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Docket No. 50-387

FEB 13 2012

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Stop OP1-17 Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 50-387/2011-004-00 LICENSE NO. NPF-14 PLA-6812

Attached is Licensee Event Report (LER) 50-387/2011-004-00. The event involved the inoperability of the 'C' Emergency Diesel Generator. This event was determined to be reportable under 10 CFR 50.73(a)(2)(i)(B) as a condition that was prohibited by Technical Specifications.

There were no actual consequences to the health and safety of the public as a result of this event.

No regulatory commitments are associated with this LER.

Sincerely,

Attachment

Copy: NRC Region I

Mr. P. W. Finney, NRC Sr. Resident Inspector

Mr. R. R. Janati, DEP/BRP

Mr. B. K. Vaidya, NRC Project Manager

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4. TITLE "C" Eme	rger	ncy Die	sel Ge	nera	tor Inc	pera	able								
5. EVENT DATE 6. LER NUMBER 7. REPORT D								ATE	8. OTHER FACILITIES INVOLVED						
	DAY	YEAR	YEAR	SEQU	JENTIAL MBER	REV NO.	MONTH	DAY	YEAR	FACILITY Susqu	Y NAME	NAME DOCKET NO 050003 anna Steam Electric Station Unit 2			
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9. OPERAT	ING I	IODE	11	. THIS	REPOF	RT IS S	SUBMITTE	D PURS	UANT TO	THE RE	QUIREM	ENTS OF 10	CFR§: (Chec	k all that a	apply)
1			☐ 20.2201(b) ☐ 20.2201(d) ☐ 20.2203(a)(1) ☐ 20.2203(a)(2)(i) ☐ 20.2203(a)(2)(ii)				20.2203(a)(3)(i) 20.2203(a)(3)(ii) 20.2203(a)(4) 50.36(c)(1)(i)(A)			$\begin{array}{c cccc} & 50.73(a)(2)(i)(C) & & 50.73(a)(2)(vii) \\ \hline & 50.73(a)(2)(ii)(A) & & 50.73(a)(2)(viii)(A) \\ \hline & 50.73(a)(2)(ii)(B) & & 50.73(a)(2)(viii)(B) \\ \hline & 50.73(a)(2)(iii) & & 50.73(a)(2)(ix)(A) \\ \hline & 50.73(a)(2)(iv)(A) & & 50.73(a)(2)(x) \end{array}$					(B)
10. POWER LEVEL			☐ 20.22 ☐ 20.22 ☐ 20.22	203(a) 203(a) 203(a) 203(a) 203(a)	(2)(iii) (2)(iv) (2)(v)		☐ 50.36(c)(1)(ii)(A) ☐ 50.36(c)(2) ☐ 50.46(a)(3)(ii) ☐ 50.73(a)(2)(i)(A) ☑ 50.73(a)(2)(i)(B)			☐ 50.73(a)(2)(v)(A) ☐ 73.71 ☐ 50.73(a)(2)(v)(B) ☐ 73.71 ☐ 50.73(a)(2)(v)(C) ☐ OTHE ☐ 50.73(a)(2)(v)(D) Specif			(a)(4) (a)(5)		
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14. SUPPLEMENTAL REPORT EXPECTED ☐ YES (If yes, complete 15. EXPECTED SUBMISSION DATE)						⊠ N			IISSION	MONTH	DAY	YEAR			
ABSTRACT											n "C" Em	organov Di	anal Canarat	or (EDC)	

On December 6, 2011, Susquehanna Steam Electric Station declared the common "C" Emergency Diesel Generator (EDG) (EIIS: EK) inoperable due to loss of firing from cylinder 8R during surveillance testing. LCO 3.8.1 was entered, the "E" Emergency Diesel Generator was substituted for the "C" EDG and the LCO exited. The direct cause of the loss of firing was due to interruption of the spray pattern in the fuel injection nozzle and partial blockage. The root causes were determined to be 1) the work package to install the delivery valve springs was insufficient, 2) the work crew proceeded using an inadequate work package and 3) Quality Control activities were insufficient to prevent the incorrect reassembly of the fuel injector pump components. Immediate corrective action was to replace the 12 fuel injection pumps. Additional corrective actions include revision to the procedure on work package standards, reinforcement of stopping work when the work package is inadequate, and revision to the procedure on Quality Control Inspection Program to include guidance on construction, formatting, wording and use of notes and verification steps in hold point development.

