

D.C. Cook 2

2Q/2006 Plant Inspection Findings

Initiating Events

G**Significance:** Jun 29, 2006

Identified By: NRC

Item Type: FIN Finding

Inadequate Preventive/Corrective Maintenance on Turbine Building Sump Overflow Check Valve 12-DR-129

The inspectors identified a finding of very low safety significance. The licensee failed to perform adequate preventive and corrective maintenance on Turbine Building sump overflow check valve 12-DR-129. As a result, the valve was found in a significantly degraded condition such that it would not function to mitigate the consequences of a design basis seiche event. No violation of regulatory requirements was identified. Immediate corrective actions to address this finding included replacing the check valve and implementing a preventive maintenance activity to ensure that it would function.

This finding was of more than minor significance because it was associated with the Protection Against External Factors attribute of the Initiating Events cornerstone and adversely affected the cornerstone objective of limiting the likelihood of events that upset plant stability and challenge critical safety functions during power operations since inadequate preventive and corrective maintenance led to the significantly degraded condition of 12-DR-129. Although this issue affected the ability of the check valve to mitigate the consequences of a design basis seiche event, the Regional Senior Reactor Analyst determined that this finding was of very low safety significance during a Phase 3 Significance Determination Process evaluation because considering the seiche initiating event frequency, the change in core damage frequency for this finding was calculated to be well below $1.0E-6$. This finding affected the cross-cutting area of problem identification and resolution because the licensee failed to identify and correct the degraded valve condition. Corrective actions that were taken were not timely, were not commensurate with the significance of the issue, and early corrective actions were ineffective.

Inspection Report# : [2006004\(pdf\)](#)G**Significance:** Jun 29, 2006

Identified By: Self-Revealing

Item Type: NCV NonCited Violation

Inadvertent Loss of Reactor Coolant System Inventory While Placing Emergency Core Cooling Systems in Standby Readiness

A finding of very low safety significance with an associated Non-Cited Violation of 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures and Drawings" was self-revealed. With Unit 2 in Mode 4 (Hot Shutdown), using an inadequate procedure, plant operators performed procedural steps to vent the residual heat removal system piping while the system was still connected to the reactor coolant system (RCS). As a result, the charging pump suction safety valve (2-SV-56) unexpectedly lifted and discharged approximately 120 gallons of water to the pressurizer relief tank. Corrective actions included revising the procedures for placing emergency core cooling systems in standby readiness and an engineering evaluation was completed to ensure that the charging pump suction header piping did not exceed its design pressure.

This finding was of more than minor significance because it was related to the Procedure Adequacy attribute of the Initiating Events cornerstone, and adversely impacted the cornerstone objective to limit the likelihood of events that upset plant stability and challenge critical safety functions during shutdown operations. Specifically, the finding resulted in an unintended loss of RCS inventory with the plant shut down in Mode 4. The finding was not greater than Green because adequate mitigation capabilities were maintained, and the finding did not represent a loss of control in that less than 2 feet of RCS inventory was lost from the pressurizer. The primary cause of this finding was related to the cross-cutting area of human performance because the procedure that was used was not complete.

Inspection Report# : [2006004\(pdf\)](#)G**Significance:** Nov 18, 2005

Identified By: Self-Revealing

Item Type: FIN Finding

Inadequate Preventive Maintenance on Main Generator Exciter Resulted in a Reactor Trip

The inspectors identified a finding of very low safety significance associated with a self-revealed event. The licensee failed to perform adequate preventive maintenance on the Unit 2 main generator exciter, which led to brush failures, loss of field excitation, and a reactor trip. No violation of regulatory requirements was identified. Immediate corrective actions to address this finding included the replacement of brushes and brush holders on the Unit 2 main generator exciter, repairs to the Unit 2 main generator exciter slip ring, and verification of proper brush installation on both the Unit 1 and Unit 2 main generators and main generator exciters.

