

June 5, 1990

Docket No. 50-286

Mr. John C. Brons
Executive Vice President - Nuclear Generation
Power Authority of the State of New York
123 Main Street
White Plains, New York 10601

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Dear Mr. Brons:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 76467)

The Commission has issued the enclosed Amendment No. 99 to Facility Operating License No. DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 23, 1990.

The amendment revises Technical Specification Table 4.1-3, "Frequencies for Equipment Tests." The change would revise testing to specify control rod movement of at least ten steps to assure control rod movement. A two week surveillance interval is also changed to 31 days.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Original signed by

Joseph D. Neighbors, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 99 to DPR-64
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1
CVogan
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PDI-1
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DOCUMENT NAME: ISSUANCE OF AMENDMENT 76467

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Mr. John C. Brons
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cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated March 23, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 5, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 99

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

Table 4.1-3

Insert Pages

Table 4.1-3 (Sheet 1 of 2)

Table 4.1-3 (Sheet 2 of 2)

TABLE 4.1-3 (Sheet 1 of 2)

FREQUENCIES FOR EQUIPMENT TESTS			
		<u>Check</u>	<u>Frequency</u>
1.	Control Rods	Rod drop times of all control rods	R
2.	Control Rods	Movement of at Least 10 steps in any one direction of all control rods	Every 31 days during reactor critical operations
3.	Pressurizer Safety Valves	Set Point	R
4.	Main Steam Safety Valves	Set Point	R
5.	Containment Isolation System	Automatic actuation	R
6.	Refueling System Interlocks	Functioning	R (Prior to movement of core components)
7.	Primary System Leakage	Evaluate	5 days/week
8.	Diesel Generators Nos. 31, 32, & 33 Fuel Supply	Fuel Inventory	Weekly
9.	Turbine Steam Stop Control Valves	Closure	Monthly
10.	L.P. Steam Dump System (6 lines)	Closure	Monthly
11.	Service Water System	Each pump starts and operates for 15 minutes (unless already operating)	Monthly
12.	City Water Connections to Charging Pumps and Boric Acid Piping	Temporary connections available and valves operable	R

Table 4.1-3 (Sheet 2 of 2)

13.	RHR Valves 730 and 731	Automatic isolation and interlock action	R*
14.	PORV Block Valves	Operability through 1 complete cycle of full travel	R
15.	PORV Valves	Operability	R
16.	Reactor Vessel Head Vents	Operability	R

R Each Refueling Outage

* If not done during the previous 18 months, the check will be performed next time the plant is cooled down.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-64
POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286

INTRODUCTION

By letter dated March 23, 1990, the Power Authority of the State of New York (the licensee) requested an amendment to Facility Operating License DPR-64 for the Indian Point Nuclear Generating Unit No. 3. The change would revise the Technical Specifications to specify control rod movement of at least ten steps to assure control rod freedom of movement and change the surveillance interval from two weeks to 31 days.

EVALUATION

Discussion and Evaluation

The present Technical Specifications require surveillance tests of all control rods for "partial movement," but do not specify the amount of movement. The proposed change would specify control rod movement of at least ten steps in any one direction, thus clarifying the minimum number of steps required to assure control rod freedom of movement.

The proposed change from a nominal two week surveillance interval to a nominal 31 day surveillance interval would reduce the mechanical wear on the drive mechanisms and reduce the wear on rod control cluster cladding caused by more frequent rod insertions.

We have reviewed the licensee's submittal and agree that specifying the minimum number of steps required in order to assure control rod freedom of movement is desirable. This change will enhance the clarity of the Technical Specifications, decrease the likelihood of misunderstanding, and ensure that the rods are moved an adequate amount.

Indian Point 3 has never found an immovable control rod during movement exercise. Never has a control rod failed to go to the fully inserted position when required. In addition, the staff concludes that the change in surveillance testing frequency is not likely to have a significant impact on plant safety from an overall risk assessment standpoint. Based on this, we agree that changing from a two week to the 31 day surveillance interval remains conservative. In addition, these changes are consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors. Therefore, we find the proposed changes to be acceptable.

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ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 5, 1990

PRINCIPAL CONTRIBUTOR:

J. D. Neighbors