

November 18, 1987

Docket No. 50-247

Mr. Murray Selman
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, New York 10511

Dear Mr. Selman:

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The Commission has issued the enclosed Amendment No. 128 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated May 29, 1987, as supplemented August 3, and September 30, 1987. (TAC 65624)

The amendment revises the Technical Specifications to permit the Residual Heat Removal pumps to remain operable during the performance of the Safety Injection Test. The change was proposed to facilitate outage planning.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Marylee M. Slosson, Project Manager
Project Directorate I-1
Division of Reactor Projects, I/II

Enclosures:

1. Amendment No. 128 to DPR-26
2. Safety Evaluation

cc: w/enclosures
See next page

PDI-1 *w*
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Mr. Murray Selman
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of New York, Inc.

Indian Point Nuclear Generating
Station 1/2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 128
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated May 29, 1987, as supplemented August 3, and September 30, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 128, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects, I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Revise Appendix A as follows:

Remove Page

4.5-1

Insert Page

4.5-1

4.5 ENGINEERED SAFETY FEATURES

Applicability

Applies to testing of the Safety Injection System, the Containment Spray System, the Hydrogen Recombiner System, and the Air Filtration System.

Objective

To verify that the subject systems will respond promptly and perform their design functions, if required.

Specification

A. SYSTEM TESTS

1. Safety Injection System

- a. System tests shall be performed at each reactor refueling interval. With the Reactor Coolant System pressure less than or equal to 350 psig and temperature less than or equal to 350°F, a test safety injection signal will be applied to initiate operation of the system. The safety injection pumps are made inoperable for this test.
- b. The test will be considered satisfactory if control board indication and visual observations indicate that all components have received the safety injection signal in the proper sequence and timing, that is the appropriate pump breakers shall have opened and closed, and the appropriate valves shall have completed their travel.
- c. Conduct a flow test of the high head safety injection system after any modification is made to either its piping and/or valve arrangement.
- d. Verify that the mechanical stops on Valve 856 A,C,D & E are set at the position measured and recorded during the most recent ECCS operational flow test or flow tests performed in accordance with (c) above. This surveillance procedure shall be performed following any maintenance on these valves or their associated motor operators and at a convenient outage if the position of the mechanical stops have not been verified in the preceding three months.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

INTRODUCTION

By letter dated May 29, 1987, as supplemented August 3, 1987, Consolidated Edison submitted a proposed amendment to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications to permit the Residual Heat Removal Pumps to remain operable during the performance of the Safety Injection System Test. The affected Technical Specification is 4.5.A.1. By letter dated September 30, 1987 Consolidated Edison provided additional information in support of the amendment application. This submittal did not affect the technical content of the amendment application in any way.

DISCUSSION AND EVALUATION

The current Indian Point 2 Technical Specifications state that during the performance of the Safety Injection System Test the Residual Heat Removal Pumps are made inoperable. Residual Heat Removal (RHR) pumps are each tested quarterly during reactor operation. It was considered acceptable to block them from starting during the system actuation test conducted during refueling outages because of the frequent testing during normal operation. By blocking the pumps the potential wear associated with frequent testing is minimized.

The Indian Point 2 Technical Specifications do not specify when the Safety Injection Test should be performed during the refueling interval provided temperature and pressure limits specified are met. However, in order to not exceed RCS temperature condition limits the test has been performed late in the refueling outage sequence. This permits sufficient time for de-energizing the RHR pumps, performing the system test, and re-energizing the RHR pumps when the decay heat load is relatively low and RHR is not required.

By letter dated May 29, 1987, as supplemented August 3, and September 30, 1987, Consolidated Edison proposed a revision to the Technical Specifications to allow one RHR pump to remain operable during the Safety Injection Test. This will allow one train of RHR to perform its decay heat removal function while testing the Safety Injection train containing the other RHR pump. This will allow the test to be performed early in the outage while the decay heat load is high.

Upon initiation of a safety injection signal all operating safeguards equipment is stripped and resequenced onto its respective busses in the safeguards mode. As a result, in order to maintain RHR during the test, the RHR pump which is not in the test mode must be blocked from stripping during the test. This

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is accomplished by removal of the DC control fuse. The other RHR pump is placed in the test mode and will not start during performance of the test. While in the test mode, it is observed whether the electrical circuitry performs as required upon receipt of the Safety Injection signal. When testing is completed and the RHR pump in the test mode is returned to service, it will then serve the decay heat removal load and the other RHR pump will be placed in the test mode. Another testing sequence will then be initiated without disrupting decay heat removal capability.

With the proposed change, the Safety Injection System Test will be performed in virtually the same manner as the current Technical Specifications for the train that is in the test mode. Upon completion of the test, the DC control fuses will be replaced and independently verified to be replaced. In the unlikely event that the operating RHR pump fails, a person is stationed in the 480 volt switch gear room to facilitate starting of the RHR pump in the test mode.

Based on the above, the staff concludes that the proposed amendment is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the surveillance of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

PRINCIPAL CONTRIBUTOR:

M. Slosson

Dated: November 18, 1987