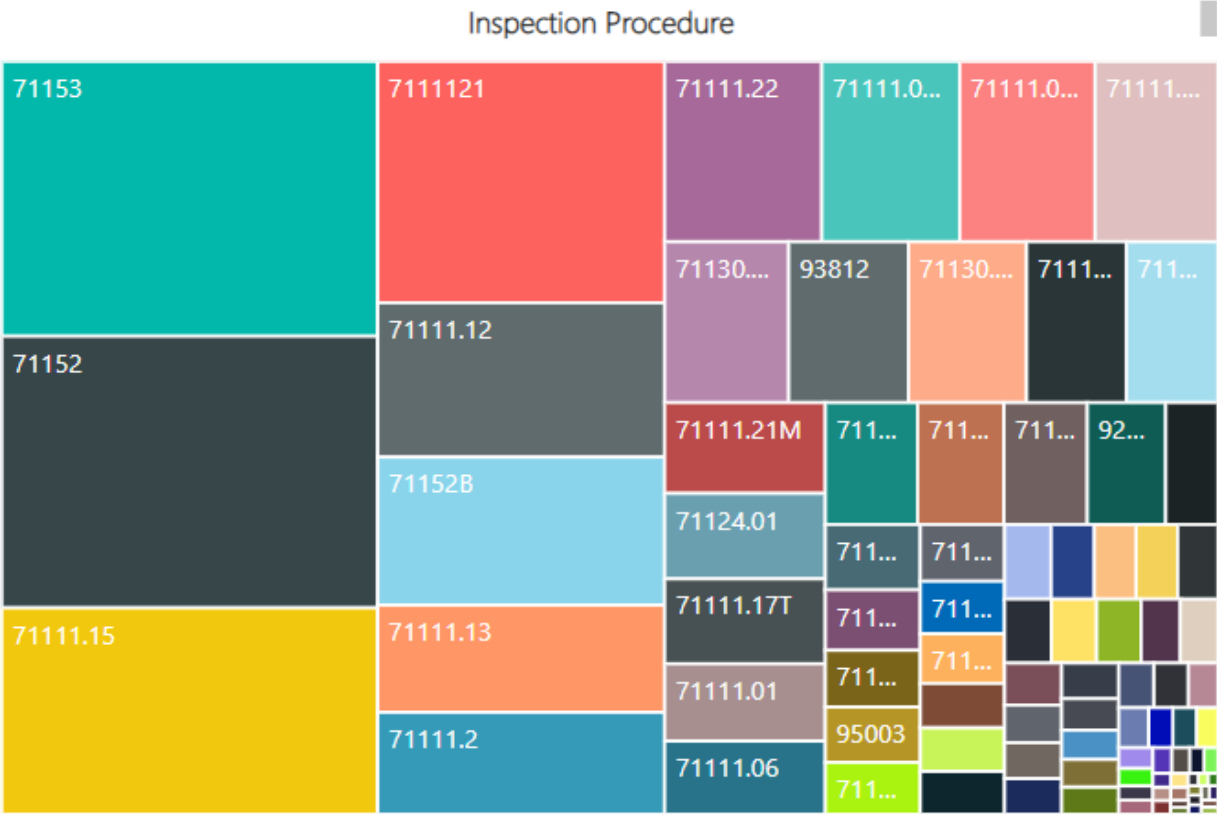
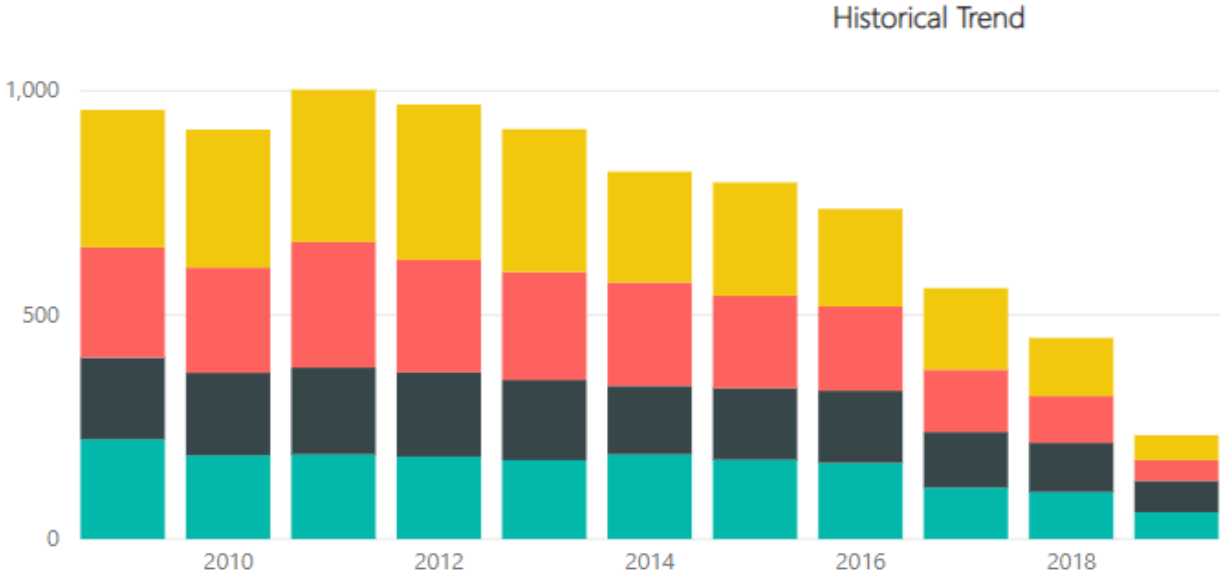


NRR Operating Experience Dashboard - Findings Data Analysis Tool



Region ● Region: I ● Region: II ● Region: III ● Region: IV



Year

2009

2019

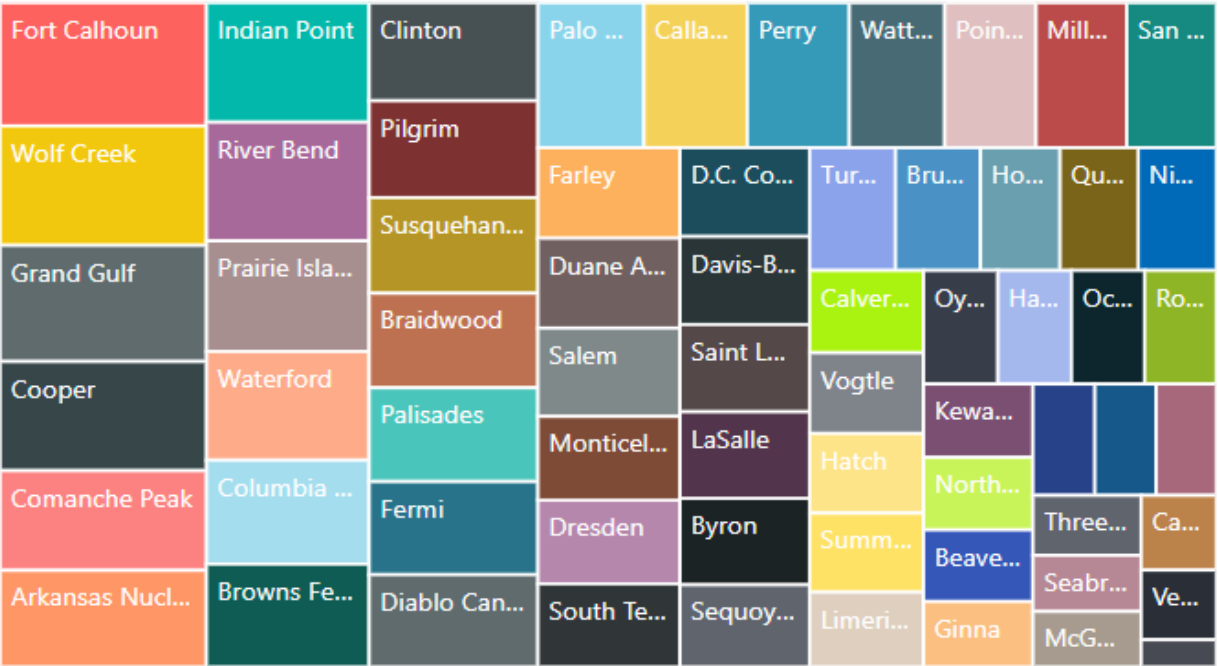
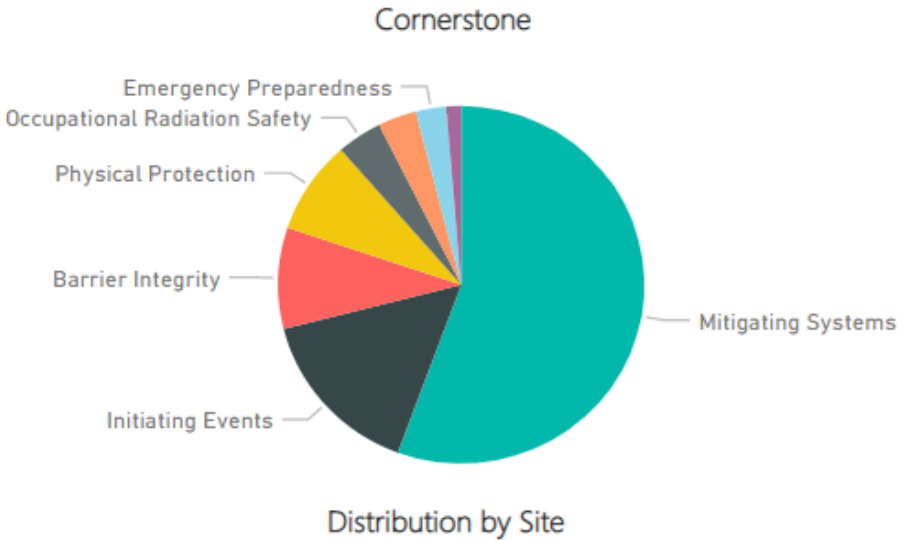
Region