This finding was of more than minor safety significance because it was associated with the Equipment Performance attribute of the Initiating Events cornerstone and adversely affected the cornerstone objective of limiting the likelihood of events that upset plant stability and challenge critical safety functions during power operations since inadequate preventive maintenance led to the main generator exciter brush failures that caused the reactor trip. Although the event contributed to the likelihood of a reactor trip, the finding is of very low significance because all mitigation systems

were available. This finding affected the cross-cutting issue of human performance (resources).
Inspection Report# : [2005013\(pdf\)](#)

Mitigating Systems

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Significance: Jun 29, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Potential External and Internal Flooding Impact on Safe Shutdown Equipment in the Lake Screen House

The inspectors identified a finding of very low safety significance with an associated Non-Cited Violation of 10 CFR 50, Appendix B, Criteria III, "Design Control." The licensee failed to correctly translate the design basis into specifications for the essential service water (ESW) system by ensuring that ESW system components in the Lake Screen House would be protected to the 595' elevation as described in Section 10.6 of the Updated Final Safety Analysis Report, in the event of flooding due to a design basis seiche event. The licensee was evaluating corrective actions for this issue at the end of the inspection period. No immediate actions were necessary due to the present low lake level.

This finding was of more than minor significance because it was associated with the Design Control attribute of the Mitigating Systems cornerstone and adversely affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences since the failure to maintain adequate design control for the affected ESW system components in the Lake Screen House could possibly have resulted in damage to safe shutdown plant equipment during a design basis seiche event. The finding was of very low safety significance because it was a design or qualification deficiency confirmed not to result in loss of operability.

Inspection Report# : [2006004\(pdf\)](#)

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Significance: Jun 29, 2006

Identified By: NRC

Item Type: FIN Finding

Inadequate Functionality Evaluation for Degraded Check Valve Condition

The inspectors identified a finding of very low safety significance. The licensee did not adequately evaluate the functionality of Turbine Building sump overflow check valve 12-DR-129, while the valve was in a significantly degraded condition such that it would not function to mitigate the consequences of a design basis seiche event. No violation of regulatory requirements was identified. Immediate corrective actions to address this finding included a detailed calculation to determine the potential for flooding in the emergency diesel generator (EDG) rooms to support a past operability evaluation for the EDGs.

This finding was of more than minor significance because if left uncorrected, the failure to properly evaluate the functionality of equipment important to safety could result in incorrectly concluding that the equipment was functional. The inspectors determined that this finding was related to the Protection Against External Factors attribute of the Mitigating Systems cornerstone and adversely impacted the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences. Consistent with the Phase 3 Significance Determination Process evaluation performed in Section 1R06.b.2, this finding was determined to be of very low safety significance. This finding affected the cross-cutting area of problem identification and resolution because the licensee did not apply appropriate rigor and detail to its evaluation of the non-functional check valve; and as a result, the potential impact on safe shutdown equipment was not evaluated and timely corrective actions were not taken.

Inspection Report# : [2006004\(pdf\)](#)

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Significance: Jun 29, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Establish Appropriate Technical Specification Surveillance Acceptance Criteria for the Emergency Diesel Generators

The inspectors identified a finding of very low safety significance with an associated Non-Cited Violation of 10 CFR 50, Appendix B, Criteria III, "Design Control." The licensee failed to establish appropriate Technical Specification (TS) surveillance acceptance criteria for full load rejection testing of the emergency diesel generators with its implementation of Improved Standard Technical Specifications. An emergency TS amendment was required to revise the acceptance criteria.

This finding was of more than minor significance based on programmatic concerns identified with the issue that could lead to worse errors if not corrected. This finding was not suitable for an evaluation using the Significance Determination Process, but has been reviewed by NRC management and was determined to be a of very low safety significance. The finding was determined not to be greater than Green because there was no actual adverse impact to plant equipment. This finding affected the cross-cutting area of human performance because the licensee did not apply appropriate rigor and detail to its evaluation of the new TS surveillance acceptance criteria; and as a result, the engines could not meet the criteria when tested.

Inspection Report# : [2006004\(pdf\)](#)

Significance: SL-IV Jun 21, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Inappropriate Deletion of Technical Requirements Manual Sections

The inspectors identified a Severity Level IV Non-Cited Violation of 10 CFR 50.59(d)(1) for the licensee's failure to perform a safety evaluation for the deletion of four sections of the Technical Requirements Manual. Specifically, the licensee deleted Sections 8.4.7, Tavg Lower Limit, 8.6.1, Ice Bed Temperature Monitoring System, and 8.6.2, Inlet Door Position Monitoring System, and 8.3.7, Post Accident Monitoring (PAM) Instrumentation, Table 8.3.7-1 without evaluating these changes per the requirements of 10 CFR 50.59.

