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50-272
311

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Document ID	Revision	Status	Quantity	Format	RecNo
PRC SECG-ATT.08 000	6	A	1	H	83776
PRC SECG-SECG-TOC 000	25	A	1	H	83735
PRC SECG-SECG-TOC-BASIS 000	7	A	1	H	83816
PRC SECG-SECT.09.1 (BASIS) 000	1	A	1	H	83856

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MC N04

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A001

SALEM GENERATING STATION
EVENT CLASSIFICATION GUIDE
October 10, 2000

PSE&G
CONTROL
COPY # SECG 0101

CHANGE PAGES FOR
REVISION #25

The Table of Contents forms a general guide to the current revision of each section and attachment of the Salem ECG. The changes that are made in this TOC Revision #25 are shown below.

1. Check that your revision packet is complete.
2. Add the revised documents.
3. Remove and recycle the outdated material listed below.

ADD			REMOVE		
<u>Pages</u>	<u>Description</u>	<u>Rev.</u>	<u>Pages</u>	<u>Description</u>	<u>Rev.</u>
ALL	TOC	25	ALL	TOC	24
All	Attachment 8	06	All	Attachment 8	05

REVISION SUMMARY:

1. Attachment 8 - Corrected typographical error.

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ECG
T.O.C.
Pg. 1 of 4

SALEM EVENT CLASSIFICATION GUIDE
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SECC0101

<u>SECTION</u>	<u>TITLE</u>	<u>REV #</u>	<u>PAGES</u>	<u>DATE</u>
T.O.C.	Table of Contents/Signature Page	25	4	10/10/00
i	Introduction and Usage	01	11	06/29/00
ii	Glossary of Acronyms & Abbreviations	00	6	01/21/97
iii	Critical Function Status Trees (CFSTs), Unit 1	21	7	04/16/98
	Critical Function Status Trees (CFSTs), Unit 2	23	6	08/19/97
1.0	Fuel Clad Challenge	00	1	01/21/97
2.0	RCS Challenge	00	1	01/21/97
3.0	Fission Product Barriers (Table)	00	1	01/21/97
4.0	EC Discretion	00	1	01/21/97
5.0	Failure to TRIP	00	1	01/21/97
6.0	Radiological Releases/Occurrences			
	6.1 Gaseous Effluent Release	00	4	01/21/97
	6.2 Liquid Effluent Release	00	1	01/21/97
	6.3 In Plant Radiation Occurrences	00	1	01/21/97
	6.4 Irradiated Fuel Event	00	2	01/21/97
7.0	Electrical Power			
	7.1 Loss of AC Power Capabilities	00	2	01/21/97
	7.2 Loss of DC Power Capabilities	00	1	01/21/97
8.0	System Malfunctions			
	8.1 Loss of Heat Removal Capability	00	2	01/21/97
	8.2 Loss of Overhead Annunciators	00	1	01/21/97
	8.3 Loss of Communications Capability	00	1	01/21/97
	8.4 Control Room Evacuation	00	1	01/21/97
	8.5 Technical Specifications	00	1	01/21/97
9.0	Hazards - Internal/External			
	9.1 Security Threats	00	1	01/21/97
	9.2 Fire	00	1	01/21/97
	9.3 Explosion	00	1	01/21/97
	9.4 Toxic/Flammable Gases	00	2	01/21/97
	9.5 Seismic Event	00	1	01/21/97
	9.6 High Winds	00	1	01/21/97
	9.7 Flooding	00	1	01/21/97
	9.8 Turbine Failure/Vehicle Crash/ Missile Impact	00	1	01/21/97
	9.9 River Level	00	1	01/21/97
10.0	Reserved for future use	N/A		
WC	Salem ECG Charts (Located In ERFs)	00	2	01/21/97
SGS				

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Licensing is responsible for the Reportable Action Level (Section 11) and associated Attachments (marked by "L")				
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11.0 Reportable Action Levels (RALs)

11.1	Technical Specifications	00	3	01/21/97
11.2	Design Basis/ Unanalyzed Condition	00	2	01/21/97
11.3	Engineered Safety Features (ESF)	01	1	08/28/97
11.4	Personnel Safety/Overexposure	00	2	01/21/97
11.5	Environmental	00	1	01/21/97
11.6	After-the-Fact	00	1	01/21/97
11.7	Security/Emergency Response	01	1	03/13/97
	Capabilities			
11.8	Public Interest	00	1	01/21/97
11.9	Accidental Criticality/ Special Nuclear Material/ Rad Material Shipments - Releases	01	2	08/15/00
11.10	Voluntary Notifications	00	1	01/21/97

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Licensing is responsible for the Reportable Action Level (Section 11)
and associated Attachments (marked by "L")

<u>ATTACHMENT</u>	<u>TITLE</u>	<u>REV #</u>	<u>PAGES</u>	<u>DATE</u>
1	UNUSUAL EVENT	03	2	02/29/00
2	ALERT	03	2	02/29/00
3	SITE AREA EMERGENCY	03	2	02/29/00
4	GENERAL EMERGENCY	04	5	02/29/00
5	L NRC Data Sheet Completion Reference	01	7	07/22/99
6	Primary Communicator Log	14	8	08/15/00
7	Primary Communicator Log (GE)	deleted		02/29/00
8	Secondary Communicator Log	06	9	10/10/00
9	L Non-Emergency Notifications Reference	13	3	08/15/00
10	L 1 Hr Report - NRC Regional Office	00	3	01/21/97
11	L 1 Hr Report (Common Site) Security/Safeguards	00	3	01/21/97
12	L 1 Hr Report - NRC Operations	00	3	01/21/97
13	L 4 Hr Report - Contaminated Events Outside Of The RCA	00	7	01/21/97
14	L 4 Hr Report - NRC Operations	01	3	05/01/98
15	L Environmental Protection Plan	01	3	03/13/97
16	L Spill / Discharge Reporting	01	7	03/29/00
17	L 4 Hr Report - Fatality or Medical Emergency	00	4	01/21/97
18	L 4 Hr Report - Radiological Transportation Accident	01	4	05/12/97
19	L 24 Hr Report - Fitness For Duty (FFD) Program Events	01	3	05/12/97
20	L 24 Hr Report - NRC Regional Office	00	3	01/21/97
21	L Reportable Event - LAC/ Memorandum Of Understanding (M.O.U.)	00	2	01/21/97
22	L T/S Required Engineering Evaluation	00	2	01/21/97
23	Reserved			
24	UNUSUAL EVENT (Common Site)	05	3	02/29/00
25	1 Hr Report (Common Site) - Major Loss of Emergency Assessment, Offsite Response, <u>OR</u> Communications Capability	01	3	07/22/99

SIGNATURE PAGE

Prepared By: William L. Decker, Rev 0 09/21/2000
(If Editorial Revisions Only, Last Approved Revision) Date

Section/Attachments Revised: N/A _____
(List Non Editorial Only - Section/Attachments) Date

Reviewed By: N/A _____
Station Qualified Reviewer Date

Reviewed By: N/A _____
Department Manager Date

Reviewed By NA _____
Manager - Licensing Date
(Reportable Action Level (Section 11) and associated Attachments marked by "L")

Reviewed By C. J. Schuman 10-2-00
Manager - EP & IT Date

Reviewed By N/A _____
Manager - Quality Assessment - NBU
(If Applicable) Date

SORC Review and Station Approvals

N/A
Mtg. No. Salem Chairman

N/A
Vice President Nuclear Operations

Date

Date

Effective Date of this Revision: 10/10/00
Date

ATTACHMENT 8

SECONDARY COMMUNICATOR LOG

Table of Contents

Pages

- 1 - 2 Notifications & Data Collection/Transmission
- 3 - 4 Incoming Calls (BNE, DEMA, OEM, AAAG, etc.)
- 5 Major Equipment & Electrical Status (MEES) form
- 6 Operational Status Board (OSB) form
- 7 - 8 Station Status Checklist (SSCL) form
- 9 Common Site UNUSUAL EVENT – Station Status Checklist form

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Emergency Classification: (circle)	UE	ALERT	SAE	GE
Name: _____ (Print)		Position: CM2 /TSC2/ EOF2 (circle)		

A. NOTIFICATIONS

NOTE

A new Attachment 8 is required to be implemented if the classification or protective action recommendation (PAR) changes.

- | | | <u>Initials</u> |
|-----|---|-----------------|
| 1. | OBTAIN a copy of Attachment 6 and ASSIST Primary Communicator with 15-minute notifications, as necessary. | |
| 2. | DIRECT the Shift Rad Pro Tech (SRPT) (x2644) to implement SC.EP-EP.ZZ-0301(Q) , Shift Radiation Protection Technician Response. (N/A for Common Site). | CM2/TSC2/EOF2 |
| | Name: _____ Time: _____ | CM2 |
| 3. | <u>For an ALERT or higher emergency;</u> | |
| () | a. CALLOUT an additional SRO and have him/her report to the OSC. | |
| | Name: _____ Time: _____ | CM2 |
| | b. ACTIVATE ERDS within 60 minutes from the Affected Unit's SPDS terminal; | |
| | 1) PRESS <UNIT MASTER MENU> key. | |
| | 2) PRESS <ERDS> key. | |
| | 3) FOLLOW screen prompts. | CM2 |

Initials

A. NOTIFICATIONS (cont'd)

4. OBTAIN a copy of the **ICMF** and FAX the ICMF to Group A. _____
CM2/TSC2/EOF2
5. COMPLETE a **Station Status Checklist (SSCL)** Form, Pg. 7 or Common Site
UNUSUAL EVENT Station Status Checklist (SSCL) Form, Pg. 9;
- () a. OBTAIN OS (TSS/SSM) assistance, as needed for SSCL Pg.1.
- () b. OBTAIN SRPT (RAC/RSM) assistance, as needed for SSCL Pg.2. (N/A
for Common Site)
- () c. FAX to Group B.
- () d. IF fax transmission of the SSCL is incomplete,
THEN CONTACT the State Agencies listed below, READ the data, AND
DOCUMENT on SSCL, Pg. 2.

DEMA	Delaware Emergency Management Agency	302-659-2290
BNE	NJ Bureau of Nuclear Engineering	609-984-7700

CM2/TSC2/EOF2

6. OBTAIN completed **NRC Data Sheet** from the CM-1, and FAX form to Group B. _____
CM2/TSC2/EOF2
7. REPEAT Step 5 approximately every half hour OR IMMEDIATELY for significant changes in Station status, until either Turnover or relief. _____
CM2/TSC2/EOF2
8. TURNOVER responsibility for offsite notifications and offsite data updates (SSCLs) to the oncoming facility (TSC or EOF);
- () a. GIVE names and phone numbers of contacts already made with any Offsite Agencies.
- () b. GIVE time for next SSCL. _____
CM2/TSC2
9. IF available for other duties AND TSC turnover is complete,
THEN obtain headset, MAN the Ops Data line and CONTACT the TSC ops advisor and establish an open line of communication from the control room to the TSC. _____
CM-2

B. DATA COLLECTION/TRANSMISSION

1. WHEN in an ALERT or higher emergency
OR AFTER significant changes in plant status;
THEN COMPLETE the **Major Equipment and Electrical Status (MEES)** Form.
- () a. OBTAIN Licensed Operator review.
- () b. GIVE a copy to the OSC Coordinator.
- () c. FAX to Group C. _____
CM2

Initials

B. DATA COLLECTION/TRANSMISSION (cont'd)

2. IF requested by the TSC,
THEN COMPLETE the **Operational Status Board (OSB)** Form every 15 minutes,
(TSS may modify the frequency or data list as appropriate)

- () a. OBTAIN Licensed Operator review.
() b. FAX to Group C.

_____ CM2

3. ENSURE the Facility OSB and MEES Status Boards are updated as follows:

- () a. OBTAIN OSB Data from **SPDS** "Unit Master Menu."
() b. IF SPDS is Out of Service,
THEN REQUEST CM2 to perform step B.2, above. (data set and frequency
of updates may be revised by the TSS based on event circumstances)
() c. WHEN significant changes in plant status occur,
THEN REQUEST CM2 to perform step B.1, above.

_____ TSC2/EOF2

4. WHEN the emergency is terminated,
THEN FORWARD this document and all completed Forms to the OS (TSS/SSM).

_____ CM2/TSC2/EOF2

C. INCOMING CALLS

STATE OFFICIALS

1. IF Notifications authority has transferred,
THEN DIRECT the caller to contact the TSC (or EOF if activated).

_____ CM2/TSC2

2. WHEN contacted by any State Agency Officials (listed here),

DEMA - Delaware Emergency Management Agency
AAAG - Delaware Accident Assessment Advisory Group
BNE - NJ Bureau of Nuclear Engineering
DEP - NJ Department of Environmental Protection
OEM - NJ Office of Emergency Management

PERFORM the following:

- () a. OBTAIN and RECORD;

Agency

Caller's Name

Phone #

- () b. READ the latest EC approved SSCL.

Initials

C. INCOMING CALLS (cont'd)

STATE OFFICIALS

- () c. IF caller is **NJ-BNE, DEMA, or AAAG**,
THEN also READ the approved NRC Data Sheet Event Description.

CM2/TSC2/EOF2

NEWS MEDIA

CAUTION

Communicators are NOT authorized to release any information to the News Media.

3. WHEN contacted by any News Media representative,
READ the appropriate message below:

- () a. IF the ENC is not activated (Unusual Event), say;

**“You are requested to contact the Nuclear Communications Office
at the following number: 856-339-1186.”**

- () b. IF the ENC is activated (ALERT or higher), say;

**“You are requested to contact the Media Information Operator at
any of the following numbers: 856-273-0188, -0282, -0479, or -
0586.”**

CM2/TSC2/EOF2

NRC OPERATIONS CENTER

4. WHEN directed by the NRC to TERMINATE ERDS transmission,
THEN GO TO any SPDS terminal of the affected Unit AND PROCEED as follows:

- a. PRESS <UNIT MASTER MENU> key.
- b. PRESS <ERDS> key.
- c. FOLLOW screen prompts.
- d. WHEN completed, NOTIFY the OS.

