

Palo Verde 3

4Q/2007 Plant Inspection Findings

Initiating Events

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO SCOPE CONDENSATE DEMINERALIZER VALVE INTO MAINTENANCE RULE

A self-revealing noncited violation of 10 CFR 50.65(b) was identified for the failure of engineering personnel to place some components of the condensate demineralizer system into the scope of its program for monitoring the effectiveness of maintenance. Specifically, on October 19, 2006, Unit 3 reactor was manually tripped when condenser vacuum was degraded due to the failure of condensate demineralizer vessel waste drain Valve 3JSCNUV0232. Prior operating experience at Palo Verde demonstrated that the failure of Valve 3JSCNUV0232 could result in a reactor trip. However, the licensee did not appropriately scope Valve 3JSCNUV0232 into its program for monitoring the effectiveness of maintenance. This issue was entered into the corrective action program as Condition Report/Disposition Request 3035444.

The finding is greater than minor because it is associated with the initiating events cornerstone attribute of equipment performance and affects the cornerstone objective to limit the likelihood of those events that upset plant stability and challenge critical safety functions during shutdown as well as power operations. Using Inspection Manual Chapter 0609, "Significance Determination Process," Phase 1 Worksheet, the finding is determined to have very low safety significance since it does not contribute to both the likelihood of a reactor trip and the likelihood that mitigation equipment or functions will not be available.

Inspection Report# : [2007003](#) (*pdf*)

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO APPLY INDUSTRY OPERATING EXPERIENCE TO MAINTENANCE ACTIVITIES RESULTS IN A PLANT TRANSIENT

The inspectors identified a noncited violation of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Actions," for the failure of inservice inspection personnel to promptly identify and correct a condition adverse to quality. Specifically, since April 19, 2006, floor-welded spray pond pipe Supports 13-SP-030-H-007 and 13-SP-030-H-008 in the essential pipe density tunnel became degraded at the weld due to long term standing water in the tunnel. The licensee thought these supports had been previously identified and placed in the corrective action program, but that was not the case. This issue was entered into the corrective action program as Palo Verde Action Request 2989960.

The finding is greater than minor because if left uncorrected the degradation would have led to a more significant safety concern. The finding is associated with the mitigating systems cornerstone. Using the Manual Chapter 0609, "Significance Determination Process," Phase 1 Worksheet, the finding is determined to have very low safety significance since it only affected the mitigating systems cornerstone and did not represent a loss of system safety function, an actual loss of safety function of a single train for greater than its Technical Specification allowed outage time, or screen as potentially risk-significant due to a seismic, flooding, or severe weather initiating event. The cause of the finding is also related to the crosscutting aspect of problem identification and resolution with a corrective action program causal factor because the threshold for identifying issues was not sufficiently low and the degraded supports were not identified completely, accurately, and in a timely manner commensurate with their safety significance (P.1. (a)).

Inspection Report# : [2007003](#) (*pdf*)

Mitigating Systems

Significance:  Aug 17, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate procedure for safe shutdown from outside the control room

Green. The team identified a noncited violation of License Conditions 2.C.(7), 2.F and 2.C.(6) for Units 1, 2, and 3, respectively. Specifically, procedures required by 10 CFR Part 50, Appendix R, Section III.G.3 and III.L.3 had deficiencies that might impact the ability to complete a number of time-critical steps required to safely shutdown the facility following a fire in the control room. This was because the licensee failed to provide a number of tools necessary to complete the procedure as written. The team determined that, although operators did not use the equipment during time-critical steps, the lack of tools could negatively impact the ability to accomplish subsequent time-critical steps.