Because the issue potentially impacted the NRC's ability to perform its regulatory function, this finding was evaluated using the traditional enforcement process. The finding was determined to be more than minor because the inspectors, at the time of the inspection, could not reasonably determine that the Updated Final Safety Analysis Report change, which adversely affected equipment important to safety, would not have ultimately required NRC approval. The inspectors completed a significance determination of the underlying technical issue using NRC's inspection manual chapter 0609, Appendix A, "Significance Determination of Reactor Inspection Findings for At-Power Situations," and answered "no" to the Mitigating Systems screening questions in the Phase 1 Screening Worksheet. Specifically, even though these TRM sections along with their associated surveillance requirements were deleted, the licensee was able to show that all deleted surveillance requirements had been performed satisfactorily and within their prescribed frequency in spite of the deletion. This issue was entered into the licensee's corrective action program. Inspection Report# : [2006009\(pdf\)](#)

Significance: SL-IV Jun 21, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform 10 CFR 50.59 Evaluation for Modification to the 2-East Centrifugal Charging Pump

The inspectors identified a Severity Level IV Non-Cited Violation of 10 CFR 50.59(d)(1) for the licensee's failure to perform a safety evaluation for the modification of the 2-East Centrifugal Charging Pump (CCP). Specifically, the licensee performed modifications to the 2-East Centrifugal Charging Pump that required more restrictive frequency requirements to be established than were already in the Technical Specifications. Had a 10 CFR 50.59 evaluation been performed, as required, the evaluation should have shown that a change to the Technical Specifications (TS) was required so that the new required frequency value could be incorporated into the applicable TS Surveillance Requirements. This issue was entered into the licensee's corrective action program.

Because the issue potentially impacted the NRC's ability to perform its regulatory function, this finding was evaluated using the traditional enforcement process. The finding was determined to be more than minor because the inspectors could not reasonably determine that the modification of the 2-East Centrifugal Charging Pump would not have ultimately required NRC approval. The inspectors evaluated the finding using IMC 0609, Appendix A, Phase 1 screening for the mitigating systems cornerstone and determined that the finding was of very low safety significance because they were able to answer "no" to the Mitigating Systems screening questions in the Phase 1 Screening Worksheet. Specifically, while the 10 CFR 50.59 evaluation, and ultimately the required license amendment, were not performed as required, administrative controls were put into place after the modification was performed such that the CCP would always be able to perform its function. Inspection Report# : [2006009\(pdf\)](#)



Significance: Jun 21, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Non-Conservative Verification of Containment Average Air Temperature

The inspections identified a Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," that was of very low safety significance. Specifically, verification of containment lower compartment average temperature per Surveillance Requirement (SR) 3.6.5.2 was being performed using temperature readings that were not representative (and non-conservative) of the true average temperature in the lower containment. The issue was entered into the licensee's corrective action program.

The issue was more than minor because it was associated with the Mitigating System Cornerstone attribute of "Design Control," and affected the cornerstone objective of ensuring the capability of systems that respond to initiating events to prevent undesirable consequences. Specifically, the methodology for determining lower containment average temperature was non-conservative and did not account for the heightened temperatures that were experienced in the Steam Generator (SG) Enclosure Rooms. Had average temperature been above the TS limits, temperatures during a Design Basis Accident could have exceeded the ratings of safety related mitigating equipment thereby challenging the functionality of the equipment. This finding was of very low safety significance, because the inspectors answered "no" to all five questions under the Mitigating Systems Cornerstone column of the Phase 1 worksheet. Specifically, after performing a calculation that included the SG Enclosure Rooms, the licensee determined that under worst case historical conditions, average air temperature was 119.5 degrees which was still less than the TS requirement of 120 degrees F. Inspection Report# : [2006009\(pdf\)](#)



Significance: Nov 18, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform Adequate Post Maintenance Testing, Resulting in a TS 3.8.1 Violation

The inspectors identified a performance deficiency that resulted in a Non-Cited Violation of Technical Specification 3.8.1, with two examples. The licensee failed to perform adequate post maintenance testing after installing a design modification, which resulted in one of the two Unit 2 AB emergency diesel generator (EDG) output breakers (breaker T21B4 supply to bus T21B) failure to automatically close on demand. The Unit 2 AB

EDG was rendered inoperable due to the T21B4 breaker malfunction and this resulted in two examples of exceeding Technical Specification allowed outage times. Immediate corrective actions to address this finding included replacing an incorrectly installed wire lug on a test switch connection and completing additional wiring inspections.