☐ Region: I

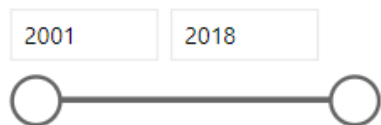
☐ Region: II

☐ Region: III

☐ Region: IV



NRR Operating Experience Dashboards - Findings Trend

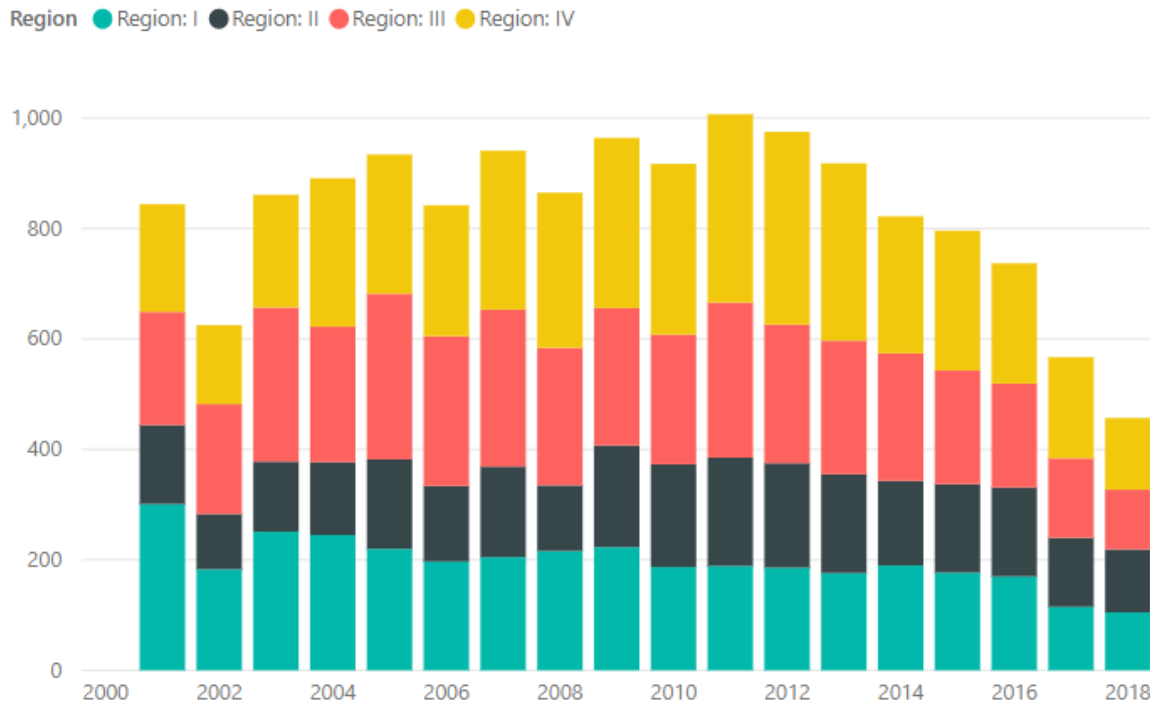


- Region
- ☐ Region: I
 - ☐ Region: II
 - ☐ Region: III
 - ☐ Region: IV

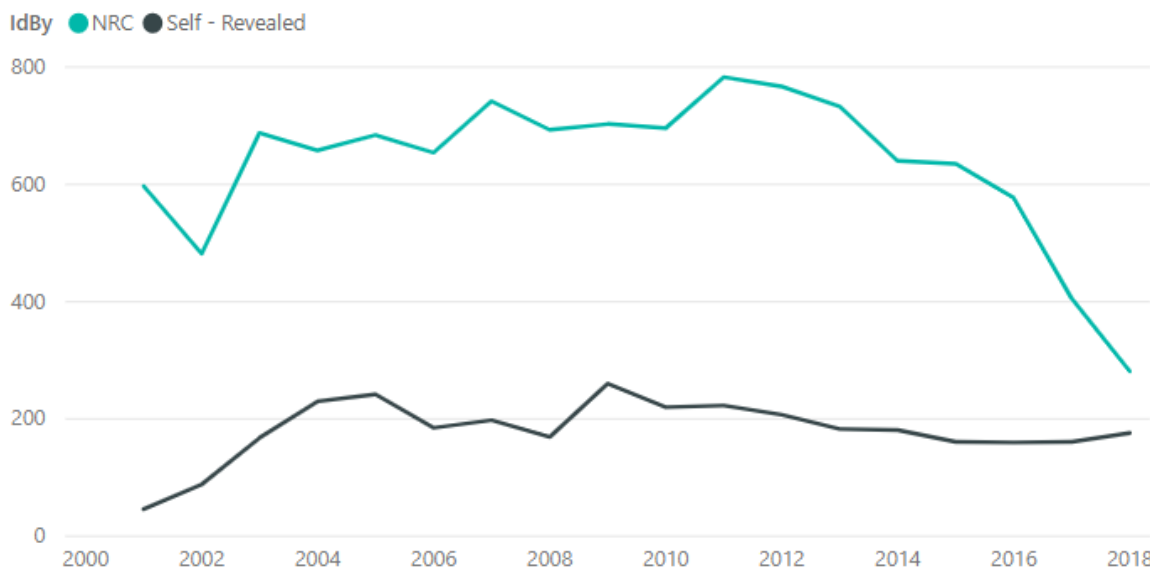
- Significance
- ☐ Select all
 - ☐ Green
 - ☐ Minor Performan...
 - ☐ No Performance ...
 - ☐ Pending
 - ☐ Preliminary Great...
 - ☐ Preliminary White
 - ☐ Red
 - ☐ TBD
 - ☐ White
 - ☐ Yellow