This deficiency was more than minor because the finding is associated with the Protection Against External Factors attribute of the Mitigating Systems Cornerstone since it could affect the the availability, reliability, and capability of systems that respond to a fire events to prevent undesirable consequences. Using the guidance of Manual Chapter 0609, Appendix F, Attachment 2, the deficiency was determined to have a low degradation rating because it involved a procedural deficiency that was compensated by operator experience/familiarity, and revised calculations demonstrated that there was sufficient time margin available to complete the actions. Based on this, the finding screened as having very low safety significance (Green) during a Phase 1 significance determination. This finding had cross-cutting aspects in the area of human performance because the licensee failed to ensure that personnel, equipment, procedures, and other resources were available and adequate to assure nuclear safety. Specifically, the licensee did not ensure that adequate emergency equipment was available to support procedure completion. (H.2(d)).
Inspection Report# : [2007008](#) (*pdf*)

Significance:  Jun 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO IDENTIFY DEGRADED STRUCTURAL SUPPORTS

A self-revealing noncited violation of 10 CFR 50.65(a)(3) was identified for failure of the licensee to incorporate internal and external industry operating experience into preventative maintenance activities that could have prevented a maintenance rule functional failure of feedwater pump Turbine A, a high risk heat removal system. Specifically, prior to March 18, 2007, the licensee did not incorporate available operating experience into preventative maintenance instructions to inspect, clean, and verify acceptable equipment condition for the linear variable differential transmitter linkage assembly. Failure to inspect and clean the linear variable differential transmitter linkage assembly resulted in a broken linkage, due to binding, causing erratic cycling of the feedwater pump turbine control valves resulting in a manual trip of feedwater Pump A and reactor power cutback to 48 percent power. This issue was entered into the corrective action program as Condition Report/Disposition Request 2984713.

The finding is greater than minor because it is associated with the initiating events cornerstone attribute of equipment performance and affects the cornerstone objective to limit the likelihood of those events that upset plant stability and challenge critical safety functions during shutdown and power operations. Using the Manual Chapter 0609, "Significance Determination Process," Phase 1 Worksheet, the finding is determined to have very low safety significance because the finding did not result in exceeding the Technical Specification limit for identified reactor coolant system leakage and did not affect other mitigation systems; the finding did not contribute to both the likelihood of a reactor trip and the likelihood that mitigation equipment or functions will not be available; and the finding did not increase the likelihood of a fire or internal/external flood. This finding has a crosscutting aspect in the area of problem identification and resolution, associated with operating experience, since engineering personnel failed to account for prior operating experience in determining the maintenance rule scope and appropriate preventive maintenance for Valve 3JSCNUV0232 (P.2(b)).

Inspection Report# : [2007003](#) (*pdf*)

Significance:  May 25, 2007

Identified By: NRC

Item Type: FIN Finding

Ineffective Demonstration of Conformance to Design for the Alternate ac Power Sources

The team identified a finding involving the implementation of Regulatory Guide 1.155, Station Blackout, Appendix A, for the demonstration of the station blackout generator design and system readiness requirements. Specifically, established preventive maintenance tasks did not demonstrate that the coping requirements for the station blackout generator would be met for the approved increase from the 4-hour to 16-hour coping duration that, at the time this finding was identified, would become effective the following month. The licensee has entered this finding into their corrective action program as Palo Verde Action Request PVAR 2982699.

The finding is greater than minor because it would become a more significant safety concern if left uncorrected following the implementation of the 16-hour coping duration. The finding affected the mitigating systems cornerstone attributes to ensure the availability of the station blackout generators to respond to initiating events necessary to prevent undesirable consequences. Using the NRC Inspection Manual Chapter 0609, Significance Determination Process, Phase 1 Worksheet, the team determined that this finding had very low safety significance because there was not a loss of system function and it did not involve an external event. The cause of the finding was related to the crosscutting element of decision making associated with human performance for the failure to adequately evaluate the design and system readiness requirements for the station blackout generators for the approved license amendment that, at the time the finding was identified, would, increase the coping period to 16-hours.

