

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

May 28, 1998

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. MA1245
AND MA1246)

Dear Mr. Cross:

The Commission has issued the enclosed Amendment No. 215 to Facility Operating License No. DPR-66 and Amendment No. 92 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2). These amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 16, 1998, as supplemented May 14, 1998, which submitted License Amendment Request Nos. 254 and 123.

These amendments revise TS Table 4.3-1 to add footnote 6 to the channel calibration requirement for all instrument channels that are provided with an input from neutron flux detectors. Footnote 6 provides that neutron detectors may be excluded from channel calibrations. In addition, BVPS-1 TS Table 4.3-1 is being revised to add channel calibration requirements to items 2.b. (Power Range, Neutron Flux, Low Setpoint), 5. (Intermediate Range, Neutron Flux), 6. (Source Range, Neutron Flux (Below P-10)), and 23. (Reactor Trip System Interlocks P-6, P-8, P-9, and P-10). Furthermore, changes are being made to correct page numbers in the BVPS-2 TS Index and to add corresponding changes to the TS Bases for both units.

Your application for these amendments was technically complete and addressed the relevant issues. The application's no significant hazards consideration was suitable for use without changes and the evaluation of environmental considerations was proper.

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9806030115 980528
PDR ADOCK 05000334
P PDR

Docket Nos. 50-334 and 50-412

- Enclosures: 1. Amendment No. 215 to DPR-66
2. Amendment No. 92 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 28, 1998

Mr. J. E. Cross
President-Generation Group
Duquesne Light Company
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 (TAC NOS. MA1245
AND MA1246)

Dear Mr. Cross:

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Your application for these amendments was technically complete and addressed the relevant issues. The application's no significant hazards consideration was suitable for use without changes and the evaluation of environmental considerations was proper.

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Sincerely,

A handwritten signature in cursive script, reading "Donald S. Brinkman", is positioned above the typed name.

Donald S. Brinkman, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures: 1. Amendment No. 215 to DPR-66
2. Amendment No. 92 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

**J. E. Cross
Duquesne Light Company**

cc:

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Beaver Valley Power Station, Units 1 & 2

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Beaver Valley Power Station
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**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 215
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated March 16, 1998, as supplemented May 14, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 215, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 28, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 215

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 3-11

3/4 3-12a

-

Insert

3/4 3-11

3/4 3-12a

B 3/4 3-1c

TABLE 4.3-1

DPR-66

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Functional Unit</u>	<u>Channel Check</u>	<u>Channel Calibration</u>	<u>Channel Functional Test</u>	<u>Modes in Which Surveillance Required</u>
1. Manual Reactor Trip	N.A.	N.A.	S/U ⁽¹⁾ , R ⁽¹⁰⁾	N.A.
2. Power Range, Neutron Flux				
a. High Setpoint	S	D ⁽²⁾ , M ⁽³⁾ and Q ⁽⁶⁾	Q	1, 2
b. Low Setpoint	S	R ⁽⁶⁾	S/U ⁽¹⁾	2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R ⁽⁶⁾	Q	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R ⁽⁶⁾	Q	1, 2
5. Intermediate Range, Neutron Flux	S	R ⁽⁶⁾	S/U ⁽¹⁾	1, 2, 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾
6. Source Range, Neutron Flux (Below P-10)	N.A.	R ⁽⁶⁾	S/U ⁽¹⁾ , Q ⁽⁸⁾	2, 3, 4 and 5
7. Overtemperature ΔT	S	R ⁽⁶⁾	Q	1, 2
8. Overpower ΔT	S	R	Q	1, 2
9. Pressurizer Pressure-Low	S	R	Q	1, 2
10. Pressurizer Pressure-High	S	R	Q	1, 2
11. Pressurizer Water Level-High	S	R	Q	1, 2

TABLE 4.3-1 (Continued)

DPR-66

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Functional Unit</u>	<u>Channel Check</u>	<u>Channel Calibration</u>	<u>Channel Functional Test</u>	<u>Modes in Which Surveillance Required</u>
22. Automatic Trip Logic	N.A.	N.A.	M ⁽⁵⁾	1 ⁽¹⁴⁾ , 2 ⁽¹⁴⁾ , 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾
23. Reactor Trip System Interlocks				
A. P-6	N.A.	R ⁽⁶⁾	R	1, 2
B. P-8	N.A.	R ⁽⁶⁾	R	1
C. P-9	N.A.	R ⁽⁶⁾	R	1
D. P-10	N.A.	R ⁽⁶⁾	R	1
E. P-13	N.A.	R	R	1
24. Reactor Trip Bypass Breakers	N.A.	N.A.	M ⁽¹²⁾ , R ⁽¹³⁾ , S/U ⁽¹⁾	1 ⁽¹⁴⁾ , 2 ⁽¹⁴⁾ , 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾

