 for failing to promptly identify and correct a condition adverse to quality associated with not performing 10 CFR 72.48(c) evaluations on five previous revisions of 10 CFR 72.212 written evaluations for the Independent Spent Fuel Storage Installation (ISFSI). This issue is in the licensee's corrective action program as PIP M-07-4321. This issue is greater than minor because the failure to promptly correct and perform 10 CFR 72.48(c) evaluations on any changes to 10 CFR 72.212 written evaluations had a reasonable likelihood that the changes could require NRC review and approval. This issue was considered as traditional enforcement because it had the potential for impacting the NRC's ability to perform its regulatory function. It was characterized as a severity level IV violation because it was evaluated as not having greater than very low safety significance. This finding has a cross-cutting aspect of timely correct action in the area of problem identification and resolution [P.1.d]. (Section 4OA5)

Inspection Report# : [2007004](#) (*pdf*)

Last modified : June 05, 2008