

Quad Cities 1

Initiating Events



Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: FIN Finding

INADEQUATE PROCEDURE AND SELF CHECKING RESULTS IN CONNECTING AIR POWERED VACUUM TO INSTRUMENT AIR SYSTEM AND TWO AIR TRANSIENTS

The failure to identify the proper plant air supply prior to installing moisture separator decontamination equipment (air powered vacuum) resulted in two unexpected instrument air system transients on October 14 and 15, 2002. The work package did not contain equipment identification numbers to aid in identifying the proper air supply. In addition, the individual instructed to identify the air supply failed to perform self-checking activities that could have identified the inappropriate selection of instrument air for the equipment installation rather than service air. This finding was more than minor because it affected the loss of instrument air initiating event frequency. The finding was of very low safety significance because the exposure time was short and all mitigating systems needed to address a loss of instrument air were available. No violation of NRC requirements occurred due to the instrument air system being non-safety-related.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: FIN Finding

WEAKNESSES IN PROBLEM IDENTIFICATION AND RESOLUTION LEADS TO 1B REACTOR RECIRCULATION PUMP TRIP

The failure to adequately correct deficiencies in the 1B reactor recirculation motor generator voltage regulator resulted in a pump trip and power transient on December 6, 2002. On November 29 and December 3, the licensee initiated two condition reports due to the motor generator voltage regulator failing to meet acceptance criteria during tuning activities. The inspectors determined that the licensee had not adequately considered changes made to the voltage regulator during the outage and power ascension which resulted in inappropriately concluding that the failure to meet the acceptance criteria was acceptable. This finding was determined to be more than minor because the reactor recirculation pump trip was a precursor to a significant transient. This finding was considered to be of very low safety significance since it did not: contribute to the likelihood of both a reactor trip and that mitigating equipment would not be available, contribute to the likelihood of a loss of coolant accident, increase the likelihood of a fire or flood, or increase the frequency of core damage scenarios using other plant specific analyses.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 20, 2002

Identified By: NRC

Item Type: FIN Finding

Failure to Recognize and Address High Vibration Indications on Plant Equipment

Green. The inspectors identified a Green finding due to the licensee's failure to recognize and address high vibration indications on plant equipment. On April 2, 2002, a Unit 2 main steam drain line broke due to high vibrations. The pipe break occurred down stream of the main steam isolation valves in a non-safety-related portion of the main steam piping. The issue was more than minor, in that if the vibrations were not corrected (on both units) they could become a more significant safety concern. However, due to the location of the actual break, the issue was determined to be of very low safety significance. This issue was not subject to NRC enforcement since the break occurred in a non-safety-related portion of the main steam line and did not impact the operation of safety-related equipment (Section 40A2.1.b1).

Inspection Report# : [2003002\(pdf\)](#)

Mitigating Systems



Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: FIN Finding

INADEQUATE DESIGN LEADS TO DELAY IN DISCOVERING SAFE SHUTDOWN MAKEUP PUMP WAS INOPERABLE DUE TO STRAINER CLOGGING

A self-revealing failure occurred on October 16, 2002, when the safe shutdown makeup pump room cooler strainer became clogged with duck weed. The inspectors determined that twice per shift rounds to verify strainer operability and multiple strainer cleanings were not effective in ensuring continued operability of this equipment. In addition, control room personnel were not immediately notified of the clogged strainer via a control room alarm or a local alarm due to a system design deficiency. This finding was more than minor because the strainer clogging impacted the operability of the safe shutdown makeup pump which can be used when responding to initiating events. In addition, the system design issues created a situation where operations personnel were unaware of equipment operability issues. This finding was of very low safety significance because the total exposure time was short, all other mitigating systems were available, and the safe shutdown makeup pump could have been recovered if needed. No violation of NRC requirements occurred due to the safe shutdown makeup only being of augmented quality per the licensee's Quality Assurance Report.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 28, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