Inspection Report# : [2007011](#) (*pdf*)

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Significance: May 25, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Control of Design Information for the Station Blackout System

The team identified a noncited violation of very low safety significance for the failure to implement the design control requirements of Regulatory Guide 1.155, Station Blackout, Appendix A, Criterion 1, Design Control and Procurement Control, to 10 CFR 50.63, Loss of All Alternating Current. Specifically, approved Design Change DMWO 2827452 did not account for key station blackout generator performance parameters that included fuel and lubricating oil consumption rates and required station blackout battery capacity for an increase in the station blackout coping period from 4 to 16-hours.

The finding is greater than minor because it would become a more significant safety concern if left uncorrected in that the critical performance parameters for ensuring the station blackout generators would meet the 16-hour coping requirement were not established. The finding affected the mitigating systems cornerstone attributes to ensure the availability of the station blackout generators to respond to initiating events necessary to prevent undesirable consequences. Using the NRC Inspection Manual Chapter 0609, Significance Determination Process, Phase 1 Worksheet, the team determined that this finding had very low safety significance because there was not a loss of system function and it did not involve an external event. The cause of the finding was related to the crosscutting element of decision making associated with human performance for the failure to evaluate the key performance parameters for the station blackout generators for the approved license amendment that increased the coping period to 16-hours. (Section 1R21b.2.)

Inspection Report# : [2007011](#) (*pdf*)

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Significance: May 25, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Non-conservative Containment Sump Level Analysis

The team identified a noncited violation of very low safety significance of 10 CFR Part 50, Appendix B, Criterion III, Design Control. Specifically, the design calculation that determined the minimum containment flood level following a loss-of-coolant accident was not based on the most limiting reactor coolant system break location. The calculated containment flood level was used to verify the adequacy of the available net positive suction head for the emergency core cooling pumps that would take suction from the containment sump during the recirculation phase of a postulated loss-of-coolant accident. The licensee has entered this issue into their corrective action program as Palo Verde Action Request PVAR 2981257.

This finding is greater than minor because this issue required accident analysis calculations to be re-performed to assure the accident requirements were met. The finding affected the mitigating systems cornerstone as related to the availability, reliability, and capability of the emergency core cooling system for post-loss-of-cooling accident. In accordance with Inspection Manual Chapter 0609, Significance Determination Process, Appendix A, Significance Determination of Reactor Inspection Findings for At-Power Situations, the team conducted a Phase 1 screening and determined the finding was of very low safety significance because it did not represent an actual loss of safety function. This deficiency would not have resulted in the emergency core cooling pumps becoming inoperable under the most limiting postulated accident conditions. This finding has cross-cutting aspects associated with corrective action of the problem identification and resolution area to ensure that issues potentially impacting nuclear safety are promptly identified, fully evaluated and that actions are taken to address safety issues in a timely manner.

Inspection Report# : [2007011](#) (*pdf*)

Significance:  May 25, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Ineffective Maintenance on Target Rock Solenoid-Operated Valves

The team identified a noncited violation of very low safety significance of 10 CFR Part 50, Criterion XVI, Corrective Actions, for the failure to identify and correct significant conditions adverse to quality involving Target Rock valve failures. The licensee has entered this issue into their corrective action program as Palo Verde Nuclear Generating Station Action Requests PVAR 2984832 and 2985372.

The failure to identify and correct the cause(s) of turbine-driven auxiliary feedwater pump Target Rock solenoid-operated valves was a performance deficiency. This issue is more than minor because it is associated separately with the mitigating systems cornerstone and on one occasion affected the containment barrier integrity cornerstone. This finding has cross-cutting aspects associated with corrective action of the problem identification and resolution area to ensure that issues potentially impacting nuclear safety are promptly identified, fully evaluated and that actions are taken to address safety issues in a timely manner.

Inspection Report# : [2007011](#) (*pdf*)

Significance:  Mar 31, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Change to Emergency Diesel Generator Intake Air Oil Bath Filter Standby Oil Level Specification

The inspectors identified a noncited violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the failure of engineering personnel to verify or check the adequacy of design for maintaining the emergency diesel generator air intake oil bath filters' oil level below the "add oil" mark. Specifically, from approximately November 1994 to January 24, 2007, engineering personnel failed to translate vendor requirements for the Air Maze oil bath air filter oil level into an appropriate operating band. This issue was entered into the corrective action program as Condition Report/Disposition Request 2963525.

