

Summer 3Q/2004 Plant Inspection Findings

Initiating Events

G**Significance:** Jun 26, 2004

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Inadequate Corrective Action Associated with Weld Repairs on C RCP Seal Injection Line

A self-revealing non-cited violation (NCV) regarding inadequate corrective action associated with weld repairs on the C reactor coolant pump seal injection line was identified.

This finding was a violation of 10 CFR 50, Appendix B, Criterion XVI, Corrective Action. This finding is more than minor because it affected the initiating event cornerstone objective and the respective attribute of equipment performance. The finding is of very low safety significance because the axial orientation of the crack in the seal injection line did not contribute significantly to the likelihood of a primary loss of coolant accident and the likelihood of both a reactor trip and the loss of mitigating functions.

Inspection Report# : [2004003\(pdf\)](#)**G****Significance:** May 14, 2004

Identified By: NRC

Item Type: FIN Finding

Ineffective Incorporation of Operating Experience into the Corrective Action Program

A self-revealing finding was identified for ineffective incorporation of operating experience (OE) into the corrective action program. A November 2003 operating experience report had identified an issue regarding the feedwater regulating valve positioners. However, because the licensee reviewer inappropriately assumed that the positioners were being replaced every outage and that this action was sufficient, no additional actions were taken or planned. As a result of a reactor trip on March 30, 2004, the licensee performed a root cause evaluation. The licensee identified that the positioners were the root cause and that the OE information, if incorporated properly into the corrective action program, could have precluded this reactor trip.

The team determined this finding was more than minor because failing to properly screen this OE and implement corrective actions would eventually have resulted in a feedwater transient and a potential for causing a reactor trip. The finding was of very low safety significance because, although it would cause a feedwater transient/reactor trip, it did not increase the likelihood of a primary or secondary system loss of coolant accident initiator, did not contribute to a combination of a reactor trip and loss of mitigation equipment functions, and did not increase the likelihood of a fire or internal/external flood. The finding was not a violation of regulatory requirements because it involved non-safety related secondary plant equipment and procedures.

Inspection Report# : [2004006\(pdf\)](#)**G****Significance:** Oct 01, 2003

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Inadequate Preventive Maintenance Results in a Reactor Trip

A self-revealing non-cited violation was identified for inadequate preventative maintenance (PM) resulting in a reactor trip. 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures and Drawings," requires, in part, that procedures shall include appropriate qualitative acceptance criteria for determining that important activities have been satisfactorily accomplished." The licensee failed to establish an adequate Electrical Maintenance Procedure (EMP)-245.005, "Main Generator and Alterrex Refueling Preventative Maintenance," to preclude a condition that resulted in a reactor trip.

The finding is more than minor because it resulted in a reactor trip. The self-revealing finding is of very low safety significance since the event did not contribute to the likelihood of a primary or a secondary system loss of coolant accident (LOCA) initiator, did not contribute to a loss of mitigation equipment functions, and did not increase the likelihood of a fire or internal / external flood.

Inspection Report# : [2003004\(pdf\)](#)

Mitigating Systems

G**Significance:** Jun 26, 2004

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Inadequate Control of Quality Related Drawings Results in Loss of RCS Pressurizer Heater Control

A self-revealing non-cited violation (NCV) regarding inadequate control of quality related drawings resulting in the loss of reactor coolant system pressurizer heater control was identified.

This finding was a violation 10 CFR Part 50, Appendix B, Criterion VI, Document Control. This finding is more than minor because if left uncorrected it would become a more significant safety concern due to the extensive use of quality related controlled drawings in the process of maintenance involving safety-related structures, systems and components. The finding is of very low safety significance due to the brief period pressurizer heater control was lost, the availability of an alternate pressurizer heater control circuit, and no actual loss of safety function occurred.

Inspection Report# : [2004003\(pdf\)](#)**G****Significance:** Jun 25, 2004

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Provide Portable Smoke Ejectors Capable of Operation During a Loss of Offsite Electrical Power

A non-cited violation of V.C. Summer Facility Operating License NFP-12, Condition 2.C.(18), was identified for failure to provide the fire brigade with portable smoke ejectors capable of operation during a loss of offsite electrical power. The licensee acquired portable, gasoline-powered electrical generators to resolve the problem.

The finding adversely affected the defense-in-depth element for fire brigade manual fire suppression capability. The finding is greater than minor because it is associated with the protection against external factors attribute and degraded the reactor safety mitigating systems cornerstone objective. Because this finding only impacted the effectiveness of the fire brigade while other fire protection features, such as passive fire barriers, physical separation, and safe shutdown capability remained available to mitigate a fire, the finding was determined to have very low safety significance

Inspection Report# : [2004007\(pdf\)](#)

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

[Physical Protection](#) information not publicly available.

Miscellaneous

Significance: N/A May 14, 2004

Identified By: NRC

Item Type: FIN Finding

Biennial Problem Identification and Resolution Inspection Results

Overall, the licensee maintained an effective program for the identification and correction of conditions adverse to quality. However, during the inspection, several minor problems were identified. The licensee was generally effective at identifying problems at a low threshold and entering them into the Corrective Action Program (CAP). However, a few instances of failing to enter or delaying entry of issues into the CAP were identified. The licensee consistently prioritized issues in accordance with their CAP and routinely performed adequate evaluations that were technically accurate and of sufficient depth. Improvements were noted in the corrective action process since the previous problem identification and resolution inspection including increased management involvement and improved management review through the use of Management Review Team meetings. Root cause analyses were performed when appropriate and problem evaluations considered extent of condition and generic implications appropriately. Corrective actions were effective in correcting problems. However, in a few cases the licensee continued to experience problems with corrective actions for issues such as ensuring closure of the steam propagation and fire doors. Management fostered a safety-conscious work environment by emphasizing safe operations and encouraging problem reporting. However, during the inspection, the NRC identified that the licensee had narrowly focused corrective actions associated with a safety-conscious work environment issue.

Inspection Report# : [2004006\(pdf\)](#)

Last modified : December 29, 2004