

Byron 2

4Q/2006 Plant Inspection Findings

Initiating Events

Mitigating Systems

Significance:  Dec 01, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Non-Conservative Errors in Unverified Design Input Data Used to Determine the Impact of Core Power Uprate on Medium Voltage Loads

Green. The team identified a NCV of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety significance. The power uprate electrical loading calculation used incorrect design input for the 4160 Vac engineered safety features (ESF) distribution system load analysis. Specifically, the licensee's contract engineering organization failed to adequately verify design input data used to determine brake horsepower loading. The incorrect horsepower values were subsequently used in revising the 4160 Vac ESF distribution system power analysis. The licensee's acceptance review did not identify the problem. Using corrected values, the licensee determined that the reduction in load margin was acceptable based on a revised loading calculation prepared informally during the inspection.

The finding was more than minor because failing to correctly identify, verify, and input the correct design data into the electrical power analysis program resulted in the load conditions not being adequately evaluated, resulting in inaccurate and non-conservative determination of loading and load margin for the 4160 Vac ESF buses. The finding was of very low safety significance based on the results of the licensee's analysis and screened as Green using the SDP Phase 1 screening worksheet.

Inspection Report# : [2006009](#) (*pdf*)

Significance:  Dec 01, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Acceptance Criteria for Safety Injection Pump NPSH Not Met

Green. The team identified a NCV of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety significance. Specifically, the calculation for evaluating the net positive suction head (NPSH) for the safety injection pump contained errors and failed to demonstrate that the acceptance criteria was met. To demonstrate operability, the licensee performed a preliminary calculation, using a less conservative pump flow value along with correcting the identified errors.

The finding was more than minor because the calculation of record was not adequate and failed to demonstrate that the NPSH available met design basis requirements. The finding was of very low safety significance based on the results of the licensee's corrected analysis and screened as Green using the SDP Phase 1 screening worksheet. The cause of the finding was related to the cross-cutting aspect of human performance.

Inspection Report# : [2006009](#) (*pdf*)

Significance:  Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO MAINTAIN FIRE BARRIERS IN ACCORDANCE WITH FIRE PROTECTION PROGRAM.

The inspectors identified a Non-Cited Violation of Byron Facility Operating License Nos. NPF-37 and NPF-66, Condition 2.c.6, for failing to maintain the firewall separating the Auxiliary Building from the penetration area in accordance with the

approved fire protection program. Fire seals were required to be provided in this firewall, except where an evaluation had been performed and approved to allow a deviation. Two sleeves containing fire seals had pulling ropes embedded in the fire seals in the firewall separating the Auxiliary Building General Area 401 from the Unit 1 piping penetration area; also, no evaluation or exemption existed to justify this configuration. The licensee entered the issue into its corrective action program for resolution and implemented compensatory measures that included hourly fire watches.

This finding was more than minor because it affected the Mitigating Systems Cornerstone objective to ensure that external factors (i.e., fire, flood, etc) do not impact the availability, reliability, and capability of systems that respond to initiating events. The finding was of very low safety significance because the fire seals were in small diameter sleeves that traveled a distance of 45 feet and had two 90 degree bends and the location of combustibles were positioned such that the piping penetration end of the fire seals would not be subject to direct flame impingement.

Inspection Report# : [2006004](#) (*pdf*)

Significance:  Mar 24, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Translate Design Basis Into Procedures for Service Water Flow to the CC Heat Exchangers

The inspectors identified a Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety significance for the licensee's failure to correctly translate the design basis into procedures. Specifically, the licensee failed to update operator rounds to verify the revised design basis minimum value for essential service water flow to the component cooling water (CC) heat exchangers. In addition, because the operator rounds were not revised, the design basis minimum flow value was not bounded by the emergency operating procedure used for establishing initial cold leg recirculation in the event of a loss of coolant accident (LOCA). This issue was entered into the licensee's corrective action program to revise the operator rounds.

The issue was more than minor because it was associated with the Mitigating System cornerstone attribute of "Design Control," and affected the cornerstone objective of ensuring the capability of systems that respond to initiating events to prevent undesirable consequences. Specifically, the failure to have operator rounds verify the design basis minimum service water flow or to have the emergency operating procedures ensure the minimum flow prior to establishing initial cold leg recirculation in the event of a LOCA could potentially have allowed the service water flow to be less than the required value to maintain the design heat load during post LOCA conditions. This finding was of very low safety significance because it screened out as Green using the SDP Phase 1 worksheet. Even though the licensee did not control their bounding design basis service water flow procedurally, the flow to the CC heat exchangers has historically been well above the bounding design basis flow.

Inspection Report# : [2006006](#) (*pdf*)

Significance:  Mar 24, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Reduction of Fire Suppression Capacity and Capability

The inspectors identified a Non-Cited Violation of 10 CFR 50.48(a)(1) having very low safety significance for the licensee's failure to provide fire fighting systems of appropriate capacity and capability to minimize the adverse effects of fires on structures, systems, and components important to safety. Specifically, the licensee abandoned standpipes and manual hose stations located near safety-related equipment (essential service water makeup pumps) which reduced the fire suppression capacity and capability to protect such equipment. In addition, the site relied on a local fire department instead of the site fire brigade to manually suppress a fire that could have affected safety-related equipment. This issue was entered into the licensee's corrective action program, and compensatory measures were taken to place dry chemical fire extinguishers in the vicinity of the fire area to take the place of the abandoned manual fire hose stations.

This finding was considered more than minor because it was associated with the Mitigating System cornerstone attribute of "Protection Against External Factors," and affected the cornerstone objective of ensuring the availability of systems that respond to initiating events to prevent undesirable consequences. Specifically, removing the manual hose stations reduced the fire suppression capacity and capability for protecting the emergency service water cooling tower makeup pumps and their diesels in the event of a fire. This finding was determined to be of very low safety significance (Green) based on a

Phase 3 SDP evaluation.
Inspection Report# : [2006006](#) (pdf)

Barrier Integrity

Significance:  Mar 31, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

DEGRADED INCORRECT PLANT BARRIER IMPAIRMENT EVALUATION RESULTED IN AN AUXILIARY BUILDING INTEGRITY

A finding of very low safety significance and associated Non-Cited Violation (NCV) of Technical Specification 5.4.1, regarding procedure adherence was inspector identified when the inspectors identified that ventilation barrier requirements were not being met during a routine assessment of work activities in the Unit 2 containment spray pump rooms. Upon identification, the licensee restored the barrier. The primary cause of this violation was related to the cross-cutting area of Human Performance.

This finding was more than minor because it affected the barrier integrity objective to provide reasonable assurance that the physical design barriers to protect the public from radionuclide releases caused by accidents or events. The finding was determined to be of very low safety significance because the issue only represented a degradation of the radiological barrier function provided for the auxiliary building.

Inspection Report# : [2006002](#) (pdf)

Emergency Preparedness

Occupational Radiation Safety

Significance:  Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO POST AND CONTROL A HIGH RADIATION AREA.

An inspector-identified finding of very low safety significance and two associated Non-Cited Violations of NRC requirements were identified for the failure to post and control access to High Radiation Areas, as required by 10 CFR Part 20, to notify individuals of the radiological hazard present and to prevent the unauthorized entry to such areas. Specifically, the entrance to the Unit 1 Filter Valve Aisle located on the 383' elevation of the Auxiliary Building, a high radiation area with a radiation dose rate of approximately 135 millirem in one hour, was not posted or controlled by any of the methods described in 10 CFR 20.1902, 10 CFR 20.1601, or Technical Specification 5.7.1.

The issue was more than minor because the issue was associated with the Program/Process attribute of the Occupational Radiation Safety Cornerstone and affected the cornerstone objective to ensure adequate protection of the worker health and safety from exposure to radiation from radioactive material during routine civilian nuclear reactor operation. The issue represents a finding of very low safety significance because the finding did not constitute an ALARA or work control issue, did not result in an overexposure or the substantial potential for an overexposure, and did not compromise the licensee's ability to assess dose. Non-Cited Violations of 10 CFR 20.1902 and 10 CFR 20.1601 were identified for the failure to post and control access to high radiation areas. Corrective actions taken by the licensee for this finding included establishing control through postings and barricades. The cause of this finding is related to the cross-cutting element of human performance.

Inspection Report# : [2006004](#) (pdf)

Public Radiation Safety

Significance:  Sep 30, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO EVALUATE THE POTENTIAL RADIOLOGICAL HAZARD ASSOCIATED WITH THE LEAKAGE OF WATER FROM THE VACUUM BREAKER VALVE VAULT.

An inspector-identified finding of very low safety significance and an associated Non-Cited Violation of NRC requirements were identified for the failure to perform surveys that are necessary to comply with the regulations in 10 CFR Part 20 and that are reasonable under the circumstances to evaluate the extent of radiation levels, concentrations or quantities of radioactive materials, and the potential radiological hazards that could be present prior to pumping liquids from blowdown line vacuum breaker valve vaults to the environment. Specifically, the conditions found at 0CW276 (vault No. 6) on July 7, 2005, were outside the parameters of the original assessment, and the licensee did not evaluate the change of conditions for the potential radiological hazards to ensure compliance with 10 CFR 20.1301, which limits radiation exposure to a member of the public to 0.1 rem.

The issue was more than minor because the issue was associated with the Program/Process attribute of the Public Radiation Safety Cornerstone and affected the cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain as a result of routine civilian nuclear reactor operation. Since the releases were limited to licensee owned property, the licensee has not measured any licensed material beyond its property line, and the licensee's REMP has a monitoring well in the vicinity of the blowdown lines, the finding did not represent a failure to assess dose nor a failure to assess the environmental impact. Consequently, the finding was determined to be of very low safety significance. A Non-Cited Violation of 10 CFR 20.1501 was identified for the failure to make surveys to ensure compliance with 10 CFR 20.1301, which limits radiation exposure to a member of the public to 0.1 rem. Corrective actions taken by the licensee for this finding included performing surveys of the soil surrounding the vacuum breaker vault for radionuclides, establishing additional groundwater monitoring wells, sealing the vacuum breaker vaults, and installing of an automated leak detection system. The cause of this finding is related to the cross-cutting element of problem identification and resolution.

Inspection Report# : [2006004](#) (*pdf*)

Physical Protection

[Physical Protection](#) information not publicly available.

Miscellaneous

Last modified : March 01, 2007