Vogtle 2
1Q/2004 Plant Inspection Findings

Initiating Events

Significance:  Sep 27, 2003
Identified By: Self Disclosing
Item Type: NCV NonCited Violation
Failure to Provide a Suitable Reactor Vessel Vent Results in Inaccurate Reactor Vessel Level Indication
A self-revealing NCV was identified for failure to maintain a suitable reactor vessel vent path which resulted in inaccurate reactor vessel water level indication and lower than expected reactor vessel level.

This finding was greater than minor because it affected the initiating events cornerstone objective of configuration control of shutdown equipment. The finding determined to be of very low safety significance because all of the equipment, procedures, and policies that are expected to be maintained in the five shutdown safety functional areas were met.
Inspection Report# : 2003004*(pdf)*

Significance:  Apr 05, 2003
Identified By: Self Disclosing
Item Type: NCV NonCited Violation
Failure to Follow Chemical Control Procedures Results in Excessive Steam Generator Sodium Concentrations and Dual Unit Forced Shutdowns
Failure to follow chemistry control procedures resulted in the wrong corrosion control chemicals being added to the feedwater systems on both units and the unplanned forced shutdown of Unit 1 and Unit 2 to Mode 5, Cold Shutdown, due to high sodium concentrations in both units' feedwater systems.

A self-revealing non-cited violation of Technical Specification 5.4.1.a was identified. This finding is greater than minor because it affected the initiating events cornerstone objective by causing a perturbation of plant secondary side chemistry resulting in the unplanned forced shutdown of both units. The finding is of very low safety significance because the consequence of the chemical addition error was limited to the unplanned forced shutdown of both units. The direct cause of this finding involved the cross-cutting area of Human Performance.
Inspection Report# : 2003002*(pdf)*

Significance:  Apr 05, 2003
Identified By: Self Disclosing
Item Type: NCV NonCited Violation
Failure to Follow Power Ascension Procedure Results in Manual Reactor Trip
Failure to follow operations startup procedures resulted in a steam generator water level transient and manual reactor trip during transfer of feedwater level control to the Main Feedwater Regulating Valves.

A self-revealing non-cited violation of Technical Specification 5.4.1.a was identified. This finding is greater than minor because it affected the initiating events cornerstone objective by causing a perturbation in plant stability that resulted in a manual reactor trip. The finding is of very low safety significance because it had no other consequence other than resulting in a reactor trip. The direct cause of this finding involved the cross-cutting area of Human Performance.
Inspection Report# : 2003002*(pdf)*

Mitigating Systems

Significance:  Dec 27, 2003
Identified By: NRC
Item Type: NCV NonCited Violation
Failure to Adequately Perform Containment Closeout Inspection Resulted in Possible Loss of Post-Accident Recirculation Function of

the Residual Heat Removal System
An NRC-identified NCV of Technical Specification (TS) 5.4.1.a was identified for failure to perform an adequate Unit 1 containment closeout inspection in accordance with plant procedures.

This finding is greater than minor because it affected the equipment performance attribute of the Mitigating System Cornerstone, in that, the failure to perform an adequate closeout inspection resulted in debris left in containment that could have resulted in inadequate net positive suction head for the Residual Heat Removal (RHR) system in the recirculation phase during a design basis loss of coolant accident (LOCA). This would have affected the cornerstone objective of ensuring the availability, reliability and capability of systems (i.e. RHR in recirculation) that respond to initiating events (such as a design basis LOCA). The direct cause of this finding involved the cross-cutting area of Human Performance.
Inspection Report# : 2003005*(pdf)*

Significance: G Sep 27, 2003
Identified By: NRC
Item Type: NCV NonCited Violation
Failure to Reactivate Part 55 Licenses in Accordance with Procedure
A non-cited violation was identified for the failure of multiple Part 55 licensees to reactivate Reactor Operator and Senior Reactor Operator licenses in accordance with procedure 10010-C, Operator Qualification Program, Revision (Rev) 2.

This finding is greater than minor because it is associated with human performance attributes of license reactivation that affect operational safety. The finding was evaluated using the Operator Requalification Human Performance SDP (IMC 0609 Appendix I) and determined of very low safety significance because more than 20 percent of the reactivation records reviewed failed to meet the requirements.
Inspection Report# : 2003004*(pdf)*

Significance: G Sep 27, 2003
Identified By: NRC
Item Type: NCV NonCited Violation
Inadequate Risk Assessment For Reduced Vessel Level Conditions
A NCV of 10 CFR 50.65(a)(4) was identified for failure to properly assess and manage the increase in risk of RCS level instrumentation unavailability during a Unit 2 RCS leak repair shutdown outage.

The failure to properly assess risk following changes to planned availability of RCS level instrumentation was greater than minor because it affected the configuration control attribute of the mitigating systems cornerstone objective of ensuring the availability, reliability, and capability of systems that respond to initiating events (such as a loss of RCS inventory in reduced level conditions). The finding was determined to be of very low safety significance because all of the equipment, procedures, and policies that are expected to be maintained in the five shutdown safety functional areas were met.
Inspection Report# : 2003004*(pdf)*

Barrier Integrity

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

Miscellaneous

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