

September 20, 1995

Mr. William J. Cahill, Jr.
Chief Nuclear Officer
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

SUBJECT: ISSUANCE OF AMENDMENT FOR INDIAN POINT NUCLEAR GENERATING
UNIT NO. 3 (TAC NO. M92994)

Dear Mr. Cahill:

The Commission has issued the enclosed Amendment No. 163 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated July 21, 1995.

The amendment revises TS Section 6.0 (Administrative Controls) to replace the title-specific list of members on the Plant Operating Review Committee (PORC) with a more general statement of membership requirements. The scope of disciplines represented on the PORC has also been expanded to include nuclear licensing and quality assurance. The amendment also changes the title "Resident Manager" to "Site Executive Officer." This title change is an administrative change that does not affect the reporting relationship, authority, or responsibility of the position.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

Original signed by:

Jefferey F. Harold, Acting Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 163 to
DPR-64
2. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: See attached sheet

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DATED: September 20, 1995

AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-64-INDIAN POINT UNIT 3

Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 20, 1995

Mr. William J. Cahill, Jr.
Chief Nuclear Officer
Power Authority of the State of New York
123 Main Street
White Plains, NY 10601

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Sincerely,

A handwritten signature in cursive script, reading "Jefferey F. Harold", is written over the typed name.

Jefferey F. Harold, Acting Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosures: 1. Amendment No. 163 to
DPR-64
2. Safety Evaluation

cc w/encs: See next page

William J. Cahill, Jr.
Power Authority of the State
of New York

Indian Point Nuclear Generating
Station Unit No. 3

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated July 21, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Ledyard B. Marsh, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 20, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

6-1
6-3
6-6
6-7
6-8
6-13

Insert Pages

6-1
6-3
6-6
6-7
6-8
6-13

Revise Appendix B, Part II as follows:

Remove Pages

5-5
5-6

Insert Pages

5-5
5-6

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

- 6.1.1 The Site Executive Officer shall be responsible for overall facility operation. During periods when the Site Executive Officer is unavailable, one of the General Managers will assume his responsibilities or the Site Executive Officer may delegate this responsibility to other qualified supervisory personnel.

6.2 ORGANIZATION

6.2.1 Facility Management and Technical Support

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a) Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR.
- b) The Site Executive Officer shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c) The Chief Nuclear Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

- g) Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the unit is operating. (Operating personnel are defined as on shift individuals holding Senior Reactor Operator or Reactor Operator licenses, nuclear plant operators, shift technical advisors and shift contingency health physicists, I&C and maintenance personnel.) However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification on a temporary basis the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Site Executive Officer or his designee, or higher levels of management, in accordance with established procedures.

- h) At least one individual holding a Senior Reactor Operator (SRO) license shall be on duty in the Control Room at all times.
- i) The Assistant Operations Manager and Shift Manager shall hold a Senior Reactor Operator (SRO) license. The Operations Manager shall either hold an SRO license or shall have held an SRO license at Indian Point Unit 3'.

* For the period ending three years after restart from the 1993/1994 Performance Improvement Outage, the Operations Manager will be permitted to have held an SRO license at a Pressurized Water Reactor other than Indian Point Unit 3.

- 6.5.0.2 Each program and procedure required by Technical Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, shall be reviewed by a minimum of two designated technical reviewers who are knowledgeable in the affected functional area.
- 6.5.0.3 Designated technical reviewers shall meet or exceed the qualifications described in Section 4 of ANSI N18.1-1971 for applicable positions, with the exclusion of the positions identified in Sections 4.3.2 and 4.5. Individuals whose positions are described in Sections 4.3.2 and 4.5 may qualify as designated technical reviewers provided they meet the qualifications described in other portions of Section 4.
- 6.5.0.4 The designated technical reviewers shall determine the need for cross disciplinary reviews. Individuals performing cross disciplinary reviews shall meet the same qualifications required for designated technical reviewers.
- 6.5.0.5 Each program and procedure required by Technical Specification 6.8 and other procedures that affect nuclear safety, and changes thereto, shall be reviewed from a safety perspective by a qualified safety reviewer. Safety and/or environmental impact evaluations, when required, shall be reviewed by PORC per Technical Specification 6.5.1.6.a.
- 6.5.0.6 Nuclear safety related procedures and procedure changes shall be reviewed and approved, prior to implementation, by the appropriate member(s) of management.
- 6.5.0.7 All changes to the Process Control Program (PCP) and the Offsite Dose Calculation Manual (ODCM) shall be reviewed by PORC and accepted by the Site Executive Officer prior to implementation.