The finding is greater than minor because it is associated with the design control attribute of the mitigating systems cornerstone and affects the associated cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Using the Manual Chapter 0609, "Significance Determination Process," Phase 1 Worksheet, the finding is determined to be of very low safety significance because it did not represent an actual loss of system safety function, did not represent an actual loss of a single train for greater than its technical specification allowed outage time, and the finding did not screen as potentially risk significant due to a seismic, flooding, or severe weather initiating event.

Inspection Report# : [2007002](#) (*pdf*)

Significance:  Mar 31, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Identify Misalignment of Spring Cans

The inspectors identified a noncited violation of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," for

the failure of inservice inspection personnel to promptly identify misalignment of spring cans on safety-related piping. Specifically, between April 2005 and May 2006, inservice inspection personnel failed to identify misalignment of spring cans associated with the auxiliary feedwater system and the emergency diesel generators. Section 8.3.5 of Procedure 73TI-9ZZ18 required that the examination of piping systems should be directed to detect any relevant conditions, including misalignment of supports. This issue was entered into the corrective action program as Palo Verde Action Request 2980767.

The finding is greater than minor because it would become a more significant safety concern if left uncorrected in that the failure to identify degraded and non-conforming equipment conditions could impact the availability of mitigating equipment. The finding affected the mitigating systems cornerstone. Using the Manual Chapter 0609, "Significance Determination Process," Phase 1 Worksheet, the finding is determined to have very low safety significance since it did not represent a loss of system safety function, an actual loss of safety function of a single train for greater than its technical specification allowed outage time, or screen as potentially risk-significant due to a seismic flooding, or severe weather initiating event. The finding has a crosscutting aspect in the area of problem identification and resolution, associated with corrective action program, since inservice inspection personnel had an inappropriately high threshold for recognizing the misalignment of spring cans on safety-related piping.

Inspection Report# : [2007002](#) (*pdf*)

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Significance: Feb 09, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Technical Evaluation of HPSI Pump Bearing Oil Leaks

A noncited violation of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures and Drawings," with two examples was identified for two inadequate operability evaluations. Prompt operability determinations in CRDRs 2941494 and 2303499 incorrectly concluded that High Pressure Safety Injection Pumps 2A and 3A, respectively, could meet their mission time with existing oil leakage from the bearings. The team concluded that these evaluations relied upon unverified and incorrect assumptions and non-conservative volumes. The apparent cause evaluation for the leakage identified contributing causes that were common to all pumps, but the operability of the other pumps was not assessed. The team identified a history of small oil leaks in high pressure safety injection pumps since 2000, but the licensee was unaware of this trend. Subsequent testing confirmed that five of the six high pressure safety injection pumps had oil leakage which would not allow running those pumps for the full mission time, but sufficient oil was available to run for at least 94 days. This finding was determined to have cross-cutting aspects in the human performance area of decision-making, because the licensee did not use conservative assumptions and demonstrate that the proposed course of action was safe.

Failure to adequately evaluate and correct oil leakage in High Pressure Safety Injection Pumps 2A and 3A, and failure to assess the extent of condition for similar pumps, was a performance deficiency. The finding was more than minor because it affected the equipment performance attribute of the mitigating systems cornerstone objective of ensuring the availability and reliability of a system that responds to initiating events. This finding screened as Green during Phase 1 of the significance determination process because it did not involve a loss of safety function. This issue was entered into the corrective action program under Condition Report/Disposition Report 2973682.

