

MAR 23 1982

Docket No. 50-286

Mr. Leroy W. Sinclair, President
and Chief Operating Officer
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Sinclair:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 19, 1982.

The amendment on a one-time only basis extends the monthly turbine steam stop closure check until March 31, 1982. This basically extends the testing interval by 8 days.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

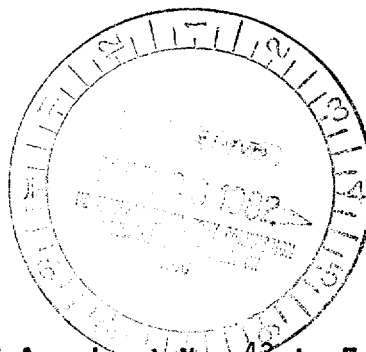
John O. Thoma, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 43 to DPR-64
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
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AMENDMENT

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Mr. Leroy W. Sinclair
Power Authority of the State of New York

cc: White Plains Public Library
100 Martine Avenue
White Plains, New York 10601

Mr. Charles M. Pratt
Assistant General Counsel
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Ms. Ellyn Weiss
Sheldon, Harmon and Weiss
1725 I Street, N.W., Suite 506
Washington, D. C. 20006

Dr. Lawrence R. Quarles
Apartment 51
Kendal at Longwood
Kennett Square, Pennsylvania 19348

Mr. George M. Wilverding, Chairman
Safety Review Committee
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Joan Holt, Project Director
New York Public Interest
Research Group, Inc.
5 Beekman Street
New York, New York 10038

Director, Technical Development
Programs
State of New York Energy Office
Agency Building 2
Empire State Plaza
Albany, New York 12223

Mr. John C. Brons, Resident Manager
Indian Point 3 Nuclear Power Plant
P. O. Box 215
Buchanan, New York 10511

Honorable George Begany
Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, New York 10511

Mr. J. P. Bayne, Senior Vice Pres.
Nuclear Generation
Power Authority of the State
of New York
Columbus Circle
New York, New York 10019

Theodore A. Rebelowski
Resident Inspector
Indian Point Nuclear Generating
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

Thomas J. Farrelly, Esquire
Law Department
Consolidated Edison Company of
New York Inc.
4 Irving Place
New York, New York 10003

Mr. A. Klausmann, Vice President
Quality Assurance
Power Authority of the State of
New York
10 Columbus Circle
New York, New York 10019

Regional Radiation Representative
EPA Region II
26 Federal Plaza
New York, New York 10007

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047

Mr. Leroy W. Sinclair
Power Authority of the State of New York

cc: Mr. D. Halama
Quality Assurance Superintendent
Indian Point 3 Nuclear Power Plant
Post Office Box 215
Buchanan, New York 10511

Ronald C. Haynes
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated March 19, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

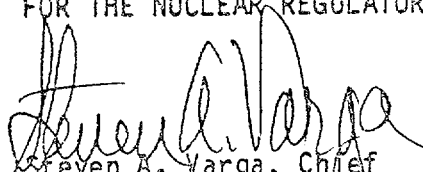
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 23, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 43

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-264

Revise Appendix A as follows:

Remove Page

Table 4.1-3

Insert Page

Table 4.1-3

TABLE 4.1-3
FREQUENCIES FOR EQUIPMENT TESTS

	<u>CHECK</u>	<u>FREQUENCY</u>
1. Control Rods	Rod drop times of all control rods	R
2. Control Rods	Partial movement of all control rods	Every 2 weeks during reactor critical operations
3. Pressurizer Safety Valves	Set point	R
4. Main Steam Safety Valves	Set point	R
5. Containment Isolation System	Automatic actuation	R
6. Refueling System Interlocks	Functioning	Prior to each refueling outage
7. Primary System Leakage	Evaluate	5 days/week
8. Diesel Fuel Supply	Fuel Inventory	Weekly
9. Turbine Steam Stop, Control Valves	Closure	Monthly ¹
10. L.P. Steam Dump System (6 lines)	Closure	Monthly
11. Service Water System	Each pump starts and operates for 15 minutes (unless already operating)	Monthly
12. City Water Connections to Charging Pumps and Boric Acid Piping	Temporary connections available and valves operable	R
13. BHR Valves 730 and 731	Automatic isolation and interlock action	R*
14. Block Valve	Operability through 1 complete cycle of full travel	R
15. PCRV Valves	Operability	R

R Each Refueling Outage
* If not done during the previous 18 months, the check will be performed the next time the plant is cooled down.

¹ Except for the test due on March 23, 1982, which will be performed no later than March 31, 1982.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

Introduction

The Indian Point Unit No. 3 Technical Specifications require performance of a monthly main turbine steam stop and control valve test. This test is due by March 23, 1982. By application dated March 19, 1982, the licensee requested a modification to this requirement to defer the test until March 31, 1982.

The basic reason for the request is to prevent possible damage to the main turbine. Westinghouse, the turbine manufacturer, has recommended specific steam flow rate ranges at which this test should be conducted for turbine safety. During a normal operating month the plant conditions can be altered to achieve the specific conditions as recommended by Westinghouse. However, during the month of March 1982 the licensee states that the specific plant conditions cannot be met without generating between 30,000 gallons and 60,000 gallons of radioactive waste and risking a possible premature plant shutdown. By license condition the plant will be shutdown by March 31, 1982 to commence a refueling outage.

Evaluation

Indian Point Unit No. 3 is being operated at reduced power to maintain a fuel burn-up consistent with the scheduled refueling outage starting date of March 31, 1982. This reduced power level is between the two specific steam flow rate ranges recommended by Westinghouse to conduct the main turbine steam stop and control valve test. To increase power to the upper specified range is not advisable due to other equipment being inoperable, i.e., one condensate pump is inoperable, and end of life core conditions may cause an unscheduled plant shutdown. The power level could be reduced to the lower range specified by Westinghouse but this action is not recommended. First, the boration and dilution of the reactor coolant system necessary to attain the lower steam flow rate and then return to the present power level would generate a significant amount of

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liquid radioactive waste solely to perform this test. Also, the plant manipulations required for temporarily reducing power increase the probability of a premature plant shutdown.

The basis for conducting a monthly main turbine steam stop and control valve test is to insure operability of these valves in a turbine overspeed condition. If the valves were to remain in a single position for an extended period of time, corrosive product buildup on the stem may prevent proper operation of the valve. Periodic cycling of this valve would remove the corrosive product buildup and allow early detection of potential problems. This test has been successfully performed thirteen times in the past 12 months. Any corrosive product buildup should be fairly minimal and extending the test interval by eight days does not significantly alter the basis for conducting this test. Also the generation of a significant amount of liquid radioactive waste solely for the purpose of conducting this test on March 23 vice March 31, 1982 is not justified. Therefore, we concur that there is sufficient justification to delay this test by eight days on a one time only basis.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 23, 1982

Principal Contributor:
John Thoma

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-286POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 43 to Facility Operating License No. DPR-64, issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective as of the date of issuance.

The amendment on a one-time only basis extends the monthly turbine steam stop closure check until March 31, 1982. This basically extends the testing interval by 8 days.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

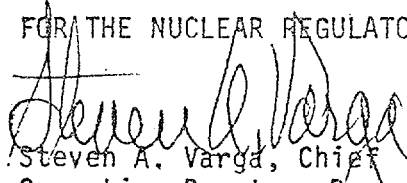
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

- 2 -

For further details with respect to this action, see (1) the application for amendment dated March 19, 1982, (2) Amendment No. 43 to License No. DPR-64, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing