

Docket No. 50-289

October 1, 1984

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Dear Mr. Hukill:

SUBJECT: AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE NO. DPR-50

The Commission has issued the enclosed Amendment No. 100 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). This amendment consists of changes to the Technical Specifications (TSs) in response to your TS change request No. 114 dated January 5, 1983, and Revision 1 dated June 13, 1984.

This amendment adds TSs for post-accident instrumentation that monitors high range noble gas effluents and containment parameters (high range radiation, pressure, water level and hydrogen). The TSs specify the required and minimum number of instrument channels, ACTION statements to be followed if channels are not available, and surveillance requirements. Some editorial changes are also included which were concurred in by Mr. R. Szech of your staff.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next Monthly Federal Register Notice.

Sincerely,

/S/

John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing

Enclosures:

1. Amendment No. 100
2. Safety Evaluation

cc w/enclosures:
See next page

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WASHINGTON, D. C. 20555

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by GPU Nuclear Corporation, et al (the licensees) dated January 5, 1983, as revised June 13, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Facility Operating License No. DPR-50 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.100 , are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 45 days after its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 1, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 100

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

vi
3-40a
3-40b

4-1
4-5a

Insert

vi
3-40a
3-40b
3-40d
3-40e
4-1
4-5a
4-10a

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3.5.5 ACCIDENT MONITORING INSTRUMENTATION

Applicability.

Applies to the operability requirements for the instruments identified in Table 3.5-2 and Table 3.5-3 during STARTUP, POWER OPERATION and HOT STANDBY.

Objectives

To assure operability of key instrumentation useful in diagnosing situations which could represent or lead to inadequate core cooling or evaluate and predict the course of accidents beyond the design basis.

Specification

- 3.5.5.1 The minimum number of channels identified for the instruments in Table 3.5-2, shall be OPERABLE. With the number of instrumentation channels less than the minimum required, restore the inoperable channel(s) to OPERABLE status within seven (7) days (48 hours for pressurizer level) or be in at least HOT SHUTDOWN within the next six (6) hours and in COLD SHUTDOWN within an additional 30 hours. Prior to startup following a COLD SHUTDOWN, the minimum number of channels shown in Table 3.5-2 shall be operable.
- 3.5.5.2 The channels identified for the instruments specified in Table 3.5-3 shall be OPERABLE. With the number of instrumentation channels less than required, restore the inoperable channel(s) to OPERABLE in accordance with the action specified in Table 3.5-3.

Bases

The Saturation Margin Monitor provides a quick and reliable means for determination of saturation temperature margins. Hand calculation of saturation pressure and saturation temperature margins can be easily and quickly performed as an alternate indication for the Saturation Margin Monitors.

Discharge flow from the two (2) pressurizer code safety valves and the PORV is measured by differential pressure transmitters connected across elbow taps downstream of each valve. A delta-pressure indication from each pressure transmitter is available in the control room to indicate code safety or relief valve line flow. An alarm is also provided in the control room to indicate that discharge from a pressurizer code safety or relief valve is occurring. In addition, an acoustic monitor is provided to detect flow in the PORV discharge line. An alarm is provided in the control room for the acoustic monitor.

The Emergency Feedwater System is provided with two channels of flow instrumentation on each of the two discharge lines. Local flow indication is also available for the emergency feedwater system.

Although the pressurizer has multiple level indications, the separate indications are selectable via a switch for display on a single display. Pressurizer level, however, can also be determined via the patch panel and the computer log. In addition, a second channel of pressurizer level indication is available independent of the NNI.

Although the instruments identified in Table 3.5-2 are significant in diagnosing situations which could lead to inadequate core cooling, loss of any one of the instruments in Table 3.5-2 would not prevent continued, safe, reactor operation. Therefore, operation is justified for up to 7 days (48 hours for pressurizer level). Alternate indications are available for Saturation Margin Monitors using hand calculations, the PORV/Safety Valve position monitors using discharge line thermocouple and Reactor Coolant Drain Tank indications, and for EFW flow using Steam Generator level and EFW pump discharge pressure. Pressurizer level has two channels, one channel from NNI (3 D/P instrument strings through a single indicator) and one channel independent of the NNI. Operation with the above pressurizer level channels out of service is permitted for up to 48 hours. Alternate indication would be available through the plant computer.