CM2

SALEM UNIT						DATE: _____		
MAJOR EQUIPMENT AND ELECTRICAL STATUS						UPDATE TIME: _____		
NOTE: Y = IN SERVICE N = OUT OF SERVICE (CIRCLE ANY UNAVAILABLE EQUIPMENT)			ECCS SYSTEMS		ELECT. FEED	Y/N		
			CHARGING PUMPS	1	B9D			
				2	C9D			
				3	A7X			
			SAFETY INJ PUMPS		1	A5D		
		2	C5D					
RHR		1	A7D					
PUMPS		2	B7D					
COOLING SYSTEMS			ELECT. FEED	Y/N				
AUX FD PUMPS	1	A1D						
	2	B1D						
	3	STM.						
SERVICE WATER PUMPS	1	3D						
	2	8D						
	3	B3D						
	4	B8D						
	5	3D						
	6	8D						
COMP. COOLING PUMPS	1	A10D						
	2	B10D						
	3	C10D						
REACTOR COOLANT PUMPS	1	H4D						
	2	E4D						
	3	F4D						
	4	G4D						
CONDENSATE PUMPS	1	H1D						
	2	E1D						
	3	F1D						
CIRC WATER PUMPS	1A	U1 / U2			COMMENTS:			
	1B	2AD/2AD						
	2A	7BD/7BD						
	2B	3AD/3AD						
	3A	6BD/6BD						
	3B	4AD/4AD						
		5BD/5BD						
ECCS SYSTEMS		ELECT. FEED		Y/N	CONTAINMENT CONTROL		ELECT. FEED	Y/N
CHARGING PUMPS		B9D			CONT. SPRAY PUMPS		A2D	
		C9D					C2D	
		A7X			CFCU		HI	LOW
SAFETY INJ PUMPS		A5D			1 A3X A4X		A2X	
		C5D			2 B3X B4X		B2X	
RHR		A7D			3 C3X C4X		C2X	
PUMPS		B7D			4 B7X B8X		B6X	
					5 C7X C8X		C6X	
ELECTRICAL STATUS			Y/N				ELECT. FEED	Y/N
OFFSITE AC POWER AVAILABLE								
EMERGENCY DIESELS			RUN	LOADED	IODINE REMOVAL		G7X	
EDG			A		2		E7X	
			B		H ² RECOM		A15X	
			C		2		B15X	
#3 GAS TURBINE					MISC. EQUIPMENT			
ELEC DISTRIBUTION AVAILABLE?			Y/N		FIRE PUMPS (DIESEL)			
VITAL BUS			A		1			
			B		2			
			C		STATION AIR COMP.			
			E		1		1H6D	
			F		2		2G1D	
			G		3		1G1D	
			H		EMERGENCY AIR COMP.		ELECT. FEED	Y/N
					1		1C14X	
					2		2C14X	

LICENSED OPERATOR REVIEW: _____
INITIALS

Operational Status Board – Salem

UPDATE:

TIME DATE

UNIT #

I. EMERGENCY CORE COOLING SYSTEM

Cent. Chrg. Pump Flow (BIT flow)	<table border="1" style="width: 80px; height: 20px;"></table>	GPM
SI P flow # <u> 1 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	GPM
SI P flow # <u> 2 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	
RHR P flow # <u> 1 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	GPM
RHR P flow # <u> 2 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	GPM
RWST LEVEL	<table border="1" style="width: 80px; height: 20px;"></table>	FT

II. CONTAINMENT

Cont. Pressure	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
Cont. Temperature (AVG)	<table border="1" style="width: 80px; height: 20px;"></table>	F
Cont. H ₂ Concen.	<table border="1" style="width: 80px; height: 20px;"></table>	%
Cont. Sump level	<table border="1" style="width: 80px; height: 20px;"></table>	%
Cont. Rad (hi range) <u> R44A </u>	<table border="1" style="width: 80px; height: 20px;"></table>	R/hr
Cont. Rad (hi range) <u> R44B </u>	<table border="1" style="width: 80px; height: 20px;"></table>	R/hr

III. REACTOR COOLANT SYSTEM

# of RCPs Running	<table border="1" style="width: 80px; height: 20px;"></table>	
RVLIS (full range)	<table border="1" style="width: 80px; height: 20px;"></table>	%
Core Exit Thermocouple (hottest)	<table border="1" style="width: 80px; height: 20px;"></table>	F
# of Thermocouples > 1200 °F	<table border="1" style="width: 80px; height: 20px;"></table>	
Tc Loop <u> 1 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Tc Loop <u> 2 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Tc Loop <u> 3 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Tc Loop <u> 4 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
*Tave (Autoneered) *If no RCPs running, Tave on	<table border="1" style="width: 80px; height: 20px;"></table>	F
PZR/RCS Pressure the Control Console is invalid.	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
PZR Level (hot)	<table border="1" style="width: 80px; height: 20px;"></table>	%
Th Loop <u> 1 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Th Loop <u> 2 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Th Loop <u> 3 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Th Loop <u> 4 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	F
Reactor Power/Neutron flux	<table border="1" style="width: 80px; height: 20px;"></table>	%/amps/CPS
Subcooling Margin	<table border="1" style="width: 80px; height: 20px;"></table>	F