Inspection Report# : [2007007](#) (*pdf*)

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Significance: Feb 09, 2007

Identified By: NRC

Item Type: FIN Finding

Preventive Maintenance Change Backlog Was Not Tracking Due Dates

A finding was identified for failure to schedule and perform preventive maintenance tasks that were in the preventive maintenance change process. The team identified that a backlog of over 2500 preventive maintenance changes existed which resulted in these preventive maintenance tasks not being scheduled or performed, potentially challenging completion within the specified frequency. The team found 438 examples of preventive maintenance tasks that were overdue, and an additional 2113 that had no due date assigned yet. This program was used to revise both safety-related and non-safety preventive maintenance tasks. Because these preventive maintenance tasks were in the change process, the tasks were not scheduled or tracked in a way that would show when they became overdue. This was contrary to Procedure 30DP-9MP08, "Preventive Maintenance Program," Revision 17, which required that "no preventive maintenance on operational equipment shall pass that late date without an approved deferral which will

address a technical justification for the identified issue.” This finding had human performance cross-cutting aspects associated with resources because the large backlog of preventive maintenance tasks was contrary to maintaining long-term equipment reliability.

Failure to track, schedule, and perform preventive maintenance activities within their specified frequencies in accordance with their preventive maintenance program was a performance deficiency. This finding was determined to be more than minor because, if left uncorrected, it could become a more significant safety concern in that the lack of preventive maintenance would affect the reliability of plant equipment which could impact the initiating events or mitigating systems cornerstones. Because of the large number of preventive maintenance tasks (over 2500) in this category, the team reviewed a sample of 79 tasks associated with safety-related or quality-class components to assess the significance. The team did not identify any examples of overdue safety-related tasks. Based on the lack of risk significant examples and the fact that this finding is not suitable for significance determination process evaluation, this issue was reviewed by NRC management and was determined to be a finding of very low safety significance. This issue was entered into the corrective action program under Palo Verde Action Request 2970076.

Inspection Report# : [2007007](#) (*pdf*)

Significance: **W** Nov 30, 2006

Identified By: NRC

Item Type: VIO Violation

FAILURE TO ESTABLISH APPROPRIATE INSTRUCTIONS

The team identified an apparent violation of 10 CFR Part 50, Appendix B, Criterion V, “Instructions, Procedures, and Drawings,” states, in part, that activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures, or drawings. Specifically, between July 25 and September 22, 2006, activities affecting quality were not prescribed by documented instructions appropriate to the circumstances. Specifically, the licensee failed to develop appropriate instructions or procedures for corrective maintenance activities on the Unit 3, Train A Emergency Diesel Generator K-1 relay. This resulted in the emergency diesel generator being inoperable between September 4 and September 22, 2006. The cause of this finding is related to the crosscutting element of human performance associated with resources in that the licensee failed to develop and implement appropriate work instructions prior to performing corrective maintenance activities on an emergency diesel generator K-1 relay.

The NRC assessed this finding through Phase 3 of the Significance Determination Process and made a preliminary determination that it is an issue with low to moderate safety significance.

After considering the information developed during the inspection, the NRC has concluded that the inspection finding is appropriately characterized as White (i.e., an issue with low to moderate increased importance to safety). On February 21, 2007, a final significance determination letter was issued which characterized VIO 050000530/2006012-01 and VIO 050000530/2006012-02 as a single White SDP finding. These violations will be inspected within the scope of a supplemental 95001 inspection

Inspection Report# : [2006012](#) (*pdf*)

Significance: **W** Nov 30, 2006

Identified By: NRC

Item Type: VIO Violation

FAILURE TO IDENTIFY AND CORRECT A CONDITION ADVERSE TO QUALITY

The team identified an apparent violation of 10 CFR Part 50, Appendix B, Criterion XVI, “Corrective Action,” states, in part, that for significant conditions adverse to quality, measures shall assure that the cause of the condition is determined and corrective action taken to preclude repetition. Specifically, on July 26, 2006, the licensee failed to assure that the cause of a significant condition adverse to quality was determined and that corrective action was taken to preclude repetition. Specifically, the licensee did not identify and correct the cause of the erratic Unit 3, Train A Emergency Diesel Generator K-1 relay operation prior to installation of the relay on July 26, 2006. This resulted in the emergency diesel generator being inoperable between September 4 and September 22, 2006. The cause of this finding is related to the crosscutting element of problem identification and resolution in that the failure to fully evaluate and implement adequate corrective maintenance actions for the Unit 3 Train A emergency diesel generator resulted in the emergency diesel generator being inoperable for 18 days.