DPR-66
INSTRUMENTATION

BASES

3/4.3.1 and 3/4.3.2 PROTECTIVE AND ENGINEERED SAFETY FEATURES (ESF)
INSTRUMENTATION (Continued)

The instrumentation functions that receive input from neutron detectors are modified by a note stating that neutron detectors are excluded from the CHANNEL CALIBRATION. The CHANNEL CALIBRATION for the power range neutron detectors consists of a normalization of the detectors based on a power calorimetric and flux map performed above 15% RATED THERMAL POWER. The power range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 or 1 on unit startup because the unit must be in at least MODE 1 to perform the test. The neutron detector CHANNEL CALIBRATION for the source range and intermediate range detectors consists of obtaining detector characteristics and performing an engineering evaluation of those characteristics. The intermediate range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 on unit startup because the unit must be in at least MODE 2 to perform the test. The source range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 or 3 on unit shutdown because the unit must be in at least MODE 3 to perform the test. The P-6 permissive neutron detector CHANNEL CALIBRATION is performed in conjunction with the intermediate range neutron detectors. The overtemperature ΔT , P-8, P-9 and P-10 permissive neutron detector CHANNEL CALIBRATIONS are performed in conjunction with the power range neutron detectors.



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duquesne Light Company, et al. (the licensee) dated March 16, 1998, as supplemented May 14, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 92, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. DLCO shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance, to be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Capra, Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: May 28, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A, Technical Specifications, with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
x	x
3/4 3-10	3/4 3-10
3/4 3-12	3/4 3-12
B 3/4 3-4	B 3/4 3-4
B 3/4 3-5	B 3/4 3-5
B 3/4 3-6	B 3/4 3-6
-	B 3/4 3-7

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TABLE 4.3-1

NPF-73

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Functional Unit</u>	<u>Channel Check</u>	<u>Channel Calibration</u>	<u>Channel Functional Test</u>	<u>Modes in Which Surveillance Required</u>
1. Manual Reactor Trip	N.A.	N.A.	S/U ⁽¹⁾ , R ⁽¹⁰⁾	1, 2, 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾
2. Power Range, Neutron Flux				
a. High Setpoint	S	D ⁽²⁾ , M ⁽³⁾ and Q ⁽⁶⁾	Q	1, 2
b. Low Setpoint	S	R ⁽⁶⁾	S/U ⁽¹⁾	1 ⁽⁷⁾ , 2
3. Power Range, Neutron Flux, High Positive Rate	N.A.	R ⁽⁶⁾	Q	1, 2
4. Power Range, Neutron Flux, High Negative Rate	N.A.	R ⁽⁶⁾	Q	1, 2
5. Intermediate Range, Neutron Flux	S	R ⁽⁶⁾	S/U ⁽¹⁾	1, 2, 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾
6. Source Range, Neutron Flux (Below P-10)	S	R ⁽⁶⁾	S/U ⁽¹⁾ , Q ⁽⁸⁾	2, 3, 4 and 5
7. Overtemperature ΔT	S	R ⁽⁶⁾	Q	1, 2
8. Overpower ΔT	S	R	Q	1, 2
9. Pressurizer Pressure-Low (Above P-7)	S	R	Q	1, 2
10. Pressurizer Pressure-High	S	R	Q	1, 2
11. Pressurizer Water Level-High (Above P-7)	S	R	Q	1, 2

TABLE 4.3-1 (Continued)

