

MAY 10 1982

Docket No. 50-286

Mr. Leroy W. Sinclair, President
and Chief Operating Officer
Power Authority of the State
of New York
10 Columbus Circle
New York, New York 10019



DISTRIBUTION
Docket
NRC PDR
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NSIC
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DEisenhut
OELD
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CParrish
JThoma
DBrinkman
LSchneider
RDiggs
OPA
ASLAB
TBarnhart-4
MConner
PStoddart
MConner
Gray

Dear Mr. Sinclair:

The Commission has issued the enclosed Amendment No. 44 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 1, 1982.

The amendment revises your Technical Specifications to require operability of a system to sample the plant effluent for indications of radiiodine and particulates. The amendment also provides a minor restructuring of Table 3.5-3 and Table 3.5-4 with no deletions or additions of new requirements.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

John O. Thoma, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 44 to DPR-64
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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PDR ADDCK 05000286
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FR NOTICE
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AMENDMENT

Previous concurrences see next page

OFFICE	ORB#1:DL	ORB#1:DL	ORB#3:DL	ETSB	ORB#1:DL	AD/OR:DL	OELD
SURNAME	CParrish	JThoma	MConner	PStoddart	SVarga	TNovak	J.Moore
DATE	04/1/82	04/27/82	04/ /82	04/ /82	04/ /82	04/ /82	05/5/82

DISTRIBUTION
Docket TBarnhart (4)
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Docket No. 50-286

Mr. Leroy W. Sinclair, President
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Power Authority of the State
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10 Columbus Circle
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Dear Mr. Sinclair:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. This amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated February 1, 1982.

The amendment revises your Technical Specifications to provide sampling of the plant effluent for indications of radioiodine and particulates. The amendment also provides a minor restructuring of Table 3.5-3 and Table 3.5-4 with no deletions or additions of new requirements.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

John O. Thoma, Project Manager
Operating Reactors Branch #1
Division of Licensing

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2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

WJ 868
#75B
P Stoddart
3/25/82

sent back for rewrite

OFFICE	ORB#1:DL	ORB#1:DL	ORB#3:DL	ORB#1:DL	AD/OR:DL	OELD	
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Mr. Leroy W. Sinclair
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated February 1, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

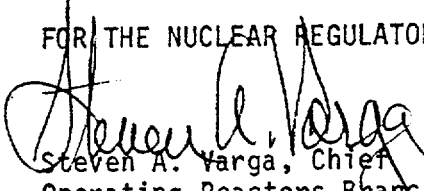
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 44, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 21 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 10, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

Table 3.5-3 (Sheet 3 of 3)
Table 3.5-4 (Sheet 1 of 2)
Table 3.5-4 (Sheet 2 of 2)
Table 4.1-1 (Sheet 4 of 4)
6-18

Insert Pages

Table 3.5-3 (Sheet 3 of 3)
Table 3.5-4 (Sheet 1 of 2)
Table 3.5-4 (Sheet 2 of 2)
Table 4.1-1 (Sheet 4 of 4)
6-18

TABLE 3.5-3 (SHEET 3 of 3)

FUNCTIONAL UNIT	1	2	3	4	5
3. AUXILIARY FEEDWATER					
a. Stm Gen. Water Level-Low-Low					
i. Start Motor Driven Pumps	3/stm. gen	2 in any stm gen.	2 chan. in each stm gen	1	Reduce system temperature such that $T \leq 350^{\circ}\text{F}$
ii. Start Turbine-Driven Pump	3/stm. gen	2/3 in each of two stm. gen.	2 chan. in each stm. gen.	1	$T \leq 350^{\circ}\text{F}$
b. S. I. Start Motor-Driven Pumps	(All safety injection initiating functions and requirements)				
c. Station Blackout Start Turbine-Driven Pump	2	1	1	0	$T \leq 350^{\circ}\text{F}$
d. Trip of Main Feedwater Pumps start Motor-Driven Pumps	2	1	1	0	Hot Shutdown
4. LOSS OF POWER					
a. 480 V Bus	2/Bus	1/bus	1/bus	0	Hot Shutdown

TABLE 3.5-4 (Sheet 1 of 2)