This finding was of more than a minor safety significance because it was associated with the Equipment Performance attribute of the Mitigating Systems cornerstone and affected the cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events to prevent undesirable consequences since the Unit 2 AB EDG was rendered inoperable, in particular breaker T21B4, for an extended period of time. Although this issue affected the capability of the EDG to provide power to bus T21B following a loss of offsite power event, the Regional Senior Reactor Analyst determined that this finding was of very low safety significance during a Phase 3 Significance Determination Process evaluation because the effect of the unavailability of bus T21B on overall plant risk was not significant.

Inspection Report# : [2005013\(pdf\)](#)

Significance: SL-IV Sep 30, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Introduction of Manual Action in Station Blackout Response Procedure

A finding of very low safety significance was identified by the inspectors associated with a non-cited violation of 10 CFR 50.59(d)(1). The issue involved an inadequate evaluation under 10 CFR 50.59 with respect to introduction of a new manual action in place of a previously automatic action. This issue was entered into the licensee's corrective action system and the licensee prepared a new evaluation in accordance with 10 CFR 50.59.

This finding was assigned a significance level of very low safety significance based on management review. The violation was categorized as Severity Level IV based on the underlying technical issue for the finding having screened out as having very low significance using the Phase 1 worksheet of Inspection Manual Chapter 0609, Appendix A, "Significance Determination of Reactor Inspection Findings for At-Power Situations." (Section 1R21.1.b)

Inspection Report# : [2005007\(pdf\)](#)

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Significance: Sep 30, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Hydrometer Not Calibrated for Temperatures Seen During Surveillances

Green. A finding of very low safety significance was identified by the inspectors associated with a non-cited violation of 10 CFR Part 50, Appendix B, Criterion XII, "Measuring and Test Equipment." Specifically, the licensee did not calibrate a digital hydrometer over all the temperature ranges under which the hydrometer was used. This issue was entered into the licensee's corrective action system and the licensee was evaluating the necessary corrective actions.

This finding was more than minor because it could lead to a more serious situation. Specifically, continued reliance on a hydrometer that was not calibrated for the temperatures at which it was being used could reasonably lead to a situation where the actual specific gravity was below the technical specification limits without that being noticed. This finding was of very low safety significance because it screened out using the Phase 1 worksheet of Inspection Manual Chapter 0609, Appendix A.

Inspection Report# : [2005007\(pdf\)](#)

G

Significance: Sep 30, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Inadequate Torquing Requirements in 250 Vdc Safety-Related Battery Procedures

Green. A finding of very low safety significance was identified by the inspectors associated with a non-cited violation of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings." Specifically, the licensee failed to ensure that adequate battery terminal connection torque values were specified in the AB, CD and N batteries maintenance and surveillance procedures. The licensee entered the issue into its corrective action system, confirmed that the N-train of safety-related batteries were correctly torqued, revised one procedure and was evaluating the additional corrective actions needed.

This finding was more than minor because the finding was associated with the attribute of equipment performance, which affected the mitigating systems cornerstone objective of ensuring the availability and reliability of the 250 VDC power system to respond to initiating events to prevent undesirable consequences. Specifically, inconsistent torquing requirements specified in maintenance and surveillance procedures used to perform maintenance activities on safety related batteries could potentially result in unacceptable battery terminal connections and render the safety-related battery incapable of performing its required safety function. This finding was of very low safety significance because it screened out using the Phase 1 worksheet of Inspection Manual Chapter 0609, Appendix A.

Inspection Report# : [2005007\(pdf\)](#)

G

Significance: Sep 30, 2005

Identified By: NRC

Item Type: NCV NonCited Violation

Single Cell Non-Class-1E Battery Charger Procedure Deficiencies

Green. A finding of very low safety significance was identified by the inspectors associated with a non-cited violation of 10 CFR Part 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings." Specifically, the licensee failed to ensure that procedure 12-IHP-5021-EMP-009 contained adequate verification such that an independent observer could ensure that adequate electrical isolation had been maintained when a non-Class 1E single cell battery charger was used to charge a single battery cell on safety-related batteries. This issue was entered into the licensee's corrective action system and the licensee was evaluating other corrective actions.