PIM Entry Contains

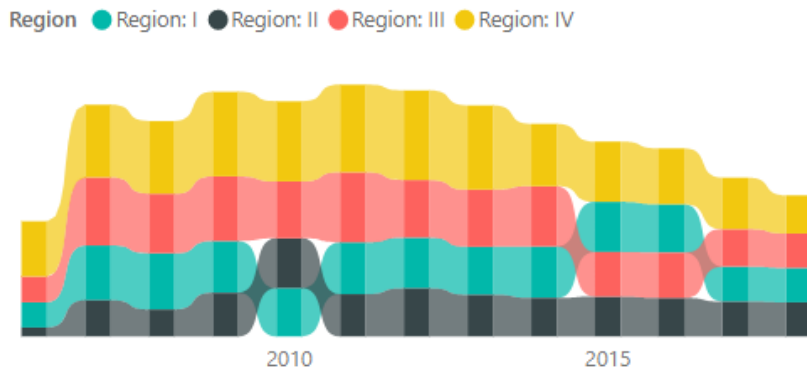
Findings By Region



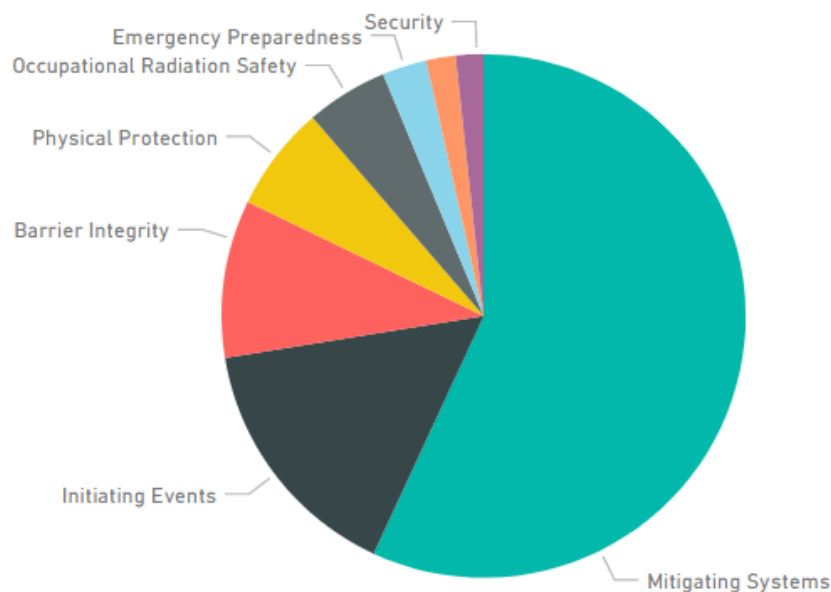
NRC-Identified vs Self-Revealed



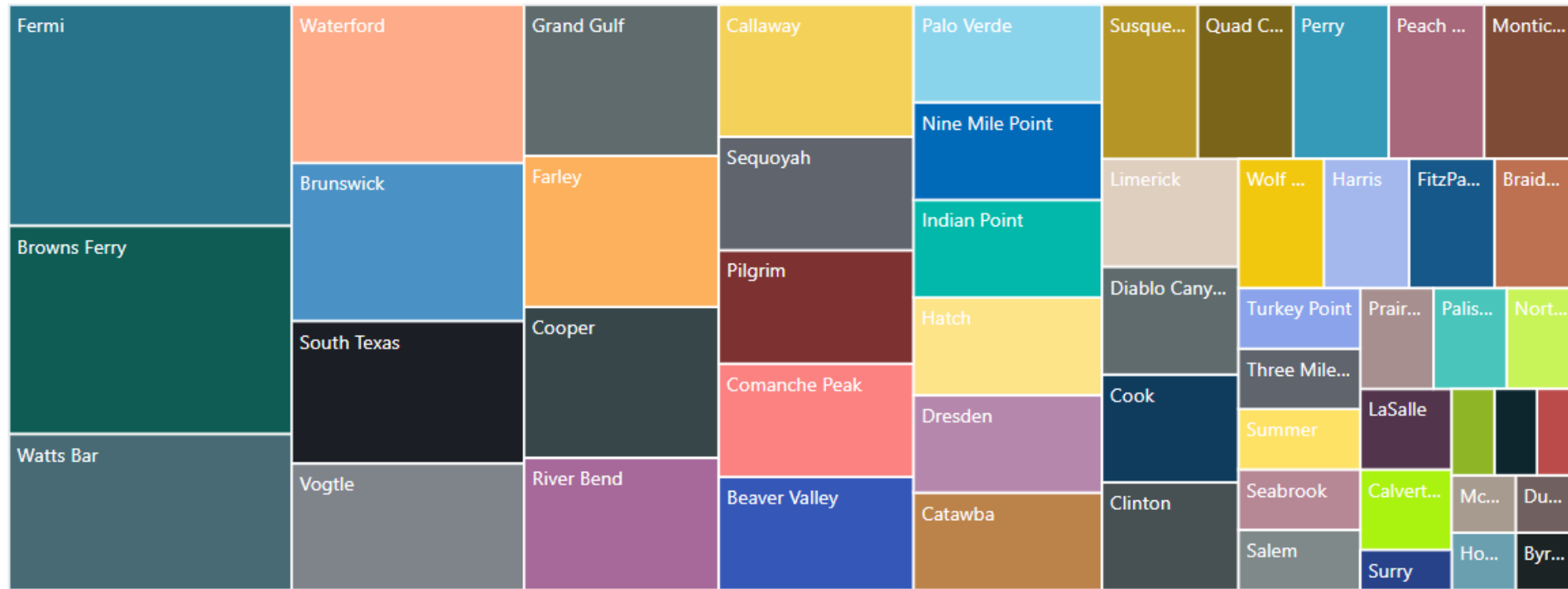
Findings with Cross Cutting Aspects Over Time