INSTRUMENT OPERATING CONDITIONS FOR ISOLATION FUNCTIONS

NO.	FUNCTIONAL UNIT	1	2	3	4	5
		NO. OF CHANNELS	NO. OF CHANNELS TO TRIP	MIN. OPERABLE CHANNELS	MIN. DEGREE OF REDUNDANCY	OPERATOR ACTION IF CONDITIONS IN COLUMN 3 OR 4 CANNOT BE MET
1.	CONTAINMENT ISOLATION					
	a. Automatic Safety Injection (Phase A)	See Item No. 1(b) of Table 3.5-3				Cold Shutdown (see note 1)
	b. Containment Pressure (Phase B)	See Item No. 2(b) of Table 3.5-3				Cold Shutdown (see note 1)
	c. Manual					
	Phase A	2	1	1	0	Cold Shutdown (see note 1)
	Phase B	See Item No. 2 (a) of Table 3.5-3				Cold Shutdown (see note 1)
2.	STEAM LINE ISOLATION					
	a. High Steam Flow in 2/4 Steam Lines Coincident with Low Tavg or Low Steam Line Pressure	See Item No. 1(e) of Table 3.5-3				Cold Shutdown and Main Steam Isolation Valves Closed (see note 1)
	b. High Containment Pressure (Hi Hi Level)	See Item No. 2(b) of Table 3.5-3				Cold Shutdown and Main Steam Isolation Valves Closed (see notes 1 and 2)
	c. Manual	1/loop	1/loop	1/loop	0	Cold Shutdown and Main Steam Isolation Valves Closed (see note 1)

TABLE 3.5-4 (Sheet 2 of 2)

	1	2	3	4	5
3. FEEDWATER LINE ISOLATION					
a. Safety Injection	See Item No. 1 of Table 3.5-3				
4. CONTAINMENT VENT AND PURGE					
a. Containment Radioactivity- High (R11 and R12 monitor)	2	1	1	0	close all containment vent and purge valves
5. PLANT EFFLUENT RADIODINE/ PARTICULATE SAMPLING (sample line common with monitor R13)	1	NA	1	0	(see note 3)

NOTES

1. If the conditions of Columns 3 or 4 cannot be met, the reactor shall be placed in the hot shutdown condition, utilizing normal operating procedures, within 4 hours of the occurrence. If the conditions are not met within 24 hours of the occurrence, the reactor shall be placed in the cold shutdown condition, or the alternate condition if applicable, within an additional 24 hours.
2. Main steam isolation valves may be closed in lieu of going to cold shutdown if the circuitry associated with closing the valves is the only portion inoperable.
3. If the plant vent sampling capability is determined to be inoperable when the reactor is above the cold shutdown condition, then restore the sampling capability within 72 hours or:
 - a) Initiate an alternate method of equivalent sampling capability and submit a 30 day report to NRC pursuant to Technical Specification 6.9.1.8, or if the alternate method cannot be established then,
 - b) Submit a Special Report to NRC pursuant to Technical Specification 6.9.2. within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

TABLE 4.1-1 (SHEET 4 of 4)

<u>CHANNEL</u> <u>DESCRIPTION</u>	<u>CHECK</u>	<u>CALIBRATE</u>	<u>TEST</u>	<u>REMARKS</u>
29. Reactor Coolant System Subcooling Margin Monitor	D	R	N.A.	
30. PORV Position Indicator (Limit Switch)	N.A.	R	R	
31. PORV Position Indicator (Acoustic Monitor)	D	R	R	
32. Safety Valve Position Indicator (Acoustic Monitor)	D	R	R	
33. Auxiliary Feedwater Flow Rate	N.A.	R	N.A.	
34. Plant Effluent Radioiodine/Particulate Sampling (sample line common with monitor R 13)	N.A.	N.A.	R	

S - Each Shift

D - Daily

W - Weekly

M - Monthly

P - Prior to each startup if not done previous week

Q - Quarterly

R - Each Refueling Outage

NA - Not Applicable

- d. Abnormal degradation of systems other than those specified in 6.9.1.7.c above designed to contain radioactive material resulting from the fission process. 7/

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Sealed source leakage on excess of limits (Specification 3.9)
- b. Inoperable Seismic Monitoring Instrumentation (Specification 4.10)
- c. Primary coolant activity in excess of limits (Specification 3.1.D)
- d. Seismic event analysis (Specification 4.10)
- e. Inoperable fire protection and detection equipment (Specification 3.14)
- f. The complete results of the steam generator tube inservice inspection (Specification 4.9.C)
- g. Inoperable plant vent sampling capability (Table 3.5-4 item 5)

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source and fission detector leak tests and results.
- h. Records of annual physical inventory of all source material of record.
- i. Records of reactor tests and experiments.