6.5.1 PLANT OPERATING REVIEW COMMITTEE (PORC)

FUNCTION

- 6.5.1.1 The Plant Operating Review Committee shall function to advise the Site Executive Officer on all matters related to nuclear safety and all matters which could adversely change the plant's environmental impact.

COMPOSITION

- 6.5.1.2 The Plant Operating Review Committee shall be composed of a Chairman, Vice Chairman and members designated in writing by the Site Executive Officer. Members shall collectively have responsibility in at least the areas of operations, maintenance, radiation safety, chemistry, engineering, instrumentation and controls, reactor engineering, nuclear licensing, and quality assurance.

ALTERNATES

- 6.5.1.3 All alternate members shall be appointed in writing by the (PORC) Chairman to serve on a temporary basis.

MEETING FREQUENCY

- 6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate.

QUORUM

- 6.5.1.5 A quorum of the PORC shall consist of the Chairman or Vice Chairman, and at least five members including alternates. Vice-Chairmen may act as members when not acting as Chairman. A quorum shall contain no more than two alternates.

RESPONSIBILITIES

- 6.5.1.6 The Plant Operating Review Committee shall be responsible for:
- a. Review of 10 CFR 50.59 safety and environmental impact evaluations associated with procedures and programs required by Technical Specification 6.8, and changes thereto.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Operating License and Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.
 - f. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Site Executive Officer, who will forward the report to the Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the Safety Review Committee.
 - g. Review of all reportable events.
 - h. Review of facility operations to detect potential nuclear safety hazards.
 - i. Performance of special reviews, investigations or analyses and reports thereon as requested by the Site Executive Officer or the Chairman of the Safety Review Committee (SRC).

j. Deleted

k. Deleted

l. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Site Executive Officer and to the Safety Review Committee.

m. Review of the Fire Protection Program and implementing procedures.

AUTHORITY

6.5.1.7 The Plant Operating Review Committee shall:

- a. Recommend to the Site Executive Officer approval or disapproval of items considered under 6.5.1.6(a) through (e) above.
- b. Render determinations with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.
- c. Provide notification within 24 hours to the Chairman of the SRC and the Chief Nuclear Officer of disagreement between the PORC and the Site Executive Officer; however, the Site Executive Officer shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and the Chief Nuclear Officer.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control

- b. Each REPORTABLE EVENT shall be reviewed by the PORC and a report submitted by the Site Executive Officer to the Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36(c)(1)(i).
- b. The Safety Limit Violation shall be reported immediately to the Commission. The Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC will be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared by the PORC. This report shall describe (1) applicable circumstances preceding the occurrences, (2) effects of the occurrence upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Chief Nuclear Officer, the Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC by the Site Executive Officer.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Process Control Program implementation.
- g. Offsite Dose Calculation Manual implementation.

5.4 RECORD RETENTION

Records associated with the radiological environmental monitoring program are to be retained as required by Appendix A Specification 6.10.2.

5.5 PROCESS CONTROL PROGRAM (PCP)

5.5.1 The PCP shall be approved by the Commission prior to implementation.

5.5.2 Licensee initiated changes to the PCP:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information;
 - b. A determination that the change did not reduce the overall conformance of the solidified waste product to existing criteria for solid wastes; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the PORC.
2. Shall become effective upon review and acceptance by the Site Executive Officer.