This finding was more than minor in that the finding was associated with the attribute of equipment performance, which affected the mitigating system's cornerstone objective of ensuring the availability and reliability of the DC power system to respond to initiating events to prevent undesirable consequences. Specifically, failure to install a fuse could result in inadequate electrical isolation between the non-Class 1E single cell battery charger and safety-related battery. Without adequate isolation, a fault on the non-Class 1E charger could potentially render the safety-related battery incapable of performing its required safety function. This finding was of very low safety significance because it screened out using the Phase 1 worksheet of Inspection Manual Chapter 0609, Appendix A. (Section 1R21.3.b1)

Inspection Report# : [2005007\(pdf\)](#)



Significance: G Sep 30, 2005

Identified By: NRC

Item Type: FIN Finding

Electrolytic Capacitors in Battery Chargers Not Energized Annually

Green. A finding of very low safety significance was identified by the inspectors which was not associated with a non-cited violation. Specifically, the licensee failed to ensure that each of the 250 VDC battery chargers was energized for a minimum of eight hours per year. The vendor required this minimum energization in order to ensure the electrolytic capacitors installed in the chargers would meet the qualified replacement life of 10 years. This issue was entered into the licensee's corrective action system and the licensee was evaluating other corrective actions.

This finding was more than minor because it was associated with the attribute of equipment performance, which affected the mitigating system's cornerstone objective of ensuring the availability and reliability of the DC power system to respond to initiating events to prevent undesirable consequences. Specifically, the failure to energize the electrolytic capacitors for at least 8 hours annually could lead to the degradation of the capacitors with resultant degradation of the voltage going to the batteries. This finding was of very low safety significance because it screened out using the Phase 1 worksheet of Inspection Manual Chapter 0609, Appendix A, "Significance Determination of Reactor Inspection Findings for At-Power Situations."

Inspection Report# : [2005007\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: VIO Violation

Failure to Accurately Report Completion of Corrective Actions from a Previous Severity Level III Violation

The licensee provided incomplete and inaccurate information in a letter to the NRC dated August 2, 2004. Specifically, the licensee, in its response to an apparent violation, which was subsequently issued as a Severity Level III Notice of Violation issued on September 29, 2004, incorrectly stated that: "a 100 percent review (self-assessment) of all operator medical records was performed in February and March of 2004;" and that full compliance was achieved on April 8, 2004. During an April 2005 followup review of the licensee's corrective actions for the Severity Level III violation, the NRC identified three additional examples of licensed operators with a potentially disqualifying medical condition that existed prior to the licensee's February and March 2004 review of its medical records, that had not been reported to the NRC. The licensee made changes to their administrative procedures to ensure clarity in regard to medical reporting requirements and required an annual medical file review in addition to an annual discussion with their medical review official to ensure a mutual understanding of the appropriate regulatory requirements.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated using the traditional enforcement process. The regulatory significance was important because the incorrect information was provided under a signed statement to the NRC and impacted an enforcement decision. The issue was preliminarily determined to be an apparent violation of 10 CFR 50.9.

Notice of Violation Issued November 23, 2005, ML0532902430.

The VIO was opened in NRC Inspection Report 05000315/316/2005012. Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005014\(pdf\)](#)

Inspection Report# : [2005012\(pdf\)](#)

Inspection Report# : [2005006\(pdf\)](#)

Inspection Report# : [2006003\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: VIO Violation

Failure to Provide Complete and Accurate Information about Operators' Health Status

The NRC identified that on May 5, 2004, a senior licensee representative submitted to the NRC a Form NRC - 396 to support an application for renewal of an SRO license, that was not complete and accurate in all material respects. The Form NRC - 396 certified that the applicant met the

medical requirements of ANSI/ANS 3.4-1983 and that the applicant would not require any restrictions to the individual's license. In fact, the applicant had a potentially disqualifying medical condition dating back to October 30, 1998. [Note: The information concerning the individual's specific medical condition is considered medical privacy information under 10 CFR 2.390(2)(6) and is not specifically discussed here.] The medical condition was potentially disqualifying in accordance with ANSI/ANS 3.4, 1983, and required that the individual's license be amended to include an operating restriction. The information is material to the NRC because the NRC relies on this certification to determine whether the applicant meets the requirements to operate the controls of a nuclear power plant pursuant to 10 CFR Part 55. The licensee made changes to their administrative procedures to ensure clarity in regard to medical reporting requirements and required an annual medical file review in addition to an annual discussion with their medical review official to ensure a mutual understanding of the appropriate regulatory requirements.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated using the traditional enforcement process. The regulatory significance was important because the incorrect information was provided under a signed statement to the NRC and impacted a licensing decision for the individual. The issue was preliminarily determined to be an apparent violation of 10 CFR 50.9.