A review of past maintenance on the "C" EDG determined that the EDG was inoperable from the time maintenance was performed on September 21, 2011 until it was shutdown on December 6, 2011 because it could not have fulfilled its mission time. This event is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

There were no actual adverse consequences to the health and safety of the public as a result of this event.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION

(10-2010)

LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET		6. LER NUMBER	3. PAGE		
Susquehanna Steam Electric Station Unit 1	05000387	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3	
·	0000007	2011	- 004 -	00	2013	

NARRATIVE

EVENT DESCRIPTION

On December 6, 2011, Susquehanna Steam Electric Station declared the common "C" Emergency Diesel Generator (EDG) (EIIS: EK) inoperable due to loss of firing from cylinder 8R during surveillance testing. LCO 3.8.1 was entered, the "E" Emergency Diesel Generator was substituted for the "C" EDG and the LCO exited. A review of past maintenance on the "C" EDG determined that the EDG was inoperable from the time maintenance was performed on September 21, 2011 until it was shutdown on December 6, 2011 because it could not have fulfilled its mission time.

CAUSE OF THE EVENT

The direct cause of the loss of firing was due to interruption of the spray pattern in the fuel injection nozzle and partial blockage. The root causes were determined to be:

- 1) the work package to install the delivery valve springs was insufficient,
- 2) the work crew proceeded using an inadequate work package, and
- 3) Quality Control (QC) activities were insufficient to prevent the incorrect reassembly of the fuel injector pump components.

ANALYSIS/SAFETY SIGNIFICANCE

This event is being reported in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications in that the "C" Emergency Diesel Generator could not fulfill its mission time in the event it was called upon to do so.

Actual Consequences

"C" EDG was unknowingly inoperable and unavailable to perform its 24 hour Probability Risk Assessment (PRA) mission time and its 30 day design bases mission time for a period of approximately 76 days. As a result, there were short periods of time when both units were unknowingly in elevated risk conditions without taking associated risk management actions.

Potential Consequences:

The potential consequence of the unavailability of the "C" EDG is an increase in core damage frequency and large early release frequency in the event of a Loss of Offsite Power combined with random failures of the remaining available EDGs, ultimately resulting in a high pressure containment challenge for the units. The increase in core damage frequency was less than 1.0 E-6 and the increase in large early release frequency was less than 1.0 E-7.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) (10-2010) **CONTINUATION SHEET** 1. FACILITY NAME 6. LER NUMBER 2. DOCKET 3. PAGE REVISION NUMBER SEQUENTIAL NUMBER YEAR Susquehanna Steam Electric Station Unit 1 05000387 3 OF 3 2011 --004--00

NARRATIVE

CORRECTIVE ACTIONS

The following corrective actions have been completed:

- The 12 fuel injection pumps, which contained the incorrectly installed delivery springs, have been replaced.
- The procedure on Work Package development has been revised to provide a clear standard for Human Factoring
 of work instructions.

The following corrective actions are planned:

- Revise the procedure governing Work Package Standards such that work packages are properly Human Factored and verified.
- Ensure that Human Factoring is part of the Maintenance Planner Qualification.
- Prior to performing work on of the remaining EDGs ensure the work packages meet the updated Human Factoring /Procedure Writing standards.
- Revise the procedure on "Hold and Notification Point Inspection and Documentation," to include guidance
 on the construct, formatting, wording, and use of notes and inspection steps in hold and notification point
 development. Ensure this guidance is consistent with procedure standards that have been established.
- Develop and implement an action plan to determine if potentially latent broken delivery valve springs exist on the A, D, or E EDGs. If broken springs are found in the investigation, then correct the condition.

No regulatory commitments are associated with this report.

ADDITIONAL INFORMATION

Failed Component Information:

Component: Delivery spring associated with the fuel injector pump

Manufacturer: Bendix

Previous Similar Events:

None