MULTIPLE EXAMPLES OF SCAFFOLDING IN CONTACT WITH SAFETY-RELATED EQUIPMENT

The inspectors identified a Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion V, due to the failure to adhere to procedural requirements regarding the erection of scaffolding near safety-related equipment. On November 6, 2002, the inspectors identified numerous examples where scaffolding was in contact with residual heat removal system piping and valves. This finding was more than minor since multiple examples of scaffolding erection deficiencies were identified which indicated that workers routinely failed to follow scaffolding erection procedural requirements. This finding was determined to be of very low safety significance since the scaffolding did not result in an actual loss of safety function of any system.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 20, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," Related to the Quality of Design Basis Engineering Calculations

Green. The inspectors identified a Green Non-Cited Violation of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," related to the quality of design basis engineering calculations. Specifically, the inspectors identified that instrument and test uncertainty was not considered in a number of design calculations, such that the calculation acceptance limits could not be validated. The diesel generator cooling water (DGCW), high pressure coolant injection (HPCI), and reactor core isolation cooling (RCIC) system design bases were not being adequately controlled by existing calculations. This finding was considered greater than minor because a loss of design control could affect the reliability of the DGCW, HPCI and RCIC systems to perform their safety functions. Because no operability concerns were identified, the issue was determined to be of very low safety significance (Section).

Inspection Report# : [2003002\(pdf\)](#)



Significance: Sep 30, 2002

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

FAILURE TO HAVE APPROPRIATE BEARING FIT-UP AND MOTOR LUBRICATION INSTRUCTIONS

Inadequate bearing fit-up measurement and motor lubrication instructions resulted in a self-revealing failure of the 1A core spray and reactor core isolation cooling room cooler fan inboard motor bearings and a Non-Cited Violation of Technical Specification 5.4.1. The inspectors determined that this finding was more than minor because the improper bearing fit-up and lubrication instructions impacted the availability, reliability, and capability of equipment used to support risk significant mitigating equipment. The failure of the 1A core spray and reactor core isolation cooling room cooler was of low risk significance because the failure was not caused by a design or qualification deficiency, did not result in an actual loss of safety function for the core spray or reactor core isolation cooling systems, and did not screen as potentially risk significant due to a seismic, fire, flooding, or severe weather initiating event.

Inspection Report# : [2002007\(pdf\)](#)



Significance: Sep 30, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO ADEQUATELY CORRECT MULTIPLE FAILURES OF THE 2A RHR NORMAL/ALTERNATE SWITCH

Ineffective corrective actions resulted in repetitive failures of the 2A residual heat removal normal/alternate switch between June 1999 and September 2002 and a Non-Cited Violation of 10 CFR 50, Appendix B, Criterion XVI. The failure to correct the multiple normal/alternate

switch failures was more than minor because the switch failures impacted the availability, reliability, and capability of equipment used to respond to initiating events and prevent undesirable consequences from a plant fire. This finding was of very low risk significance because the switch failures did not result in an actual loss of function for the residual heat removal system. The switch failures also failed to screen as a risk significant fire issue because the room cooler was not needed until 52 hours after a fire which provided the licensee adequate time to correct the failure.

Inspection Report# : [2002007\(pdf\)](#)

G

Significance: Sep 30, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO DOCUMENT AN OVERGREASING ISSUE IN THE CORRECTIVE ACTION PROGRAM AND TAKE ACTION TO ADDRESS THE EXTENT OF CONDITION

The licensee failed to follow procedural requirements regarding the initiation of condition reports and determining the extent of condition following the discovery of a large amount of grease in the 1A core spray room cooler motor. As a result, the licensee did not provide a basis for continued operability of potentially impacted plant motors for approximately 40 days. This finding was more than minor because the licensee's lack of action resulted in the inability to ensure the availability and reliability of mitigating systems equipment used to respond to initiating events and prevent undesirable consequences. The inspectors determined that this finding was of very low risk significance because subsequent reviews determined that even if the motors susceptible to overgreasing failed, the motors are not credited in the licensee's probabilistic risk assessment.