5.6 OFFSITE DOSE CALCULATION MANUAL (ODCM)

5.6.1 The ODCM shall be approved by the Commission prior to implementation.

5.6.2 Licensee initiated changes to the ODCM:

1. Shall be submitted to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the change(s) was made effective. This submittal shall contain:
 - a. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the changes(s);

- b. A determination that the change will not reduce the accuracy of reliability of dose calculations or setpoint determination; and
 - c. Documentation of the fact that the change has been reviewed and found acceptable by the PORC.
2. Shall become effective upon review and acceptance by the Site Executive Officer.

5.7 MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS*

Licensee initiated major changes to the radioactive waste systems (liquid, gaseous and solid):

1. Shall be reported to the Commission in the Semiannual Radioactive Effluent Release Report for the period in which the evaluation was reviewed by the PORC. The discussion of each shall contain:
 - a. A summary of the evaluation that led to the determination that the change could be made in accordance with 10 CFR Part 50.59.
 - b. Sufficient detailed information to totally support the reason for the change without benefit of additional or supplemental information;
 - c. A detailed description of the equipment, components and processes involved and the interfaces with other plant systems;
 - d. An evaluation of the change, which shows the predicted released of radioactive materials in liquid and gaseous effluents and/or quantity of solid waste that differ from those previously predicted in the license application and amendments thereto;
 - e. An evaluation of the change, which shows the expected maximum exposures to individual in the UNRESTRICTED AREA and to the general population that differ from those previously estimated in the license application and amendments thereto;
 - f. A comparison of the predicted releases of radioactive materials, in liquid and gaseous effluents and in solid waste, to the actual releases for the period prior to when the changes are to be made;
 - g. An estimate of the exposure to plant operating as a result of the change; and

* The information called for in this Specification will be submitted as part of the annual FSAR update.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 163 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

1.0 INTRODUCTION

By letter dated July 21, 1995, the Power Authority of the State of New York (the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 3 (IP3) Technical Specifications (TSs). The requested changes would revise TS Section 6.0 (Administrative Controls) to replace the title-specific list of members on the Plant Operating Review Committee (PORC) with a more general statement of membership requirements. The scope of disciplines represented on the PORC would also be expanded to include nuclear licensing and quality assurance. The amendment would also change the title "Resident Manager" to "Site Executive Officer."

2.0 EVALUATION

2.1 Plant Operating Review Committee Changes:

The current PORC is comprised of the following: (1) Chairman: General Manager - Operations, (2) Vice-Chairman: General Manager - Maintenance or General Manager - Support Services, and (3) Members: Operations Manager, Technical Services Manager, Maintenance Manager, Instrument & Controls Manager, Radiological and Environmental Services Manager, Site Engineering Manager. The Chairman or a Vice-Chairman and five members constitute a quorum. The proposed revision would read as follows: "The Plant Operating Review Committee shall be composed of a Chairman, Vice Chairman and members designated in writing by the Site Executive Officer. Members shall collectively have responsibility in at least the areas of operations, maintenance, radiation safety, chemistry, engineering, nuclear licensing, and quality assurance." The quorum would remain the same.

The licensee indicated that changes in organization and position titles affect the list of designated PORC members. The proposed approach to replace the title specific positions for PORC membership with a description of membership requirements reduces the need for future license amendments. Specific individuals who have responsibility in these disciplines will be designated in writing by the Site Executive Officer. The addition of nuclear licensing and quality assurance disciplines on PORC will provide additional expertise and will provide an additional mechanism for the oversight function to be performed.

The NRC staff has reviewed the proposed changes to TS Section 6.0 resulting from the new PORC membership criteria and has concluded that they are acceptable since they meet the appropriate acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan and do not involve any changes that would reduce plant safety.

2.2 Management Title Changes:

In order to reflect changes in the licensee's organization and position titles, the licensee has proposed changing the title of "Resident Manager" to "Site Executive Office." The licensee has indicated that this title change would be an administrative change that does not affect the reporting relationship, authority, or responsibility of the position.

The NRC staff has reviewed the proposed changes to the TS resulting from the new management title and has concluded that they are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. Conicella

Date: September 20, 1995