NPF-73

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Functional Unit</u>	<u>Channel Check</u>	<u>Channel Calibration</u>	<u>Channel Functional Test</u>	<u>Modes in Which Surveillance Required</u>
22. Automatic Trip Logic	N.A.	N.A.	M ⁽⁵⁾	1 ⁽¹⁴⁾ , 2 ⁽¹⁴⁾ , 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾
23. Reactor Trip System Interlocks				
A. Intermediate Range Neutron Flux, P-6	N.A.	R ⁽⁶⁾	R	1, 2
B. Power Range Neutron Flux, P-8	N.A.	R ⁽⁶⁾	R	1
C. Power Range Neutron Flux, P-9	N.A.	R ⁽⁶⁾	R	1
D. Power Range Neutron Flux, P-10	N.A.	R ⁽⁶⁾	R	1, 2
E. Turbine Impulse Chamber Pressure, P-13	N.A.	R	R	1
24. Reactor Trip Bypass Breakers	N.A.	N.A.	M ⁽¹²⁾ , R ⁽¹³⁾	1 ⁽¹⁴⁾ , 2 ⁽¹⁴⁾ , 3 ⁽¹⁴⁾ , 4 ⁽¹⁴⁾ , 5 ⁽¹⁴⁾

BASES3/4.3.1 and 3/4.3.2 REACTOR TRIP SYSTEM AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION (Continued)

Table 3.3-1 Action 2 has been modified by two notes. Note (4) allows placing the inoperable channel in the bypass condition for up to 4 hours while performing: a) routine surveillance testing of other channels, and b) setpoint adjustments of other channels when required to reduce the setpoint in accordance with other technical specifications. The 4 hour time limit is justified in accordance with WCAP-10271-P-A, Supplement 2, Revision 1, June 1990. Note (5) only requires SR 4.2.4 to be performed if a Power Range High Neutron Flux channel input to QPTR becomes inoperable. Failure of a component in the Power Range High Neutron Flux channel which renders the High Neutron Flux trip function inoperable may not affect the capability to monitor QPTR. As such, determining QPTR using the movable incore detectors once per 12 hours may not be necessary.

The following discussion pertains to Table 3.3-3, Functional Units 6.b and 6.c and the associated ACTION 34. The degraded voltage protection instrumentation system will automatically initiate the separation of the offsite power sources from the emergency buses. This action results in an automatic diesel generator start signal being generated as a direct result of the supply breakers opening between the normal and emergency buses. The failure of the degraded voltage protection system results in a loss of one of the automatic start signals for the diesel generator. Therefore, the ACTION statement requires the affected diesel generator to be declared inoperable if the required actions cannot be met within the specified time period.

The instrumentation functions that receive input from neutron detectors are modified by a note stating that neutron detectors are excluded from the CHANNEL CALIBRATION. The CHANNEL CALIBRATION for the power range neutron detectors consists of a normalization of the detectors based on a power calorimetric and flux map performed above 15% RATED THERMAL POWER. The power range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 or 1 on unit startup because the unit must be in at least MODE 1 to perform the test. The neutron detector CHANNEL CALIBRATION for the source range and intermediate range detectors consists of obtaining detector characteristics and performing an engineering evaluation of those characteristics. The intermediate range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 on unit startup because the unit must be in at least MODE 2 to perform the test. The source range neutron detector CHANNEL CALIBRATION is performed every 18 months but is not required for entry into MODE 2 or 3 on unit

3/4.3 INSTRUMENTATIONBASES3/4.3.1 and 3/4.3.2 REACTOR TRIP SYSTEM AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION (Continued)

shutdown because the unit must be in at least MODE 3 to perform the test. The P-6 permissive neutron detector CHANNEL CALIBRATION is performed in conjunction with the intermediate range neutron detectors. The overtemperature ΔT , P-8, P-9 and P-10 permissive neutron detector CHANNEL CALIBRATIONS are performed in conjunction with the power range neutron detectors.

3/4.3.3 MONITORING INSTRUMENTATION3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that: 1) the radiation levels are continually measured in the areas served by the individual channels; 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded; and 3) sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of NUREG-0737, "Clarification of TMI Action Plan Requirements," October, 1980.

3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the reactor core. The OPERABILITY of this system is demonstrated by irradiating each detector used and determining the acceptability of its voltage curve.

For the purpose of measuring $F_Q(Z)$ or $F_{\Delta H}^N$, a full incore flux map is used. Quarter-core flux maps, as defined in WCAP-8648, June 1976, may be used in re-calibration of the excore neutron flux detection system, and full incore flux maps or symmetric incore thimbles may be used for monitoring the Quadrant Power Tilt Ratio when one Power Range Channel is inoperable.

3/4.3.3.3 SEISMIC INSTRUMENTATION

The OPERABILITY of the seismic instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the facility and is consistent with the recommendations of Regulatory Guide 1.12, "Instrumentation for Earthquakes."