The NRC assessed this finding through Phase 3 of the Significance Determination Process and made a preliminary determination that it is an issue with low to moderate safety significance.

After considering the information developed during the inspection, the NRC has concluded that the inspection finding is appropriately characterized as White (i.e., an issue with low to moderate increased importance to safety). On February 21, 2007, a final significance determination letter was issued which characterized VIO 050000530/2006012-01 and VIO 050000530/2006012-02 as a single White SDP finding. These violations will be inspected within the scope of a supplemental 95001 inspection

Inspection Report# : [2006012](#) (*pdf*)

Significance: N/A Sep 30, 2006

Identified By: NRC

Item Type: FIN Finding

SUMMARY FINDING. 95002 TEAMS ASSESSMENT OF IR 2004-14 (YELLOW) 10 CFR PART 50, APPENDIX B, CRITERION III, VIOLATION

The NRC performed a followup supplemental inspection to assess the licensee's corrective actions associated with a Yellow design control finding involving the potential for air entrainment into the emergency core cooling system. The team concluded that the technical issues specifically associated with the voided emergency core cooling system piping have been addressed. However, the Yellow finding will remain open because the licensee did not implement effective corrective actions for all of the causes associated with the Yellow finding. Specifically, the licensee's actions to improve questioning attitude, technical rigor, and technical review were not fully effective. Also, the implementation of performance measures and metrics to monitor the effectiveness of corrective actions associated with the Yellow finding were not adequate to assess effectiveness. This performance issue was previously characterized as a 10 CFR Part 50, Appendix B, Criterion III, violation having substantial safety significance (Yellow), and was originally identified in NRC Inspection Report 05000528; 05000529; 05000530/2004014.

The licensee's corrective actions taken in response to the root causes and related programmatic concerns involving questioning attitude, technical rigor, and technical review have not been completely effective. Specifically, following implementation of corrective actions between September 2005 and March 2006, the licensee: (1) continued to conduct inadequate technical reviews of emerging issues; (2) did not routinely question the validity of engineering assumptions used to support operability decisions; (3) did not consistently implement a qualify, validate, and verify process; and (4) did not consistently notify operations personnel of immediate operability concerns.

The team concluded that adequate qualitative or quantitative measures for determining the effectiveness of the corrective actions to prevent recurrence have not been established. For example, not all relevant performance data was considered when performance monitoring measures were developed to assess the effectiveness of corrective actions. When the pertinent data was considered, or otherwise clarified, the performance measures suggested declining rather than improving performance in some areas.

The team also concluded that the licensee had not completed adequate reviews of the effectiveness of corrective actions prior to their notifying the NRC of their readiness for inspection of the Yellow finding. Specifically, several assessments were completed after the requested dated of the inspection (June 2006). Several of the assessments noted that insufficient progress in resolving some of the root and contributing causes had been made. Additionally, a standard guideline for metrics was not issued and implemented until July 2006.

Inspection Report# : [2006010](#) (*pdf*)

Significance:  Mar 16, 2005

Identified By: NRC

Item Type: FIN Finding

FAILURE TO TRACK CONTROL ROOM DISCREPANCIES

The inspectors identified a finding for the failure to follow administrative guidelines provided to operations personnel for identifying, documenting, and tracking main control room deficiencies. Specifically, approximately 75 control room instrument and control room meter face plates in Units 1, 2, and 3 were degraded and were not individually tracked in the control room discrepancy log. Furthermore, discrepancy labels containing the control room discrepancy log number and description of the discrepancy were not placed adjacent to or as close as possible to each affected

device. This issue was entered into the corrective action program as Condition Report/Disposition Request 2782501.