Findings By Cornerstone

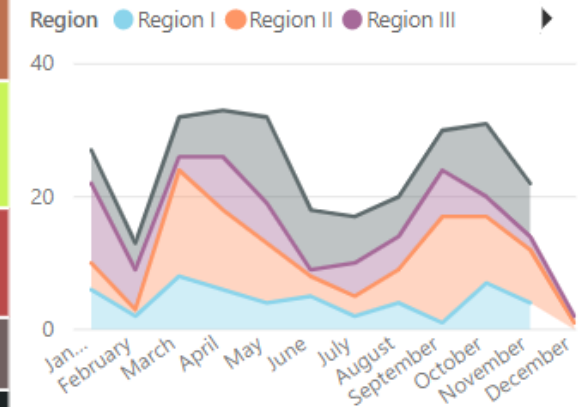


NRR Operating Experience Dashboards - 2019 Event Notifications



2019 Event Notifications by Site

Region
☐ Region I
☐ Region II
☐ Region III
☐ Region IV



Year	Month	Day	Site Name	Rx Type	Event Text
2019	May	9	ARKANSAS NUCLEAR	[1] B&W-L-LP,[2] CE	60-DAY TELEPHONIC NOTIFICATION OF INVALID ENGINEERED SAFETY FEATURE ACTUATION SIGNAL (iv)(A) to provide information pertaining to an invalid Engineered Safety Feature actuation signal. "On May 9, 2019, at Arkansas Nuclear Control (EFIC) Channel B monthly test, a test pushbutton was mispositioned, resulting in an inadvertent initiation of the Emergency Feedwater (EFW) System (ESAS) Trip Test portion of the surveillance, the first technician placed EFIC Train B in the tripped condition. The second technician then went to the first technician recalls thinking he was given the order to reset Train B EFW Bus 1 Trip. Therefore, the first technician performed the step using three-part con "Due to the amount of time the second technician spent in front of the control room, the first technician assumed Operations reset the RSM to complete the ESAS cabinet and directed the first technician to perform the reset of Train B EFW Bus 1 Trip. The first technician, expecting his next action to be the trip of Train B EFW