3/4.3 INSTRUMENTATIONBASES

3/4.3.3.4 METEOROLOGICAL INSTRUMENTATION

The OPERABILITY of the meteorological instrumentation ensures that sufficient meteorological data is available for estimating potential radiation doses to the public as a result of routine or accidental release of radioactive materials to the atmosphere. This capability is required to evaluate the need for initiating protective measures to protect the health and safety of the public and is consistent with the recommendations of Regulatory Guide 1.23, "Onsite Meteorological Programs."

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.6 (This Specification number is not used).

3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection systems ensures that sufficient capability is available to promptly detect and initiate protective action in the event of an accidental chlorine release. This capability is required to protect control room personnel and is consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release," January 1977.

3/4.3.3.8 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Plants to Assess Plant Conditions During and Following an Accident," December 1975 and NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."

3/4.3 INSTRUMENTATIONBASES

3/4.3.3.11 EXPLOSIVE GAS MONITORING INSTRUMENTATION

This instrumentation includes provisions for monitoring (and controlling) the concentrations of potentially explosive gas mixtures in the waste gas holdup system. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63 and 64 of Appendix A to 10 CFR Part 50.

3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment or structures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 215 AND 92 TO FACILITY OPERATING

LICENSE NOS. DPR-66 AND NPF-73

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letter dated March 16, 1998, as supplemented May 14, 1998, the Duquesne Light Company (the licensee) submitted a request for changes to the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2), Technical Specifications (TSs). The requested changes would revise TS Table 4.3-1 to add footnote 6 to the channel calibration requirement for all instrument channels that are provided with an input from neutron flux detectors. Footnote 6 provides that neutron detectors may be excluded from channel calibrations. In addition, BVPS-1 TS Table 4.3-1 would be revised to add channel calibration requirements to items 2.b. (Power Range, Neutron Flux, Low Setpoint), 5. (Intermediate Range, Neutron Flux), 6. (Source Range, Neutron Flux (Below P-10)), and 23. (Reactor Trip System Interlocks P-6, P-8, P-9, and P-10). Furthermore, changes would be made to correct page numbers in the BVPS-2 TS Index and to add corresponding changes to the TS Bases for both units. The May 14, 1998, letter provided minor editorial changes to the TS pages that did not change the initial proposed no significant hazards consideration determination or expand the amendment request beyond the scope of the April 22, 1998 Federal Register notice.

2.0 EVALUATION

TS Table 4.3-1 specifies the periodic surveillance test requirements for the reactor trip system instrumentation, including the channel calibration requirements for instrument channels provided with inputs from neutron flux detectors. The TS definition of channel calibration includes provisions that the channel output shall respond with the necessary range and accuracy to known values of the parameter which the channel monitors and that a channel calibration shall encompass the entire channel including the sensor and alarm and/or trip functions. Thus, to perform a channel calibration of a neutron flux channel, as required by this definition, would require including the neutron flux detector in the calibration. Inclusion of neutron flux detectors in

the channel calibration process is impractical in power reactor applications since to do so would require subjecting the detectors to known neutron fluxes. Therefore, the NRC staff has historically excluded neutron detectors from channel calibration requirements. This exclusion is currently applicable to TS Table 4.3-1, item 2.a. (Power Range, Neutron Flux, High Setpoint).

The proposed TS change would not change this provision but would add the exclusion to all other channels which receive inputs from neutron flux detectors. This proposed change is consistent with historical and current NRC staff requirements, as reflected in the NRC's Improved Standard Technical Specifications for Westinghouse Reactors (NUREG-1431, Revision 1) and is, therefore, acceptable.

Currently the channel calibration surveillance requirements for items 2.b., 5., 6., and 23. of BVPS-1 TS Table 4.3-1 are designated as "N.A." (not applicable). The proposed change for these surveillance requirements to a required frequency of at least once per 18 months with the provision that the neutron flux detectors may be excluded from channel calibration. Adding these periodic channel calibration requirements to the noted BVPS-1 channels is appropriate to ensure adequate surveillance testing of these channels and is acceptable since these changes would make the BVPS-1 surveillance requirements for the affected channels consistent with the BVPS-2 surveillance requirements for similar channels.

The proposed changes to correct page numbers in the BVPS-2 TS Index pages is purely administrative and is, therefore, acceptable.

The NRC staff has no objection to the corresponding changes to the TS Bases for both units.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (63 FR 19969). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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