7/ Sealed sources or calibration sources are not included under this item. Leakage of packing, caskets, mechanical joints and seal welds within the limits for identified leakage set forth in technical specifications need not be reported under this item.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

Background

As a result of the NRC investigation of the Three Mile Island nuclear plant accident of March 1979, and as a result of the staff's evaluation of potential accidents which could occur at other nuclear plants, the NRC has established requirements for the monitoring and sampling of radioactive gaseous effluents under accident conditions. These requirements are detailed in NUREG-0737, Clarification of TMI Action Plan Requirements, November 1980, under Item II.F.1, Attachments 1 and 2. The provisions of Item II.F.1, Attachments 1 and 2, require that each operating nuclear power plant install additional radioactive effluent monitoring equipment with capacity sufficient to permit determination of effluent releases from all identified release points under worst-case accident conditions. Operators of nuclear power plants who determined that their proposed systems involved technical deviations from the guidance of NUREG-0737 were to document such deviations and to describe their alternative proposals. Operators who did not submit deviation requests or who did not submit requests for relief from NUREG-0737 requirements were considered, under the terms of the NUREG-0737 transmittal, to be in conformance with those requirements.

Discussion

The Power Authority of the State of New York (PASNY) has submitted information to the effect that the licensee's provisions to meet the requirements of Item II.F.1, Attachments 1 and 2, contained technical deviations (or clarifications) for Indian Point Unit 3.

In discussions and memoranda between PASNY, the staff, and the staff's technical assistance contractor, the deviations have been resolved to the satisfaction of the staff and, therefore, the deviations either no longer exist or have been judged to be an acceptable alternative to the guidance of NUREG-0737. According to information submitted by the licensee, radioiodine and particulate sampling equipment was installed and operational by January 1, 1982. By letter dated December 29, 1981, the licensee requested to defer the

installation of the Noble Gas monitor until the cycle 4/5 refueling outage (expected during the Fall of 1983). By letter of February 1, 1982, the licensee has submitted proposed modifications to the Technical Specifications for Indian Point 3 to require the operability of the sampling system for radioiodine and particulates in the plant effluent.

Evaluation

The staff has reviewed the licensee's submittals concerning equipment provisions for the implementation of NUREG-0737, Item II.F.1, Attachments 1 and 2. There are no unresolved deviations from the requirements or specifications of NUREG-0737, Item II.F.1, Attachments 1 and 2 with the exception of a schedular deviation which is not being considered in this evaluation.

Subject to after-the-fact on-site inspection of the installed equipment and evaluation of equipment performance under actual plant operational conditions, the staff finds the licensee's equipment provisions to be satisfactory.

The staff has reviewed the Indian Point 3 proposed Technical Specification changes for the operation and calibration of instrumentation to meet the requirements of NUREG-0737, Item II.F.1, Attachment 2. The Technical Specifications include action items requiring timely reporting to the NRC of instrument outages. The staff finds the licensee's proposed Technical Specification changes to be acceptable. The staff will review the licensee's proposed Technical Specifications when they are submitted concerning the noble gas monitor (NUREG-0737 Item II.F.1). The schedular deviation requested for Item II.F.1 is under consideration by the Commissioners directly and will be addressed in future correspondence.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 10, 1982

Principal Contributors:

P. Stoddart, ETSB

J. Thoma, ORR#1

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-286POWER AUTHORITY OF THE STATE OF NEW YORKNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 44 to Facility Operating License No. DPR-64, issued to the Power Authority of the State of New York (the licensee), which revised Technical Specifications for operation of the Indian Point Nuclear Generating Unit No. 3 (the facility) located in Buchanan, Westchester County, New York. The amendment is effective 21 days from the date of issuance.

The amendment revises the plant Technical Specifications to require operability of a system to sample the plant effluent for indications of radioiodine and particulates. The amendment also provides a minor restructuring of Table 3.5-3 and Table 3.5-4 with no deletions or additions of new requirements.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

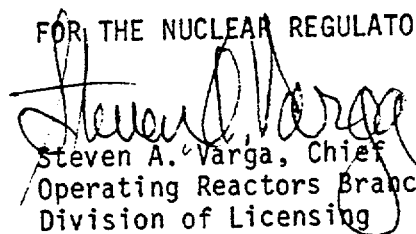
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

- 2 -

For further details with respect to this action, see (1) the application for the amendment dated February 1, 1982, (2) Amendment No. 44 to License No. DPR-64, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the White Plains Public Library, 100 Martine Avenue, White Plains, New York. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of May, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing