

Oyster Creek

Initiating Events



Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: FIN Finding

Inadequate procedures and personnel error cause (NOUE) offgas ignition

An inadequate maintenance procedure resulted in the inadvertent ignition of Hydrogen gasses contained in the offgas system during air in-leakage testing. The procedure failed to provide instructions to properly isolate and vent the test device sample chamber from the process stream before ionizing the test sample chamber. A self-revealing finding was identified. This finding is greater than minor because it had an actual impact of igniting the offgas system hydrogen gas, causing the main condenser offgas system to be isolated and therefore could be viewed as a precursor to a significant event. If the offgas system could not have been quickly restored, it would have caused a reactor scram. The finding is of very low significance because all mitigation systems were available during this event, the hydrogen ignition did not result in damage to the plant and was contained within a system designed for such events, and operators restored the offgas system before main condenser vacuum degraded to a trip condition. In addition, this finding had a human performance aspect, in that plant technicians proceeded to perform the test without a plant specific procedure and they did not fully adhere to the guidance provided with the equipment which had a direct causal affect on the event initiation.

Inspection Report# : [2002008\(pdf\)](#)

Mitigating Systems



Significance: Dec 28, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to adhere to design control specifications

A non-cited violation of 10 CFR 50 Appendix B, Criterion V was identified for failure to implement engineering instructions provided in an engineering change request document. AmerGen personnel installed three pipe fittings in the isolation condenser system using material which was specifically prohibited from use by the engineering document. Oyster Creek personnel had not adhered to procedural requirements governing the control of materials used for the installation of piping in the isolation condenser system. This finding is greater than minor because it affected the design control attribute of the Mitigating Systems Cornerstone and could have affected the reliability of the isolation condenser system. The finding is of very low safety significance because the plant was not operational at the time and subsequent analysis verified the vent line modification was in compliance with the applicable Code and design requirements. In addition, this finding had a human performance aspect, in that plant technicians did not adhere to installation guidance provided in the modification package.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Failure to maintain ST procedure acceptance criteria - SWS pump test

A non-cited violation of Oyster Creek Technical Specification 6.8, Procedures and Programs, was identified for failure to have an adequate surveillance procedure for the emergency service water pump. AmerGen failed to maintain appropriate acceptance criteria in the quarterly emergency service water pump inservice test procedure. The finding is considered more than minor because it is associated with the Mitigating Systems cornerstone attribute of procedure quality and affects the Mitigating Systems cornerstone objective to ensure the availability, reliability, and capability of systems that respond to initiating events. The finding is of very low safety significance because the finding was a qualification deficiency confirmed to not result in the loss of the safety function of the Emergency Service Water System.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Dec 28, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to Conduct Post-Maintenance Test for HCU 42-27

A non-cited violation of Oyster Creek Technical Specification 6.8, Procedures and Programs, was identified for failure to adequately implement a Control Rod Drive system procedure. AmerGen declared a control rod operable, following maintenance work, without performing post-maintenance testing as required by the procedure. The finding is considered greater than minor in that the issue was associated with the Mitigating System Cornerstone and potentially affected the scram function of a control rod in response to an initiating event. The finding is of very low safety significance because the control rod remained at the full in position (notch 00) throughout the performance of the maintenance work and no other control rods were concurrently inoperable.

Inspection Report# : [2002008\(pdf\)](#)



Significance: Oct 12, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Identification and Resolution of Problems

The inspectors identified a non-cited violation of 10 CFR 50 App. B criterion XVI. AmerGen corrective actions for controlling accumulator pressure on the Control Rod Drive System Hydraulic Control Units did not prevent recurrence of the problem. Specifically, corrective actions taken in January 2002 to prevent exceeding the pressure limit permitted by Procedure 302.1, "Control Rod Drive System," were ineffective in preventing recurrence of the issue on July 25, 2002. This finding was considered to have very low safety significance using the SDP Phase 1 assessment and the inspector's review of immediate and subsequent corrective actions.