The finding is determined to be greater than minor because if left uncorrected, it could become a more significant safety concern in that the condition could cause an operator to take an inappropriate action based on expected plant response or conversely cause an operator not to take action when action is required. The senior reactor analyst determined that this finding was not appropriate to be evaluated using the significance determination process since this finding was associated with multiple human performance actions. Based on management review, the finding is determined to have very low safety significance because it only affected the mitigating systems cornerstone, and there was no adverse impact to plant equipment.

Inspection Report# : [2005002](#) (*pdf*)

Significance:  Dec 09, 2004

Identified By: NRC

Item Type: VIO Violation

FAILURE TO MAINTAIN DESIGN CONTROL OF CONTAINMENT SUMP RECIRCULATION PIPING

The team identified an apparent violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," for the failure to establish measures to assure design basis information was translated into specifications, drawings, procedures, and instructions. Specifically, the licensee failed to maintain the safety injection sump suction piping full of water in accordance with the Updated Final Safety Analysis Report. This nonconformance had the potential to significantly affect the available net positive suction head described in the Updated Final Safety Analysis Report for the high pressure safety injection and containment spray pumps, since the analysis assumed the piping would be maintained full of water.

{Note: Finding remains open - IP 95002 results pending 12/16/2005 }

This finding is more than minor because it is associated with the equipment performance attribute of the Mitigating Systems cornerstone and adversely affects the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events. The NRC assessed this finding through Phase 3 of the Significance Determination Process and made a preliminary determination that the issue had substantial safety significance (Yellow). After considering the information developed during the inspection and the results of testing sponsored by the licensee, the NRC has concluded that this inspection finding is appropriately characterized as Yellow. The final Significance Determination Process letter was issued on April 8, 2005. This issue was inspected within the scope of a Supplemental 95002 Inspection in August - September 2005.

{NOTE: Yellow finding remains open because the corrective actions taken in response to the root causes and related programmatic concerns involving questioning attitude, technical rigor, and operability determinations have not been fully effective. - IP 95002 Supplemental Inspection completed December 2005, IR 05000528/20050112, 05000529/20050112 and 05000530/2005012, IP 95002 Followup Supplemental Inspection completed August 2006, IR 05000528/2006010, 05000529/2006010 and 05000530/2006010 }

Inspection Report# : [2004014](#) (*pdf*)

Barrier Integrity

Emergency Preparedness

Significance:  Sep 30, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO IDENTIFY AND CRITIQUE AN EVENT CLASSIFICATION WEAKNESS

The inspectors identified a noncited violation of 10 CFR 50.54(q) for failure of the emergency planning organization's emergency exercise critique process to identify for correction an emergency plan weakness associated with a risk

significant planning standard. Specifically, during the critique of the Emergency Preparedness portion of the August 22, 2007, Force-On-Force exercise, the licensee failed to identify for correction an event classification weakness. The weakness occurred during the exercise when the shift manager did not recognize a credible security threat notification was made to the facility. As a result, the shift manager did not declare a Notice of Unusual Event as required by EPIP-99, Appendix A, "Emergency Actions Levels - EAL 7-1." This issue was entered into the licensee's corrective action program as Condition Report/Disposition Request 3056153.

This finding is greater than minor because it is associated with the Emergency Response Organization Performance attribute of the Emergency Preparedness Cornerstone and affects the cornerstone objective to ensure that the licensee is capable of implementing adequate measures to protect the health and safety of the public in the event of a radiological emergency. In accordance with Manual Chapter 0609, "Significance Determination Process," Appendix B, Emergency Preparedness Significance Determination Process, this finding is determined to have very low safety significance because, although it was a failure to comply with NRC requirements, it did not involve the risk-significant aspects of a planning standard as defined in Manual Chapter 0609, Appendix B, Section 2.0; and was not a planning standard functional failure because the critique failure occurred in a small scale drill with limited emergency response organization participation and evaluation. This finding has a crosscutting aspect in the area of problem identification and resolution associated with corrective action program because the threshold for identifying issues was not sufficiently low. Specifically, the emergency planning evaluator did not recognize the shift manager's failure to make the Notice of Unusual Event classification during the Force-On-Force exercise. Therefore, the exercise critique did not identify and correct the event classification deficiency as required (P.1(a)).
Inspection Report# : [2007004](#) (*pdf*)