Monitors for containment pressure, containment water level, containment hydrogen level and various high range radiation monitors are useful to evaluate and predict the course of accidents which go beyond the plant design basis (See Table 3.5-3). These instruments should be maintained for that purpose.

TABLE 3.5-3

Post Accident Monitoring Instrumentation

<u>FUNCTION</u>	<u>INSTRUMENTS</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM NUMBER OF CHANNELS</u>	<u>ACTION</u>
1.	High Range Noble Gas Effluent			
	a. Condenser Vacuum Pump Exhaust (RM-A5-H1)	1	1	A
	b. Condenser Vacuum Pump Exhaust (RM-G25)	1	1	A
	c. Auxiliary and Fuel Handling Building Exhaust (RM-A8-H1)	1	1	A
	d. Reactor Building Purge Exhaust (RM-A9-H1)	1	1	A
	e. Reactor Building Purge Exhaust (RM-G24)	1	1	A
	f. Main Steam Lines Radiation (RM-G26/RM-G27)	1 each OTSG	1 each OTSG	A
2.	Containment High Range Radiation (RM-G22/23)	2	2	A
3.	Containment Pressure	2	1	B
4.	Containment Water Level	2	1	D
5.	Containment Hydrogen	2	1	D

TABLE 3.5-3 (Continued)

ACTIONS

- A. With the number of OPERABLE channels less than required by the Minimum Channels OPERABLE requirements:
1. either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or
 2. prepare and submit a Special Report within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- B. 1. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Channels OPERABLE requirements, restore the inoperable channel(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
2. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.

4. SURVEILLANCE STANDARDS

Specified intervals may be adjusted plus or minus 25 percent to accommodate normal test schedules.

4.1 OPERATIONAL SAFETY REVIEW

Applicability

Applies to items directly related to safety limits and limiting conditions for operation.

Objective

To specify the minimum frequency and type of surveillance to be applied to unit equipment and conditions.

Specification

4.1.1 The minimum frequency and type of surveillance required for reactor protection system and engineered safety feature protection system instrumentation when the reactor is critical shall be as stated in Table 4.1-1.

4.1.2 Equipment and sampling test shall be performed as detailed in Tables 4.1-2 and 4.1-3.

4.1.3 Each post accident monitoring instrumentation channel shall be demonstrated OPERABLE by the performance of the check, test and calibration at the frequencies shown in Table 4.1-4.

Bases

Check

Failures such as blown instrument fuses, defective indicators, or faulted amplifiers which result in "upscale" or "downscale" indication can be easily recognized by simple observation of the functioning of an instrument or system. Furthermore, such failures are, in many cases, revealed by alarm or annunciator action. Comparison of output and/or state of independent channels measuring the same variable supplements this type of built-in surveillance. Based on experience in operation of both conventional and nuclear systems, when the unit is in operation, the minimum checking frequency stated is deemed adequate for reactor system instrumentation.

Calibration

Calibration shall be performed to assure the presentation and acquisition of accurate information. The nuclear flux (power range) channels amplifiers shall be checked and calibrated if necessary, every shift against a heat balance standard. The frequency of heat balance checks will assure that the difference between the out-of-core instrumentation and the heat balance remains less than 4%.

TABLE 4.1-1 (Continued)

	<u>CHANNEL DESCRIPTION</u>	<u>CHECK</u>	<u>TEST</u>	<u>CALIBRATE</u>	<u>REMARKS</u>
28.	Radiation Monitoring Systems*	W(1)(3)	M(3)	Q(2)	(1) Using the installed check source when background is less than twice the expected increase in cpm which would result from the check source alone. Background readings greater than this value are sufficient in themselves to show that the monitor is functioning. (2) Except area gamma radiation monitors RM-G6, RM-G7, and RM-G8, which are located in high radiation areas of the Reactor Building. These monitors will be calibrated quarterly or at the next scheduled reactor shutdown following the quarter in which calibration would normally be due, if a shutdown during the quarter does not occur. (3) Surveillances are required to be performed only when containment integrity is required. This applies to monitors which initiate containment isolation only.
29.	High and Low Pressure Injection Systems: Flow Channels.	NA	NA	R	