Inspection Report# : [2002007\(pdf\)](#)



Significance: Jun 07, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Control room ventilation air flow -Criteria XVI problem.

The inspectors identified a Non-Cited Violation for failure to promptly identify and correct a condition adverse to quality regarding a low air flow condition in Train "A" of the control room ventilation system (10 CFR Part 50 App B Crt XVI). The licensee had not adequately monitored and evaluated surveillance test trend data since at least 1996, to identify decreasing air flow measurements. Consequently, the condition had not been evaluated in the licensee's corrective action process, and neither has the condition been evaluated as acceptable nor have corrective actions been taken. This issue was more than minor because the condition could impact the habitability and equipment qualification for the control room. The finding was evaluated using Phase 1 of the NRC SDP and determined to have very low significance (Green) since the equipment remained operable with no loss of safety function for the control room.

Inspection Report# : [2002003\(pdf\)](#)



Significance: Feb 09, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Control Rod Drive Partial Equipment Condition and Alignment Verification

The inspectors identified a Non-Cited Violation for failure to assure that design control measures were in place regarding deviation from original design replacement parts on the Control Rod Drive system (10 CFR 50 Appendix B, Criterion III). The inspectors identified two hydraulic control units that had water accumulator belly band clamps which were not of the original design and had not been evaluated by engineering for use on the system. However, the failure to evaluate the adequacy of the replacement part was considered to have very low safety significance (Green) using the Significance Determination Process (SDP) phase 1 assessment since the band clamps in question were replaced within 24 hours of initial notification of the issue and a subsequent engineering evaluation determined the replacement part to be equivalent. This violation is being treated as a Non-Cited Violation (NCV) consistent with section VI.A.1 of the NRC Enforcement Policy. (NCV 50-219/01-13-01) This NCV was closed in IR2001013.

Inspection Report# : [2001013\(pdf\)](#)



Significance: Feb 09, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Event Follow-up

The inspectors identified a Non-Cited Violation for failure to correct a significant condition adverse to quality identified in 1999, 2000, and again in 2001 (10 CFR 50 Appendix B, Criterion XVI). This condition, associated with degraded control circuit components within 480 volt breakers, is more than minor since multiple failures of a safety related breaker could have a credible impact on safety. The issue affects mitigating systems including containment spray, core spray and control rod drive pumps. The finding was evaluated using an NRC SDP phase 2 assessment and determined to have very low safety significance (Green). This violation is being treated as a Non-Cited Violation (NCV) consistent with section VI.A.1 of the NRC Enforcement Policy. The licensee entered this issue into the CAP (02002-0157). (NCV 50-219/01-13-02) This NCV was closed in IR2001013

Inspection Report# : [2001013\(pdf\)](#)

Barrier Integrity

**Significance:** Dec 28, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Failure to identify degraded condition - SGTS charcoal filter

A non-cited violation of 10 CFR 50 Appendix B Criterion XVI, Corrective Actions, was identified for failure to adequately identify and correct a condition adverse to quality involving the continued operability of the No. 2 Standby Gas Treatment System charcoal filter. In May 2001, testing indicated that the charcoal filter efficiency would not remain fully operable for the ensuing test interval and this condition was not identified in a timely manner to ensure that corrective actions could be taken. Subsequent testing in October 2002, confirmed that the filter had degraded below the technical specified minimum efficiency during the surveillance interval. The finding is considered greater than minor because it had an actual impact in that the No. 2 SGTS was inoperable. The finding is of very low safety significance because the finding only represented a degradation of the radiological barrier function provided for by the standby gas treatment system. In addition, this finding had a corrective action performance aspect, in that degraded or non-conforming conditions adverse to quality had not been identified in a timely manner to ensure appropriate corrective actions were taken.