Occupational Radiation Safety

Public Radiation Safety

Significance:  Feb 22, 2007

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to control the release of radioactive material

A self-revealing, noncited violation of Technical Specification 5.4.1 was reviewed regarding the failure to control the release of radioactive material. On February 2, 2006, the licensee was notified by another site that equipment received was labeled as radioactive material. Specifically, five items, with a maximum activity of 280 counts per minute, were inappropriately released from the radiologically controlled area and subsequently the protected area. The licensee's corrective actions include evaluating and implementing changes to the material release program and processes.

The finding is greater than minor because it was associated with the Public Radiation Safety cornerstone attribute of human performance and affected the associated cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain as a result of routine civilian nuclear reactor operation. Using the Public Radiation Safety Significance Determination Process, the team determined that the finding had very low safety significance because: (1) it was a radioactive material control finding, (2) it was not a transportation finding, (3) it did not result in public dose greater than 0.005 rem, and (4) the number of occurrences was not greater than five. In addition, this finding had a human performance cross-cutting aspect associated with work practices because the licensee failed to ensure supervisory and management oversight of work activities, including contractors

Inspection Report# : [2007010](#) (*pdf*)

Physical Protection

Although the NRC is actively overseeing the Security cornerstone, the Commission has decided that certain findings pertaining to security cornerstone will not be publicly available to ensure that potentially useful information is not provided to a possible adversary. Therefore, the [cover letters](#) to security inspection reports may be viewed.

Miscellaneous

Significance: N/A Feb 09, 2007

Identified By: NRC

Item Type: FIN Finding

Summary Finding. Biennial PI&R Assessment

The team concluded that the thresholds for identifying and classifying issues were appropriately low, although several instances were identified where new aspects to complex problems were identified but not broken out and addressed properly. Numerous changes were made to the corrective action program and some improvement was evident, but some of the changes were not yet fully effective. The new Palo Verde Action Request was introduced, and senior managers were assigned to determine which actions were required in order to improve the consistency of problem treatment. Problems involving operability questions were getting to control room operators more consistently, but NRC inspectors continued to identify operability concerns that were missed by the licensee. However, having the Action Request Review Committee review all problem reports created a bottleneck in the process, creating delays in getting problems from the identification to a working stage. Problems continue to exist in the quality of problem description and significance determination. The timeliness of problem cause evaluations were improving due to management attention, but were still several times longer than station goals and industry standards.

Palo Verde Nuclear Generating Station continued to have a large number of latent equipment issues. Numerous longstanding material conditions exist which have received limited assessments and get added to the backlog with routine priority. The NRC continued to identify examples where the significance was underestimated by the licensee and were not being addressed with the timeliness commensurate with the actual safety significance until the NRC gets involved.

The team noted that significant challenges have been created because there are large backlogs of work affecting work control, maintenance support, and a variety of engineering activities. These backlogs are affecting the site's ability to address problems in a timely manner. It is apparent that these backlogs have built up over a period of years with the knowledge of management.

The Nuclear Assurance Department was active in the internal oversight role and focused on current performance problems, issuing reports that provided useful assessments. Other self-assessments reviewed were frequently narrow in scope and of limited depth. Interviews with site workers indicated that a safety-conscious work environment exists at Palo Verde Nuclear Generating Station, and that workers had an improved confidence in the strength of the safety culture. However, there was less confidence that routine priority issues will get addressed in a timely manner.

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