

May 1, 1996

Mr. J. E. Cross
Senior Vice President and
Chief Nuclear Officer
Nuclear Power Division
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. M94428
AND M94429)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 199 to Facility Operating License No. DPR-66 and Amendment No. 81 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2). These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 27, 1995.

These amendments modify Tables 3.3-11 and 4.3-7 of BVPS-1 and BVPS-2 TS 3.3.3.8 (Accident Monitoring Instrumentation) such that only one valve position indication system for the power-operated relief valves and safety valves is required to be operable. Minor editorial changes to BVPS-1 TS 3.3.3.8 and its associated Action Statements are also being made. These changes make the requirements of TS 3.3.3.8 consistent with the NRC's Improved Standard Technical Specifications (NUREG-1431, Revision 1) and with the guidance of Regulatory Guide 1.97, NUREG-0578, and NUREG-0737.

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
/s/

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334/412

- Enclosures: 1. Amendment No. 199 to
License No. DPR-66
2. Amendment No. 81 to
License No. NPF-73
3. Safety Evaluation

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DOCUMENT NAME: BVM94428.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Senior Vice President and
Chief Nuclear Officer
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Duquesne Light Company
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AND M94429)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. ¹⁹⁹ to Facility Operating License No. DPR-66 and Amendment No. ⁸¹ to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2). These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated December 27, 1995.

These amendments modify Tables 3.3-11 and 4.3-7 of BVPS-1 and BVPS-2 TS 3.3.3.8 (Accident Monitoring Instrumentation) such that only one valve position indication system for the power-operated relief valves and safety valves is required to be operable. Minor editorial changes to BVPS-1 TS 3.3.3.8 and its associated Action Statements are also being made. These changes make the requirements of TS 3.3.3.8 consistent with the NRC's Improved Standard Technical Specifications (NUREG-1431, Revision 1) and with the guidance of Regulatory Guide 1.97, NUREG-0578, and NUREG-0737.

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Donald S. Brinkman, Senior Project Manager
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Office of Nuclear Reactor Regulation

Docket Nos. 50-334/412

Enclosures: 1. Amendment No. 199 to
License No. DPR-66
2. Amendment No. 81 to
License No. NPF-73
3. Safety Evaluation

cc w/encls: See next page

J. E. Cross
Duquesne Light Company

Beaver Valley Power Station
Units 1 & 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 199
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated December 27, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 199, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Donald A. Birkman
for John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 199

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-50

3/4 3-51

3/4 3-52

Insert

3/4 3-50

3/4 3-51

3/4 3-52

DPR-66
INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.8 The accident monitoring instrumentation channels shown in Table 3.3-11 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours (follow Specification 3.4.11 when determining ACTIONS for Items 5 and 6).
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.8 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-11

DPR-66

ACCIDENT MONITORING INSTRUMENTATION

	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Pressurizer Water Level	3	2
2. Auxiliary Feedwater Flow Rate	1 per steam generator	1 per steam generator
3. Reactor Coolant System Subcooling Margin Monitor	1	1
4. Deleted		
5. PORV Limit Switch Position Indicator	1/valve	0/valve
6. PORV Block Valve Limit Switch Position Indicator	1/valve	0/valve
7. Safety Valve Acoustical Detector Position Indicator	2/valve*	1/valve
8. Deleted		
9. Containment Sump Wide Range Water Level	2	1
10. Containment Wide-Range Pressure	2	0
11. In-Core Thermocouples (Core-Exit Thermocouples)	4/core quadrant	2/core quadrant
12. Reactor Vessel Level Indicating System	1	1

* One Detector Active, Second Detector Passive

TABLE 4.3-7

DPR-66

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Pressurizer Water Level	M	R
2. Auxiliary Feedwater Flow Rate	S/U ⁽¹⁾	R
3. Reactor Coolant System Subcooling Margin	M	R
4. Deleted		
5. PORV Limit Switch Position Indicator	M	R
6. PORV Block Valve Limit Switch Position Indicator	M	R
7. Safety Valve Acoustical Detector Position Indicator	M	R
8. Deleted		
9. Deleted		
10. Containment Sump Wide-Range Water Level	M	R
11. Containment Wide-Range Pressure	N/A	R
12. In-Core Thermocouples (Core-Exit Thermocouples)	M	R
13. Reactor Vessel Level Indicating System	M	R

- (1) Channel check to be performed in conjunction with Surveillance Requirement 4.7.1.2.c following an extended plant outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated December 27, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DFCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION,

Donald A. Brinkman
for John F. Stolz, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 1, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 81

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-58

3/4 3-59

Insert

3/4 3-58

3/4 3-59

TABLE 3.3-11
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
1. Pressurizer Water Level	3	2	a, b
2. Auxiliary Feedwater Flow Rate	2 per steam generator	1 per steam generator	a, b
3. Reactor Coolant System Subcooling Margin Monitor	2	1	c
4. PORV Limit Switch Position Indicator	1/valve	0/valve	a, b
5. PORV Block Valve Limit Switch Position Indicator	1/valve	0/valve	a, b
6. Safety Valve Position Indicator	1/valve	0/valve	a, b
7. Deleted			
8. Containment Sump Wide Range Water Level	2	1	a, b
9. Containment Wide-Range Pressure	2	1	a, b
10. Reactor Vessel Level Indication System	2	1	a, b
11. Core Exit Thermocouples	4/core quadrant	2/core quadrant	a, b

