

## Nine Mile Point 2

### 1Q/2004 Plant Inspection Findings

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#### Initiating Events

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#### Mitigating Systems



**Significance:** Jun 20, 2003

Identified By: NRC

Item Type: NCV NonCited Violation

**Failure to Promptly Correct Fire Protection Deficiency.**

The inspectors identified a non-cited violation of 10 CFR 50.54(a)(1) that occurred because the fire protection corrective action requirements of the quality assurance program were not properly implemented to promptly address a problem with the adequacy of fire brigade member familiarity with all areas of the plants.

This finding adversely impacted the manual fire suppression capability and because it affects the reactor safety mitigating systems cornerstone objective, the finding is greater than minor. The finding is of very low safety significance because delays in the fire brigade response during fire drills have not been frequent and the duration of the delay during the observed drill was relatively small with respect to the established response time goal such that equipment required for safe shutdown of the plant would not have been adversely affected. (Section 1R05.6).

Inspection Report# : [2003007\(pdf\)](#)

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#### Barrier Integrity

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#### Emergency Preparedness

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#### Occupational Radiation Safety

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#### Public Radiation Safety

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#### Physical Protection

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#### Miscellaneous

**Significance:** N/A Oct 24, 2003

Identified By: NRC

Item Type: FIN Finding

**Problem Identification and Resolution Team Assessment**

The team determined that, in general, Nine Mile Point Nuclear Station (NMPNS) properly identified, evaluated and corrected problems. Corrective actions, when specified, were generally implemented in a timely manner. Audits and self-assessments were found to be acceptable. Since the last problem identification and resolution (PIR) inspection, weaknesses associated with your corrective action program have been identified as a contributing root cause for an unplanned scram performance indicator that crossed the white threshold and for a white finding associated with degraded reactor building closed loop cooling system piping. These equipment reliability issues contributed to the 2003 NRC Reactor Oversight Program (ROP) mid-cycle performance assessment that a substantive cross-cutting issue existed in the PIR area. Although the long term effectiveness of recent changes to your corrective action program cannot yet be evaluated, the team determined that the recent improvements to the corrective action program appeared appropriate. On the basis of interviews conducted during the inspection, workers at the site felt free to input safety findings into the corrective action program.

Inspection Report# : [2003011\(pdf\)](#)

Last modified : May 05, 2004