in trip and caused the actuation of P-7A EFW Pump. "This inadvertent actuation was caused by human error and was not Consideration define an invalid signal to include human error. Therefore, this actuation is considered invalid. "This event was entered into any adverse impact to the health and safety of the public. The plant responded as expected. "In accordance with 10 CFR 50.73(a)(i) a tele Report. "The licensee has notified the NRC Resident Inspector."
2019	May	19	ARKANSAS NUCLEAR	[1] B&W-L-LP,[2] CE	LOSS OF THE SAFETY PARAMETER DISPLAY SYSTEM DUE TO INVERTER FAILURE "On May 19, 2019, at 1809 CDT, the Safety Parameter Display to the SPDS Inverter (2Y-26) failure. The SPDS Inverter is the power supply to both units' SPDS. The Unit 2 Control Room dispatched operators in response to was reported emanating from 2Y-26. There was no report of fire at any time. Field operators de-energized 2Y-26 and the smoke ceased. The loss of SPDS also Operating Limits Supervisory System (COLSS) to be lost, so Unit 2 reduced power to 91 [percent] in accordance with Technical Specifications. Both units are i "This is reportable per 10 CFR 50.72(b)(3)(xiii)."

NRR Operating Experience Dashboards - Plant Risk Concept

Birnbaum by Component Description

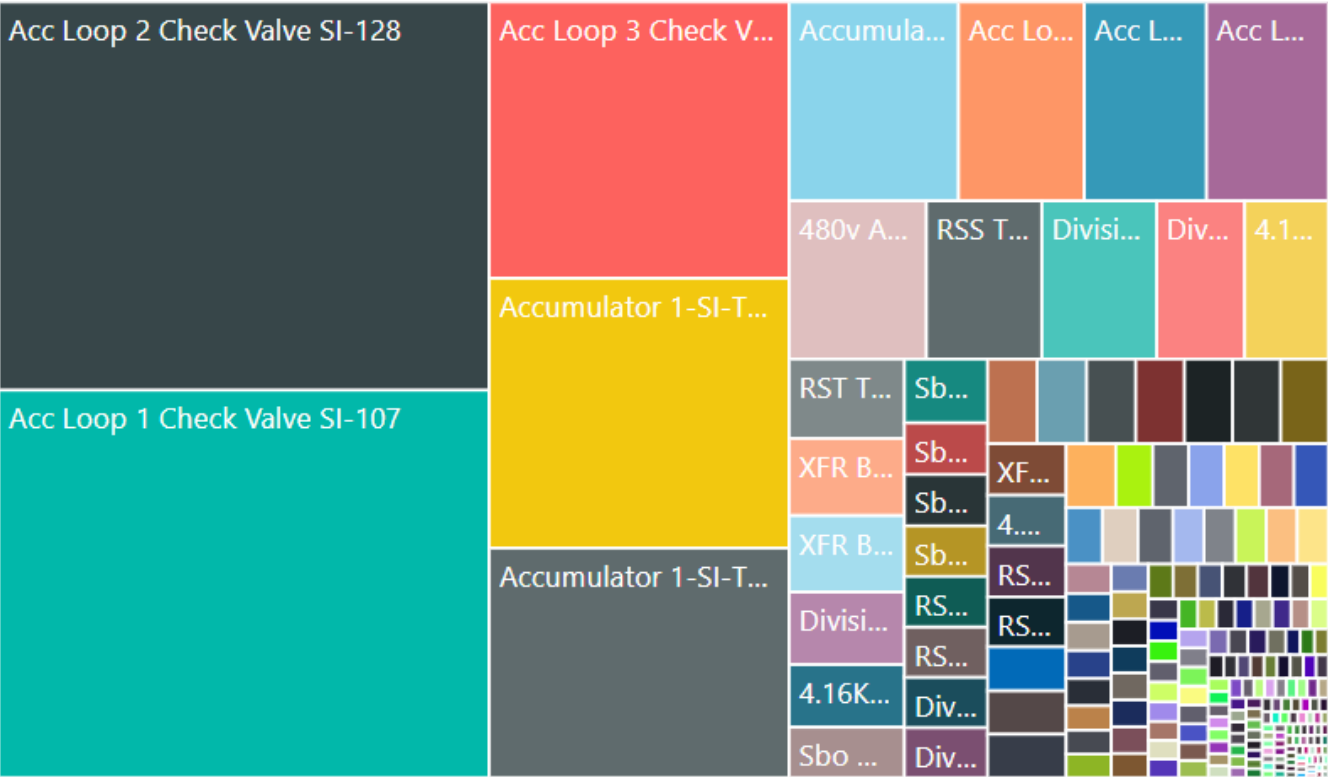
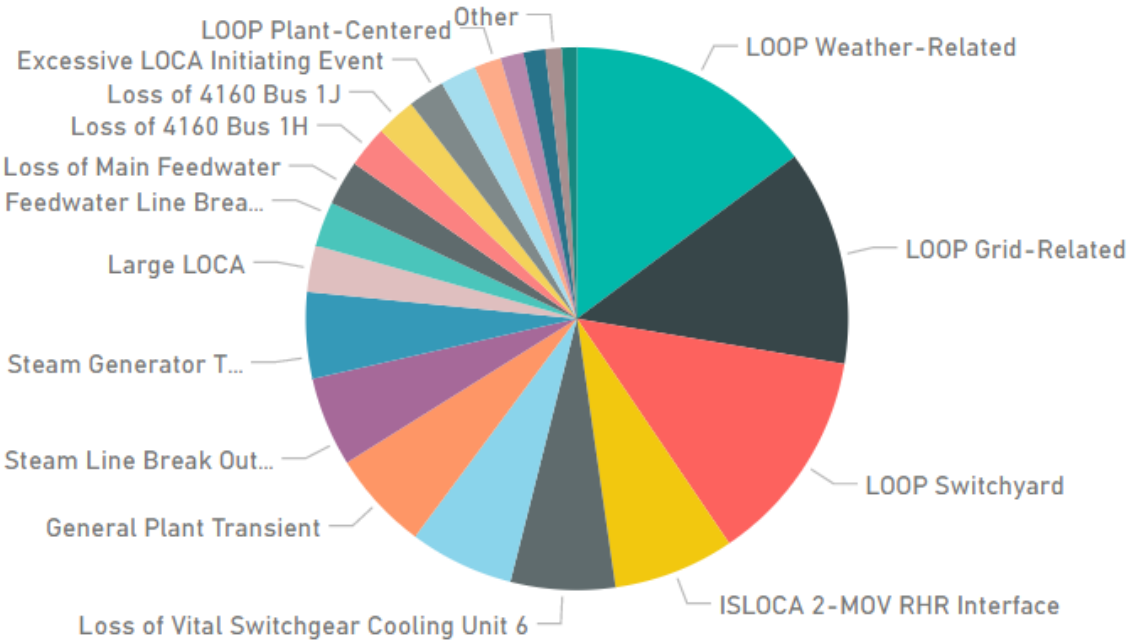