TABLE 4.3-7
ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1.	Pressurizer Water Level	M	R
2.	Auxiliary Feedwater Flow Rate	S/U*	R
3.	Reactor Coolant System Subcooling Margin Monitor	M	R
4.	PORV Limit Switch Position Indicator	M	R
5.	PORV Block Valve Limit Switch Position Indicator	M	R
6.	Safety Valve Position Indicator	M	R
7.	Deleted		
8.	Containment Sump Wide-Range Water Level	M	R
9.	Containment Wide-Range Pressure	N/A	R
10.	Reactor Vessel Level Indication System	M	R
11.	Core Exit Thermocouples	M	R

* Channel check to be performed in conjunction with Surveillance Requirement 4.7.1.2.b following an extended plant outage.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 199 AND 81 TO FACILITY OPERATING
LICENSE NOS. DPR-66 AND NPF-73
DUQUESNE LIGHT COMPANY
OHIO EDISON COMPANY
PENNSYLVANIA POWER COMPANY
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY
THE TOLEDO EDISON COMPANY
BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letter dated December 27, 1995, the Duquesne Light Company (the licensee) submitted a request for changes to the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2), Technical Specifications (TSs). The requested changes would modify Tables 3.3-11 and 4.3-7 of BVPS-1 and BVPS-2 TS 3.3.3.8 (Accident Monitoring Instrumentation) such that only one valve position indication system for the power-operated relief valves and safety valves would be required to be operable. Minor editorial changes would also be made to BVPS-1 TS 3.3.3.8 and its associated Action Statements. The proposed changes to TS 3.3.3.8 and to TS Tables 3.3-11 and 4.3-7 would make them consistent with the NRC's Improved Standard Technical Specifications (NUREG-1431, Revision 1) and with the guidance of Regulatory Guide (R.G.) 1.97, "Instrumentation For Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations," and NUREG-0737, "Clarification of TMI Action Plan Requirements."

2.0 BACKGROUND

The licensee proposes to delete the TS requirements for the operability of the BVPS-1 power-operated relief valves (PORVs) acoustic detectors, and BVPS-1 and BVPS-2 TS requirements for the operability of the safety valves (SVs) tailpipe temperature detectors.

BVPS-1 TS 3.3.3.8 presently requires that both redundant position indication systems for the PORVs and the SVs be operable or BVPS-1 shall be placed in at least HOT SHUTDOWN per the applicable Action Statement. BVPS-2 TS 3.3.3.8 presently requires that the primary and backup SV position indication systems

be operable or BVPS-2 shall be placed in at least HOT SHUTDOWN per the applicable Action Statement. Inoperability of any one of these systems is not a safety concern since they perform no safety function, and because the other position indication system is available to meet TS operability requirements.

The proposed amendment modifies TS 3.3.3.8 Action Statements and Surveillance Requirements, but does not affect the associated Bases. The proposed changes are limited to PORV and SV position indication and do not involve any physical changes to the PORVs, SVs, or their setpoints. The licensee stated that the proposed amendment is consistent with the NRC's Improved Standard Technical Specifications (NUREG-1431, Revision 1) and meets the guidance of R.G. 1.97, NUREG-0578, and NUREG 0737.

3.0 EVALUATION

The primary purpose of the accident monitoring instrumentation is to display plant variables that provide information to the control room operators regarding plant status during accident situations. The instruments that monitor these variables are identified by the licensee in accordance with guidance contained in R.G. 1.97.

R.G. 1.97 defines five types of variables (Types A, B, C, D, and E) to be monitored by the control room operator during the course of an accident and during the long term stable shutdown phase following the accident. The R.G. also defines three categories (Category I, II, and III) of qualification criteria for this instrumentation, depending on the importance to safety of the measurement of a specific variable. Type A variables provide the primary information required for the control room operator to take specific manual action for which no automatic control is provided, and require Category I qualification which includes redundancy of the monitoring instrumentation.

The proposed amendment addresses instrumentation which is neither Type A nor Category I because there is no required manual safety control action associated with the PORV and SV position indication system for which no automatic action is provided. Therefore, the NRC staff agrees that redundant PORV and SV position indication is not required since it is not relied on for the performance of any safety functions.

TS 3.3.3.8 and TS Table 3.3-11 will continue to require operability of appropriate position indication for the PORVs and SVs as recommended by R.G. 1.97, NUREG-0578, and NUREG-0737. Position indication for the PORVs on both units is provided by qualified limit switches on the valves. For the SVs, in BVPS-1, position indication is provided by acoustic detectors and in BVPS-2 by a reed switch indicating device. Therefore, the proposed changes meet the recommendations of R.G. 1.97, the criteria contained in NUREG-0578 and NUREG-0737, and are consistent with the guidance provided in NUREG-1431, Revision 1.

4.0 SUMMARY

Based on review of the proposed amendment, the NRC staff concludes that the deletion of the redundant PORV and SV position indication from TS Tables 3.3-11 and 4.3-7 is consistent with guidance provided by the NRC staff for non-Category I post-accident monitoring instrumentation in R.G. 1.97 since this position indication provides no safety function. The proposed TS change is also consistent with the guidance provided in NUREG-1431, Revision 1, and therefore, is acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 3499). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Mario C. Gareri

Date: May 1, 1996