Inspection Report# : [2002007\(pdf\)](#)

G

Significance: Jun 30, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO PERFORM REQUIRED PARTS EVALUATION FOR CONTROL ROD DRIVE ACCUMULATOR CLAMPS

The inspectors identified a design deficiency and a Non-Cited Violation in that licensee personnel failed to perform a parts evaluation when installing hose clamps on the control rod drive system hydraulic accumulators instead of the seismically-qualified steel band clamps. This issue was of very low safety significance because the design deficiency did not result in a loss of function as described in Generic Letter 91-18, "Resolution of Degraded and Non-Conforming Conditions and on Operability."

Inspection Report# : [2002005\(pdf\)](#)

G

Significance: Jun 30, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

FAILURE TO MEET 10 CFR 50.62 DUE TO RELIEF VALVES LIFTING

The inspectors documented a Non-Cited Violation of 10 CFR 50.62, "Anticipated Transient Without Scram Rule," due to the potential to lift the standby liquid control system relief valves during an anticipated transient without scram. The inspectors determined that this finding was of very low safety significance because the standby liquid control system could be recovered during an anticipated transient without scram event, the cycling of the relief valves would allow a portion of the sodium pentaborate solution to be injected into the reactor vessel, and the plant remained within the acceptance criteria of the original anticipated transient without scram analyses during the relief valve lifts.

Inspection Report# : [2002005\(pdf\)](#)

G

Significance: Feb 01, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adequately address the erosion of the RHRSW room coolers' supply piping

Green. The inspectors identified a failure to promptly identify and correct conditions adverse to quality involving the erosion of safety-related residual heat removal service water piping. The licensee's corrective actions for the piping leak included replacing the affected piping and performing ultrasonic testing on similar piping for the other trains. During this inspection, NRC inspectors identified that the corrective actions were inadequate in that the ultrasonic testing was not able to examine the area of the piping affected by the erosion as evidenced by the subsequent failure. This finding was determined to be of very low safety significance because the equipment was still capable of performing its intended safety function. A Non-Cited Violation of 10CFR 50 Appendix B, Criterion XVI was identified.

Inspection Report# : [2001015\(pdf\)](#)

G**Significance:** Dec 28, 2002

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

FAILURE TO FOLLOW PROCEDURE AND WEAK CONTROL BOARD MONITORING LEADS TO INADVERTENT REACTOR WATER CLEANUP ISOLATION

The failure to adhere to procedure precautions and perform timely control room panel monitoring resulted in the inadvertent isolation of the reactor water cleanup system while the system was being used to remove decay heat from the Unit 1 reactor vessel. A Non-Cited Violation of Technical Specification 5.4.1 was identified. This finding was determined to be more than minor because the isolation impacted the reactor water cleanup system's continued ability to provide cooling of the reactor fuel and fuel cladding while the Unit 1 reactor was in a shutdown condition. The finding was of very low safety significance since the isolation did not significantly degrade the licensee's ability to recover decay heat removal through the use of the reactor water cleanup or residual heat removal systems once the isolation occurred.

Inspection Report# : [2002008\(pdf\)](#)

Emergency Preparedness

Occupational Radiation Safety

Public Radiation Safety

Physical Protection

Miscellaneous

Significance: N/A Dec 20, 2002

Identified By: NRC

Item Type: FIN Finding

Corrective Action Program Generally Effective in Ensuring that Conditions Adverse to Quality were Being Adequately Addressed

Although issues were generally entered into the corrective action process at an appropriate level, there were times when opportunities to identify issues were missed due to a narrow focus. This led to issues either being self-revealing or being identified by outside organizations. Several of these issues resulted in plant shutdowns. Minor issues were generally properly categorized and evaluated. However, there were a number of examples where it appeared that the initial evaluation was limited and narrowly focused. These examples tended to be non-routine and involved multiple organizations and layers of management. In general, the licensee effectively corrected plant problems. On the positive side, the inspectors noted that Nuclear Oversight appeared to be an effective source for identifying performance issues and that plant employees, in general, indicated a strong willingness to report problems "to the highest levels." The inspectors concluded that corrective action program was generally effective in ensuring that conditions adverse to quality were being adequately addressed

Inspection Report# : [2003002\(pdf\)](#)

Last modified : March 25, 2003