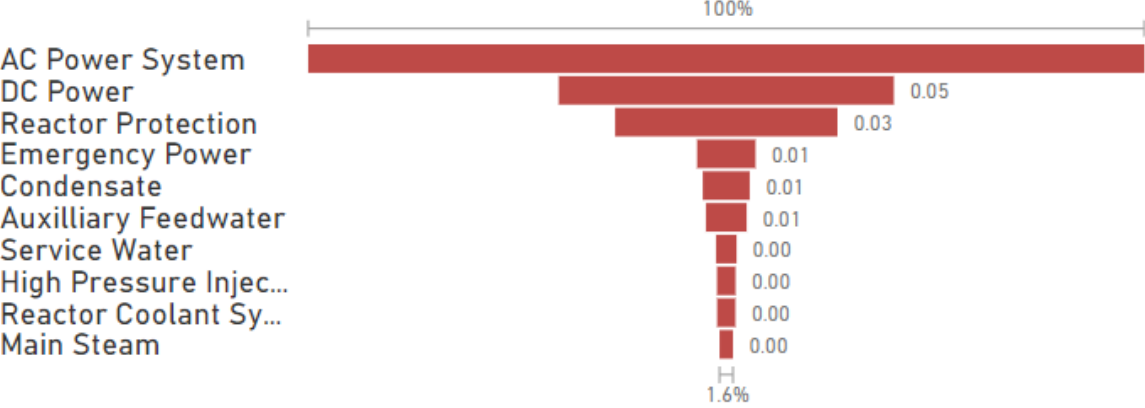
Table of Component RAW

Component Description	RAW
Acc Loop 1 Check Valve SI-107	872.00
Acc Loop 2 Check Valve SI-128	872.00
Acc Loop 3 Check Valve SI-145	380.00
Accumulator 1-SI-TK-1A	371.00
Accumulator 1-SI-TK-1B	317.00
Accumulator 1-SI-TK-1C	155.00
Acc Loop 1 MOV 1865a	116.00
Total	5,130.03

