Davis-BesseNPEm Resource

From: CuadradoDeJesus, Samuel

Sent: Tuesday, February 07, 2012 7:40 PM

To: 'dorts@firstenergycorp.com'

Cc: 'custerc@firstenergycorp.com'; Davis-BesseHearingFile Resource

Subject: 9/16/2011 teleconference call summary
Attachments: 9/16/2011 DB NRC Telecon Summary.docx

Importance: High

Steve,

Let me know if FENOC has any comment.

Regards,

Samuel Cuadrado de Jesús

Project Manager
Projects Branch 1
Division of License Renewal
U.S. Nuclear Regulatory Commission

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Hearing Identifier: Davis_BesseLicenseRenewal_Saf_NonPublic

Email Number: 3508

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Subject: 9/16/2011 teleconference call summary

Sent Date: 2/7/2012 7:40:16 PM
Received Date: 2/7/2012 7:40:18 PM
From: CuadradoDeJesus, Samuel

Created By: Samuel.CuadradoDeJesus@nrc.gov

Recipients:

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9 16 2011 DB NRC Telecon Summary.docx 32213

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Priority:HighReturn Notification:NoReply Requested:NoSensitivity:Normal

Expiration Date: Recipients Received:

LICENSEE: FirstEnergy Nuclear Operating Company

FACILITY: Davis-Besse

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON

SEPTEMBER 16, 2011, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND FIRSTENERGY NUCLEAR OPERATING COMPANY, CONCERNING REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE DAVIS-BESSE, LICENSE RENEWAL APPLICATION (TAC. NO.

ME4640)

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on September 16, 2011, to discuss and clarify the applicant's responses to the staff's requests for additional information (RAIs) concerning the Davis-Besse license renewal application.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a description of the staff concerns discussed with the applicant. A brief description on the status of the items is also included.

The applicant had an opportunity to comment on this summary.

Samuel Cuadrado de Jesús, Project Manager Projects Branch 1 Division of License Renewal Office of Nuclear Reactor Regulation

Docket Number:50-346 Enclosures:

List of Participants
 List of Requests for Additional

Information

cc w/encls: See next page

LICENSEE: FirstEnergy Nuclear Operating Company

FACILITY: Davis-Besse

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Enclosure 1 provides a listing of the participants and Enclosure 2 contains a description of the staff concerns discussed with the applicant. A brief description on the status of the items is also included.

The applicant had an opportunity to comment on this summary.

Samuel Cuadrado de Jesús, Project Manager Projects Branch 1 Division of License Renewal Office of Nuclear Reactor Regulation

Docket Number: 50-346

Enclosures:

1. List of Participants

2. List of Requests for Additional Information

cc w/encls: See next page DISTRIBUTION: See next page

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SUMMARY OF TELEPHONE CONFERENCE CALL DAVIS-BESSE LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS September 16, 2011

<u>PARTICIPANTS</u> <u>AFFILIATIONS</u>

Samuel Cuadrado de Jesús U.S. Nuclear Regulatory Commission (NRC)

Allen Hiser NRC Ching Ng NRC

Omesh Chopra Argonne National Laboratory

Allen McAllister FirstEnergy Nuclear Operating Company (FENOC)

Larry Hinkle FENOC
John Hartigan FENOC

Tony Giannuzzi Structural Integrity Associates

SUMMARY OF TELEPHONE CONFERENCE CALL DAVIS-BESSE LICENSE RENEWAL APPLICATION September 16, 2011

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of FirstEnergy Nuclear Operating Company (FENOC or the applicant) held a telephone conference call on September 16, 2011, to discuss and clarify the following response to requests for additional information (RAIs) concerning the Davis-Besse license renewal application (LRA).

Response to RAI B.2.22-7

Discussion:

The staff stated that during a telephone conference call held on September 13, 2011, the applicant agreed to submit a supplemental response to provide additional information related to examinations to monitor for cracking of the containment penetration sleeves, dissimilar metal welds, bellows, and steel components that are subject to cyclic loading. This supplemental response was to include frequency, sample size, basis for sample size, and to emphasize the use of Appendix J testing.

As a follow-up to the September 13, 2011, telephone conference call the staff requested the applicant to also address scheduling of the subject inspections. The applicant action item is updated as follows.

Action: The applicant to provide additional information related to containment penetration component cracking examination scheduling, frequency, sample size, basis for sample size, and to emphasize the use of Appendix J testing. In addition, the applicant to clarify that development of penetration fatigue analyses would remove the requirement to perform examinations for cracking. The applicant to provide a supplemental response to RAI B.2.22-7.

Response to RAI 4.1-2

Discussion:

In its response dated Aug 17, 2011, the applicant stated that the fracture toughness of the cast austenitic stainless steel is not time-dependent as the analysis used a lower bound fracture toughness of 139 ksi x in^{1/2} that bounds the saturated fracture toughness of the Davis-Besse material. The staff's concern is that the applicant's basis may be predicated on charpy or thermal aging data that are not up-to-date or conservative when compared to the most recent data for the state of the industry.

It is not clear to the NRC staff whether the assumption that "the lower bound fracture toughness of 139 ksi x in $^{1/2}$ that bounds the saturated fracture toughness of the applicant's materials" remains valid.

To address the staff's concern that the applicant's basis may be predicated on charpy or thermal aging data that are not up-to-date or conservative when compared to the most recent

data for the state of the industry, the applicant agreed to compare the thermal aging data used in the ASME Code Case N-481 evaluation to the most recent industry data (i.e., NUREG/CR-4513, Revision 1, "Estimation of Fracture Toughness of Cast Stainless Steels During Thermal Aging in LWR Systems," and NUREG/CR-6428, "Effects of Thermal Aging on Fracture Toughness and Charpy-Impact Strength of Stainless Steel Pipe Welds"), and provide the results in a supplemental response to RAI 4.1-2.

Action: The applicant to provide a supplemental response to RAI 4.1-2.

Other Follow-Up Items

1) LRA Section 4.7.3 – The applicant determined that the fracture mechanics analysis that evaluated the integrity of the reactor vessel against pressurized thermal shock (PTS) for 52 effective full power years (EFPY) considering the 35°F minimum temperature for the borated water storage tank is not docketed. In addition, Section 5.2 of the Updated Safety Analysis Report (USAR) has not been updated at this time with the results of the analysis. The applicant stated that the results are included in approved USAR Change Notice 09-183.

Action: The applicant to provide the USAR changes from USAR Change Notice 09-183 in a supplemental response.

- 2) The staff requested the applicant to identify the location in the LRA of the USAR supplement (Appendix A)for LRA Section 4.7.5.1 The applicant stated that the applicable LRA Appendix A section is A.2.6.1. The staff stated that no other information was needed.
- 3) The staff requested the applicant to identify the location in the LRA of the USAR supplement (Appendix A) for Section 4.7.5.2 The applicant stated that applicable LRA Appendix A section is A.2.6.2. The staff stated that no other information was needed.

There was no further discussion, and the call was concluded.

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