* Does not include the monitors covered under specification 3.5.5.2 and 4.1.3

TABLE 4.1-4
POST ACCIDENT MONITORING INSTRUMENTATION

<u>FUNCTION</u>	<u>INSTRUMENTS</u>	<u>CHECK</u>	<u>TEST</u>	<u>CALIBRATE</u>	<u>REMARKS</u>
1.	Noble Gas Effluent				
	a. Condenser Vacuum Pump Exhaust (RM-A5-H1)	W	M	R	(1) Using the installed check source when background is less than twice the expected increase in cpm which would result from the check source alone. Background readings greater than this value are sufficient in themselves to show that this monitor is functioning.
	b. Condenser Vacuum Pump Exhaust (RM-G25)	W(1)	M	R	
	c. Aux. & Fuel Handling Building Exhaust (RM-A8-H1)	W	M	R	
	d. Reactor Building Purge Exhaust (RM-A9-H1)	W	M	R	
	e. Reactor Building Purge Exhaust (RM-G24)	W(1)	M	R	
	f. Main Steam Lines Radiation (RM-G26/ RM-G27)	W(1)	M	R	
2.	Containment High Range Radiation (RM-G 22/23)	W	M	R	
3.	Containment Pressure	W	N/A	R	
4.	Containment Water Level	W	N/A	R	
5.	Containment Hydrogen	W	M	R	

4-10a



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-289

INTRODUCTION AND BACKGROUND

In November 1980, the NRC staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letters dated January 5, 1983 and June 13, 1984, GPU Nuclear Corporation (the licensee) responded to Generic Letter 83-37 by submitting Technical Specification change requests for Three Mile Island Nuclear Station, Unit No. 1 (TMI-1). This evaluation covers the following TMI Action Plan items:

1. Noble Gas Effluent Monitors (II.F.1.1)
2. Containment High-Range Radiation Monitor (II.F.1.3)
3. Containment Pressure Monitor (II.F.1.4)
4. Containment Water Level Monitor (II.F.1.5)
5. Containment Hydrogen Monitor (II.F.1.6)

EVALUATION

1. Noble Gas Effluent Monitors (II.F.1.1)

The licensee has supplemented the existing normal range monitors to provide noble gas monitoring in accordance with Item II.F.1.1. Proposed TSs were submitted that are consistent with the guidelines provided in our Generic Letter 83-37. We conclude that the proposed TSs for Item II.F.1.1 are acceptable as they meet the intent of the guidelines contained in Generic Letter 83-37.

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2. Containment High-Range Radiation Monitor (II.F.1.3)

The licensee has installed two in-containment monitors in TMI-1, which are consistent with the guidance of TMI Action Plan Item II.F.1.3. Generic Letter 83-37 provided guidance for limiting conditions for operation and surveillance requirements for these monitors. The licensee proposed TSs that are consistent with the guidance provided in our Generic Letter 83-37. We conclude that the proposed TSs for Item II.F.1.3 are acceptable.

3. Containment Pressure Monitor (II.F.1.4)

TMI-1 has been provided with two supplementary channels for monitoring containment pressure following an accident. In addition to these two channels, a control grade channel is available as a backup monitor which covers the range from 0 to 100 psi. The licensee has proposed TSs that are consistent with the guidelines contained in Generic Letter 83-37. We conclude that the proposed TSs for the containment pressure monitor are acceptable.

4. Containment Water Level Monitor (II.F.1.5)

The containment water level monitor provides the capability required by TMI Action Plan Item II.F.1.5. In addition to the two channels required by Item II.F.1.5, a control grade single channel monitor is available as a backup monitor. This monitor covers the range from 0 to 10 feet. The proposed TSs for TMI-1 contain limiting conditions for operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-37. We conclude that the proposed TSs for the containment water level monitor are acceptable.

5. Containment Hydrogen Monitor (II.F.1.6)

The licensee installed containment hydrogen monitors that provide the capability required by TMI Action Plan Item II.F.1.6. The proposed TSs contain appropriate limiting conditions for operation and surveillance for these monitors. We conclude that the proposed TSs are acceptable as they are consistent with the guidance contained in Generic Letter 83-37.

6. The administrative change in Section 3.5.5.2 is a correction regarding local flow indication in the Emergency Feedwater System which deletes reference to the non-nuclear instrumentation (NNI) since that indication is independent of NNI. The other administrative changes are editorial. The staff finds all the administrative changes acceptable.

ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 1, 1984

Principal Contributor: - C. Patel