Inspection Report# : [2002008\(pdf\)](#)**Significance:** Dec 28, 2002

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Failure to maintain secondary containment configuration - trunnion room door

A non-cited violation was identified during the performance of the primary containment isolation valve test on October 22, 2002, for failure to maintain the secondary containment configuration in accordance with technical specification 3.5.B, when the trunnion room door was opened and not administratively controlled, which resulted in a temporary loss of secondary containment. A self-revealing finding was identified. The finding is considered more than minor because the reactor safety barrier integrity cornerstone attribute of human performance was involved and adversely affected the cornerstone objective to provide reasonable assurance that physical design barriers protect the public from radio nuclide releases caused by accidents or events. The finding is of very low safety significance since the finding involved a BWR in a Cold Shutdown condition with time to boil being greater than 2 hours and reactor coolant system level less than 23 feet above the top of reactor flange and the inspector verified that secondary containment closure could be accomplished in sufficient time before a release of fission products, including the unavailability of AC power and the expected environmental condition in containment. In addition, this finding had a human performance aspect, in that plant operators did not adhere to directions provided to ensure that the trunnion room door was maintained closed and only opened for the short time for passage through the area as required by the licensee's administrative controls.

Inspection Report# : [2002008\(pdf\)](#)**Significance:** Oct 12, 2002

Identified By: NRC

Item Type: NCV NonCited Violation

Permanent Plant Modifications

The inspectors identified a Non-Cited Violation of Oyster Creek Technical Specification 6.8, Procedures and Programs. The inspectors found that AmerGen failed to maintain procedures No. 317, Feedwater System, and No. 202.1, Power Operation, following the installation of the Digital Average Power Range Monitor Flow Control Trip Reference Card permanent modification (Engineering Change Request 01-01193), which occurred during the week of September 1, 2002. Specifically, the feedwater system procedure was not revised to reflect a maximum core flow limitation, as prescribed in the vendors' analysis that was referenced in the 10 CFR 50.59 evaluation for the modification installation. This finding was considered to have very low safety significance using the SDP Phase 1 assessment, the inspector's review of immediate and subsequent corrective actions, and a review of control room logs, in which the inspector verified that the maximum core flow limitation was not exceeded.

Inspection Report# : [2002007\(pdf\)](#)

Emergency Preparedness

Occupational Radiation Safety

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Significance: Dec 28, 2002

Identified By: Self Disclosing

Item Type: NCV NonCited Violation

Ineffective Problem Resolution - Self-Reading Dosimeter Alarms

A NCV of TS 6.13 was identified for failure to establish fully effective problem resolution relative to recurring problems involving personnel failing to hear the integrated dose alarm of their electronic self-reading personnel dosimetry equipment and to promptly respond to such an alarm. A self-revealing finding was identified due to repeat events in violation of TS was more than minor in that worker safety could be impacted in similar circumstances if workers failed to properly respond to alarming dosimeters in situations with the potential for unplanned radiation dose. SDP determined it was Green since no overexposure occurred, no substantial potential for an overexposure, and the licensee's ability to assess dose to workers was not affected. Therefore, the issues were determined to be of very low safety significance.

Inspection Report# : [2002008\(pdf\)](#)

Public Radiation Safety

Physical Protection

Miscellaneous

Significance: N/A Jun 07, 2002

Identified By: NRC

Item Type: FIN Finding

Identification and Resolution of Problems

The PI&R team concluded the implementation of the corrective action program at Oyster Creek was adequate. Problems were identified at an appropriate level. Problems were generally prioritized and evaluated in a timely manner; however, some contrary examples were identified, including a Green finding for not trending degrading performance for the control room ventilation system. Licensee identified and implemented corrective actions were implemented in a timely fashion, except for problems involving tagging. The team determined that while the licensee identified and evaluated a continuing trend in equipment clearance and tagging errors, the corrective actions to date have not been effective in improving performance, in disagreement with the licensee's March 2002 effectiveness review for these problems.

Inspection Report# : [2002003\(pdf\)](#)

Last modified : March 25, 2003