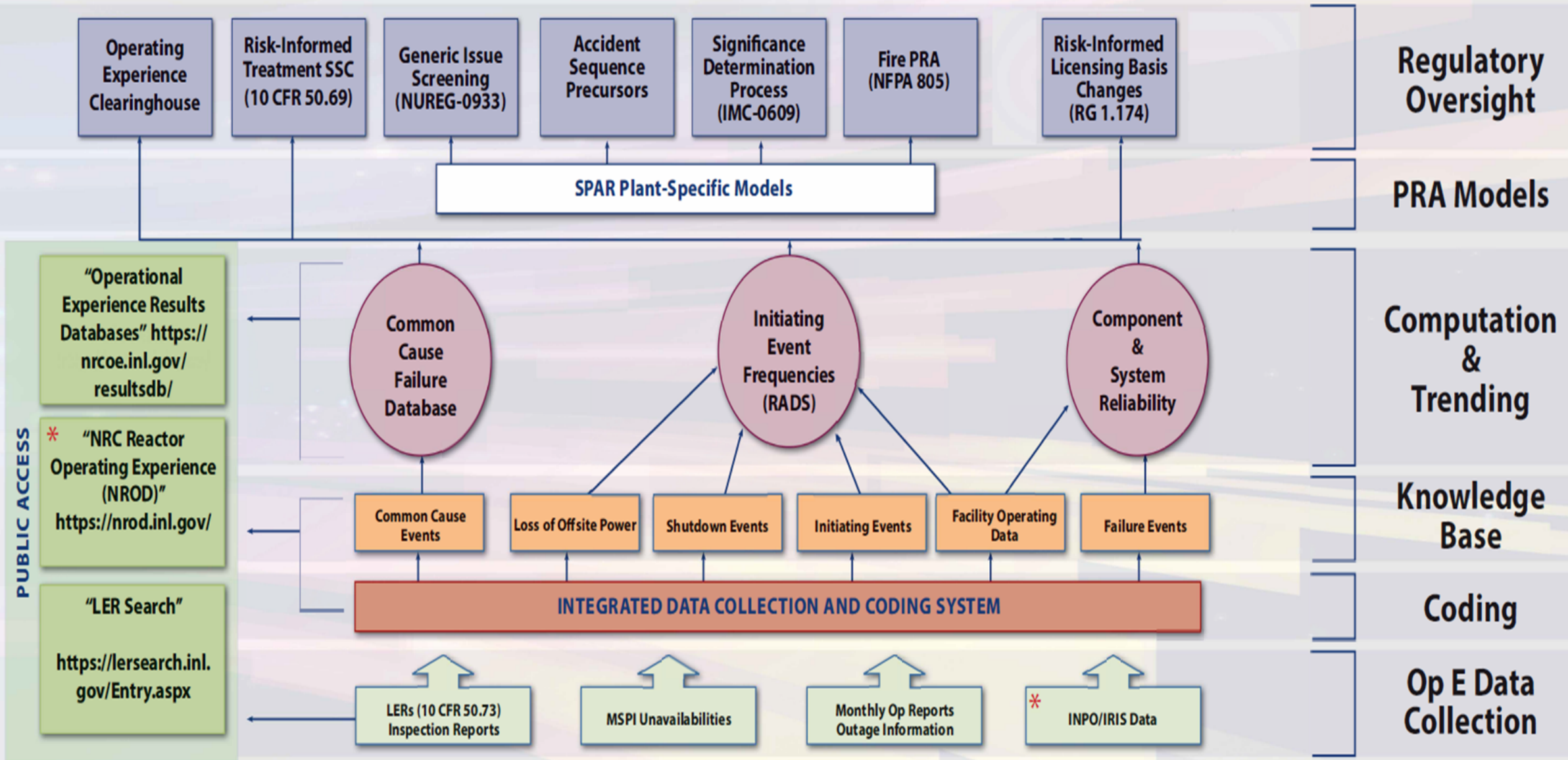
Percent Contribution to CDF by Initiating Event



CDF Increase from Failure by Risk Important Systems



Operational Data Collection and Analysis for Risk-Informed Activities



Data Storage
Data Collection
Computation & Trending
Web Site
Coding
PRA Models

* Proprietary

INPO/IRIS-Institute of Nuclear Power Operations/Industry Reporting & Information System
 IMC-Inspection Manual Chapter; LER-Licensee Event Report; MSPI-Mitigating Systems Performance Index
 OpE-Operating Experience; PRA-Probabilistic Risk Assessment; Rads-Reliability & Availability Data System
 SPAR- Standard Plant Analysis Risk; SSC= Structures, Systems & Components