

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555

Docket No. 50-286

February 23, 1984

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Docketing and Service Section Office of the Secretary of the Commission

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 Power Authority of the State of New York

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Two signed originals of the <u>Federal Register</u> Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ($_6$) of the Notice are enclosed for your use.

- □ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- □ Notice of Availability of Applicant's Environmental Report.
- □ Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- □ Notice of Availability of NRC Draft/Final Environmental Statement.
- □ Notice of Limited Work Authorization.

Enclosure:

- □ Notice of Availability of Safety Evaluation Report.
- □ Notice of Issuance of Construction Permit(s).
- □ Notice of Issuance of Facility Operating License(s) or Amendment(s).
- & Other: Order Confirming Licensee Commitments on Emergency Response Capability which authorizes the operation of the facility at steadystate power levels not in excess of 3025 megawatts thermal.

Division of Licensing ORB#1: DL

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UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of

POWER AUTHORITY OF THE STATE OF NEW YORK Docket No. 50-286

(Indian Point Nuclear Generating Unit No. 3)

ORDER CONFIRMING LICENSEE COMMITMENTS ON EMERGENCY RESPONSE CAPABILITY

I.

The Power Authority of the State of New York (the licensee) is the holder of Facility Operating License No. DPR-64 which authorizes the operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) at steady-state power levels not in excess of 3025 megawatts thermal. The facility is a pressurized water reactor (PWR) located in Westchester County, New York.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities and significant upgrading of emergency response capability based on the experience from the accident at TMI-2 and the official studies and investigations of the

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accident. The requirements are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements," and in Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability." Among these requirements are a number of items consisting of emergency response facility operability, emergency procedure implementation, addition of instrumentation, possible control room design modifications, and specific information to be submitted.

On December 17, 1982, a letter (Generic Letter 82-33) was sent to all licensees of operating reactors, applicants for operating licenses, and holders of construction permits enclosing Supplement 1 to NUREG-0737. In this letter operating reactor licensees and holders of construction permits were requested to furnish the following information, pursuant to 10 CFR 50.54(f), no later than April 15, 1983:

- A proposed schedule for completing each of the basic requirements for the items identified in Supplement 1 to NUREG-0737, and
- (2) A description of plans for phased implementation and integration of emergency response activities including training.

III.

The licensee responded to Generic Letter 82-33 by letters dated April 18, 1983, and August 23, 1983. In these submittals, the licensee made commitments to complete the basic requirements. The attached Table, summarizing the licensee's schedular commitments or status, was developed by the NRC staff from the Generic Letter and the information provided by the licensee. The licensee's commitments include (1) dates for providing required submittals to the NRC, (2) dates for implementing certain requirements, and (3) a schedule for providing implementation dates for other requirements. These latter implementation dates will be reviewed, negotiated and confirmed by a subsequent order.

The NRC staff reviewed the licensee's April 18, 1983 letter and entered into negotiations regarding schedules for meeting the requirements of Supplement 1 to NUREG-0737. Subsequently, the licensee submitted modified dates by letter dated August 23, 1983. The NRC staff finds that the dates are reasonable, achievable dates for meeting the Commission requirements. The NRC staff concludes that the schedule initially proposed by the licensee will provide timely upgrading of the licensee's emergency response capability.

In view of the foregoing, I have determined that the implementation of the licensee's commitments is required in the interest of the public health and safety and should, therefore, be confirmed by an immediately effective Order.

IV.

Accordingly, pursuant to Sections 103, 161i, 161o and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED, EFFECTIVE IMMEDIATELY, THAT the licensee shall:

Implement the specific items described in the licensee's submittals noted in Section III herein no later than the dates in the Attachement.

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Extensions of time for completing these items may be granted by the Director, Division of Licensing, for good cause shown.

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The licensee may request a hearing on this Order within 20 days of the date of publication of this Order in the <u>Federal Register</u>. Any request for a hearing should be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy should also be sent to the Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section IV of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Division of Likensing

Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland, this 21stday of February 1984.

Attachment: Licensee's Commitments on Requirements Specified in Supplement 1 to NUREG-0737

Attachment page 1 of 2

Cycle 4/5 reload Mid 1985

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LICENSEE'S COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737 a/

	TITLE	REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)	
1.	Safety Parameter Display System (SPDS)	la. Submit a safety analysis and an implementation plan to the NRC.	6 months prior to cycle 4/5 refueling outage	
		<pre>1b. SPDS fully operational and initial operator training complete.</pre>	Cycle 4/5 refueling outage	
2.	Detailed Control Room Design Review (DCRDR)	2a. Submit a program plan to the NRC.	9 months prior to cycle 4/5 refúeling outage	
	•	2b. Submit a summary report to the NRC including a proposed schedule for implementation.	Cycle 4/5 refueling outage	
3.	Regulatory Guide 1.97 - Application to Emergency Response Facilities	 3a. Submit a report to the NRC describing how the requirements of Supplement 1 to NUREG-0737 	June 30, 1984	
		<pre>have been or will be met. 3b. Implement (installation or upgrade) requirements.</pre>	Submit implementation schedule by June 30, 1984	

 \underline{a} Cycle 4/5 refueling outage is presently scheduled for mid-1985.

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LICENSEE'S COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

REQUIREMENT	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)	
4a. Submit a Procedures Generation Package to the NRC:	9 months prior to cycle 4/5 refueling outage	(
4b. Implement the upgraded EOPs.	Cycle 4/5 refueling outage	
5a. Technical Support Center fully functional.	Cycle 4/5 refueling outage	
5b. Operational Support Center fully functional.	Cycle 4/5 refueling outage	
5c. Emergency Operations Facility fully functional.	Cycle 4/5 refueling outage	(
	 4a. Submit a Procedures Generation Package to the NRC: 4b. Implement the upgraded EOPs. 5a. Technical Support Center fully functional. 5b. Operational Support Center fully functional. 5c. Emergency Operations Facility 	REQUIREMENTSCHEDULE (OR STATUS)4a. Submit a Procedures Generation Package to the NRC:9 months prior to cycle 4/5 refueling outage4b. Implement the upgraded EOPs.Cycle 4/5 refueling outage5a. Technical Support Center fully functional.Cycle 4/5 refueling outage5b. Operational Support Center fully functional.Cycle 4/5 refueling outage5c. Emergency Operations FacilityCycle 4/5 refueling outage

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