Notice of Violation Issued November 23, 2005, ML 0532902430.

The VIO was opened in NRC Inspection Report 05000315/316/2005012. Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005006\(pdf\)](#)

Inspection Report# : [2005012\(pdf\)](#)

Inspection Report# : [2005014\(pdf\)](#)

Inspection Report# : [2006003\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: VIO Violation

Failure to Report a Change in Operator Medical Status

The NRC identified that from November 29, 1998, until May 18, 2005, the licensee did not report the change in medical status of an SRO that acquired a potentially disqualifying medical condition as required by 10 CFR 55.25. The medical condition was potentially disqualifying in accordance with ANSI/ANS 3.4, 1983, and required that the individual's license be amended when it was finally reported on May 18, 2005, to include an operating restriction. [Note: The information concerning the individual's specific medical condition is considered medical privacy information under 10 CFR 2.390(2)(6) and is not specifically discussed here.] The issue was more than minor because the NRC relies on this certification to determine whether the applicant meets the requirements to operate the controls of a nuclear power plant pursuant to 10 CFR Part 55. The licensee made changes to their administrative procedures to ensure clarity in regard to medical reporting requirements and required an annual medical file review in addition to an annual discussion with their medical review official to ensure a mutual understanding of the appropriate regulatory requirements.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated with the traditional enforcement process. The regulatory significance was important because information was not provided that would have resulted in a licensing decision for the individual. The issue was preliminarily determined to be an apparent violation of 10 CFR 55.25.

Notice of Violation Issued November 23, 2005, ML0532902430.

The VIO was opened in NRC Inspection Report 05000315/316/2005012. Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005012\(pdf\)](#)

Inspection Report# : [2005006\(pdf\)](#)

Inspection Report# : [2006003\(pdf\)](#)

Inspection Report# : [2005014\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: AV Apparent Violation

Failure to Provide Complete and Accurate Information to the NRC Which Impacted A Licensing Decision

On April 26, 2004, a senior licensee representative submitted to the NRC a Form NRC - 396 to support an amendment request of a Senior Reactor Operator (SRO) license, that was not complete and accurate in all material respects. The Form NRC - 396 certified that the applicant met the medical requirements of ANSI/ANS 3.4-1983 and that the applicant would not require any restrictions to the individual's license. In fact, the applicant had a potentially disqualifying medical condition dating back to 2003. The medical condition was potentially disqualifying in accordance with ANSI/ANS 3.4-1983, and required that the individual's license be amended to include an operating restriction. [Note: The information concerning the individual's specific medical condition is considered medical privacy information under 10 CFR 2.390(2)(6) and is not specifically discussed here.] The issue was more than minor because the NRC relies on this certification to determine whether the applicant meets the requirements to operate the controls of a nuclear power plant pursuant to 10 CFR Part 55. The licensee made changes to their administrative procedures to ensure clarity in regard to medical reporting requirements and required an annual medical file review in addition to an annual

discussion with their medical review official to ensure a mutual understanding of the appropriate regulatory requirements. Since NRC intervention was required to identify the requirement for the operator to have a license restriction, this issue was considered NRC-identified.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated with the traditional enforcement process. The regulatory significance was important because the incorrect information was provided under a signed statement to the NRC and impacted a licensing decision for the individual. The issue was preliminarily determined to be an apparent violation of 10 CFR 50.9.

Apparent violation AV closed in NRC Inspection Report 05000315/316/2005012. Notice of Violation Issued November 23, 2005, ML0532902430.

Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005006\(pdf\)](#)

Inspection Report# : [2005014\(pdf\)](#)

Inspection Report# : [2005012\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: AV Apparent Violation

Failure to Report A Change In A License Operators Medical Condition

The NRC identified that from January 6, 2003, until May 18, 2005, the licensee did not report the change in medical status of an SRO that acquired a potentially disqualifying medical condition as required by 10 CFR 55.25. The medical condition was potentially disqualifying in accordance with ANSI/ANS 3.4, 1983, and required that the individual's license be amended when it was finally reported on May 18, 2005, to include an operating restriction. [Note: The information concerning the individual's specific medical condition is considered medical privacy information under 10 CFR 2.390(2)(6) and is not specifically discussed here.] The issue was more than minor because the NRC relies on this certification to determine whether the applicant meets the requirements to operate the controls of a nuclear power plant pursuant to 10 CFR Part 55. The apparent violation was determined to be of significant regulatory concern because a licensing action was not taken because information was not provided by the licensee. Since NRC intervention was required to identify the requirement for the operator to have a license restriction, this issue was considered NRC identified.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated with the traditional enforcement process. The regulatory significance was important because information was not provided that would have affected a licensing decision for the individual. The issue was preliminarily determined to be an apparent violation of 10 CFR 55.25.

Apparent violation AV closed in NRC Inspection Report 05000315/316/2005012. Notice of Violation Issued November 23, 2005, ML0532902430.

Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005006\(pdf\)](#)

Inspection Report# : [2005012\(pdf\)](#)

Inspection Report# : [2005014\(pdf\)](#)

Significance: SL-III Aug 31, 2005

Identified By: NRC

Item Type: AV Apparent Violation

Failure to Provide Complete and Accurate Information to the NRC Which Impacted A Licensing Decision

On November 4, 2002, a senior licensee representative submitted to the NRC a Form NRC - 396 to support an application for an SRO license, that was not complete and accurate in all material respects. The Form NRC - 396 certified that the applicant met the medical requirements of ANSI/ANS 3.4-1983 and that the applicant would not require any restrictions to the individual's license. In fact, the applicant had a potentially disqualifying medical condition in accordance with ANSI/ANS 3.4-1983. The medical condition required that the individual's license be amended to include an operating restriction. [Note: The information concerning the individual's specific medical condition is considered medical privacy information under 10 CFR 2.390(2)(6) and is not specifically discussed here.] The issue is more than minor because the NRC relies on this certification to determine whether the applicant meets the requirements to operate the controls of a nuclear power plant pursuant to 10 CFR Part 55. The licensee made changes to their administrative procedures to ensure clarity in regard to medical reporting requirements and required an annual medical file review in addition to an annual discussion with their medical review official to ensure a mutual understanding of the appropriate regulatory requirements. Since NRC intervention was required to identify the requirement for the operator to have a license restriction prior to his initial license being issued, this issue was considered NRC-identified.

Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated with the traditional enforcement process. The regulatory significance was important because the incorrect information was provided under a signed statement to the NRC and impacted a licensing decision for the individual. The issue was preliminarily determined to be an apparent violation of 10 CFR 50.9.

Apparent violation AV closed in NRC Inspection Report 05000315/316/2005012. Notice of Violation Issued November 23 2005, ML0532902430.

Apparent violation AV 05000315/316/2005006-01 is updated to VIO 05000315/316/2005006-01 (Failure of the licensee to accurately report the completion of corrective actions from a previous SLIII violation in 2004.); AV 05000315/316/2005006-02, 04, and 06 are updated to VIO 05000315/316/2005006-02 (Failure to provide accurate and complete information about operators' health prior to the NRC performing a licensing action.); and AV 05000315/316/2005006-03, and 05 are updated to VIO 05000315/316/2005006-03 (Failure of the licensee to report the change in operator medical status.).

Inspection Report# : [2005014\(pdf\)](#)

Inspection Report# : [2005012\(pdf\)](#)

Inspection Report# : [2005006\(pdf\)](#)

Barrier Integrity



Significance: Jun 29, 2006

Identified By: Self-Revealing

Item Type: FIN Finding

Heavy Load Dropped While Removing Vertical Bulkhead Blocks in Unit 2 Containment

A finding of very low safety significance was self-revealed, when the lift rig device failed and a 37 ton vertical bulkhead block dropped approximately 15 feet inside Unit 2 containment with the plant in Mode 5 (Cold Shutdown). The plant procedure utilized did not require a load cell while lifting the vertical bulkhead blocks and therefore adequate detection of load binding was not provided. Consequently, load binding during the lift was not detected and the lift rig assembly was overloaded and failed.

This finding was of more than minor significance because if left uncorrected, this issue could lead to a more significant safety concern in that a dropped heavy load could impact and adversely affect plant safety-related structures, systems or components. This finding was not suitable for an evaluation using the Significance Determination Process, but has been reviewed by NRC management and was determined to be a findings of very low safety significance. This finding was not greater than Green because no adverse consequences to plant safety-related or risk significant structures systems or components resulted from the dropped load. The primary cause of this finding was related to the cross-cutting area of human performance because the procedure that was used was not complete.

Inspection Report# : [2006004\(pdf\)](#)



Significance: Jun 29, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform As-found Local Leak Rate Testing for a Containment Isolation Valve

The inspectors identified a finding of very low safety significance with an associated Non-Cited Violation of TS Surveillance Requirement 3.6.1.1. The licensee failed to perform an as-found local leak rate test (LLRT) for containment isolation valve 2-SI-189 (emergency core cooling system safety valves discharge to the primary relief tank containment isolation check valve) prior to performing maintenance that affected the valve's leak tightness as required by the plant's TSs. Immediate corrective actions to address this finding were to revise the planning and scheduling activities for testing this valve.

This finding was of more than minor significance because it was associated with the SSC [Structure, System and Component] and Barrier Performance attribute of the Barrier Integrity cornerstone and adversely affected the cornerstone objective of providing reasonable assurance that the physical design barriers (e.g., containment) protect the public from radio-nuclide releases caused by accidents or events since the true as-found condition of 2-SI-189 for the previous operating cycle was unknown and could not be evaluated. This finding was of very low safety significance because Unit 2 was defueled at the time and containment integrity was not required. This finding affected the cross-cutting area of human performance because the licensee failed to properly sequence the valve's visual inspection activity after the as-found LLRT into its scheduling process.

Inspection Report# : [2006004\(pdf\)](#)

Significance: SL-IV Jun 29, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Submit a Required Licensee Event Report

The inspectors identified a Non-Cited Violation of 10 CFR 50.73(a)(1) because the licensee failed to submit a required Licensee Event Report within 60 days after discovery of an event requiring a report. The event involved the licensee's failure to meet Containment Leakage Rate Testing Program requirements in accordance with Technical Specification (TS) Surveillance Requirement 3.6.1.1, a condition prohibited by the plant's TSs. No immediate corrective actions were taken to address this finding; however, the issue was entered into the licensee's corrective action program.

This finding was of more than minor significance because the NRC relies on licensee's to identify and report conditions or events meeting the criteria specified in the TSs and the regulations to perform its regulatory function. Because the issue affected the NRC's ability to perform its regulatory function, it was evaluated with the traditional enforcement process. Consistent with the guidance in Section 7.10 and Supplement I, Paragraph D.4 of the NRC Enforcement Policy, this issue was determined to be a Severity Level IV violation. This finding affected the cross-cutting area of problem identification and

resolution because the licensee incorrectly concluded that the failure to perform an as-found LLRT for containment isolation valve 2-SI-189 was not a condition prohibited by the plant's TSs.

Inspection Report# : [2006004\(pdf\)](#)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Significance:  Mar 31, 2006

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Perform Adequate Checks of the Automatic Gas Analyzer System Oxygen Monitor Channel

An inspector-identified finding of very low safety significance and an associated violation of NRC requirements were identified for the failure to perform adequate daily checks for the in-service oxygen monitor channel of the automatic gas analyzer system, as required by Technical Specifications.

The issue was more than minor because if left uncorrected the issue could become a more significant safety concern, since this monitor provides early indication of a potential explosive gas mixture in the waste gas decay system. The issue represents a finding of very low safety significance because alternate methods were available to assess the potential for an explosive gas mixture in the waste decay system, and, therefore, there was minimal actual risk to the public. A Non-Cited Violation of Technical Specification Surveillance Requirement 4.3.3.9 was identified for the failure to perform adequate daily checks for the in service oxygen monitor channel of the automatic gas analyzer system. Corrective actions planned by the licensee for this finding include enhancing the applicable procedure that governs the daily check of the oxygen monitor channel of the automatic gas analyzer to provide more specific direction to plant staff on equipment acceptance criteria.

Inspection Report# : [2006003\(pdf\)](#)

Physical Protection

[Physical Protection](#) information not publicly available.

Miscellaneous

Last modified : August 25, 2006