IV. C.V.C.S

Letdown flow	<table border="1" style="width: 80px; height: 20px;"></table>	GPM
Charging flow	<table border="1" style="width: 80px; height: 20px;"></table>	GPM

V. SECONDARY COOLANT SYSTEM

NO. <u> 1 </u> SG level	<table border="1" style="width: 80px; height: 20px;"></table>	% (NR or WR)
NO. <u> 2 </u> SG level	<table border="1" style="width: 80px; height: 20px;"></table>	% (NR or WR)
NO. <u> 3 </u> SG level	<table border="1" style="width: 80px; height: 20px;"></table>	% (NR or WR)
NO. <u> 4 </u> SG level	<table border="1" style="width: 80px; height: 20px;"></table>	% (NR or WR)
NO. <u> 1 </u> SG pressure	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
NO. <u> 2 </u> SG pressure	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
NO. <u> 3 </u> SG pressure	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
NO. <u> 4 </u> SG pressure	<table border="1" style="width: 80px; height: 20px;"></table>	PSIG
NO. <u> 1 </u> SG feedflow	<table border="1" style="width: 80px; height: 20px;"></table>	% or LBS/HR
NO. <u> 2 </u> SG feedflow	<table border="1" style="width: 80px; height: 20px;"></table>	% or LBS/HR
NO. <u> 3 </u> SG feedflow	<table border="1" style="width: 80px; height: 20px;"></table>	% or LBS/HR
NO. <u> 4 </u> SG feedflow	<table border="1" style="width: 80px; height: 20px;"></table>	% or LBS/HR
AFST level	<table border="1" style="width: 80px; height: 20px;"></table>	%

VI. MISC. TANKS LEVEL

Waste Hold-Up Tank # <u> 1 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	%
Waste Hold-Up Tank # <u> 2 </u>	<table border="1" style="width: 80px; height: 20px;"></table>	%
Waste Monitor HUT	<table border="1" style="width: 80px; height: 20px;"></table>	%

VII. SSCL INFORMATION

Offsite power available?	<table border="1" style="width: 80px; height: 20px;"></table>	YES	<table border="1" style="width: 80px; height: 20px;"></table>	NO
Two or more diesels available?	<table border="1" style="width: 80px; height: 20px;"></table>		<table border="1" style="width: 80px; height: 20px;"></table>	
Did ECCS actuate?	<table border="1" style="width: 80px; height: 20px;"></table>		<table border="1" style="width: 80px; height: 20px;"></table>	
Is the containment barrier failed?	<table border="1" style="width: 80px; height: 20px;"></table>		<table border="1" style="width: 80px; height: 20px;"></table>	

SIGNIFICANT PLANT EVENTS

Licensed Operator Review

 Initials

STATION STATUS CHECKLIST

(Pg. 1 of 2)

Operational Information

SALEM GENERATING STATION Unit No. _____ Message Date _____ Time _____

Transmitted By: Name _____ Position _____
(CR/TSC/EOF)

1. Date and Time Event Declared: Date _____ Time _____ (24 hr clock)

2. Event Classification: ☐ Unusual Event ☐ Site Area Emergency
☐ Alert ☐ General Emergency

3. Cause of Event: Primary Initiating Condition used for declaration

EAL #(s) _____

Description of the event _____

4. Status of Reactor: ☐ Tripped/Time _____ ☐ At Power ☐ Startup
☐ Hot Standby ☐ Hot Shutdown ☐ Cold Shutdown ☐ Refuel

5. RZR/RCS Pressure _____ psig Core Exit TC _____ °F

6. Is offsite power available? ☐ YES ☐ NO7. Are two or more diesel generators available? ☐ YES ☐ NO8. Did any Emergency Core Cooling Systems actuate? ☐ YES ☐ NO9. Is the Containment barrier failed? (Loss per EAL section 3.3) ☐ YES ☐ NO

10. Other pertinent information _____

Approved: _____
EC or TSS or SSM

STATION STATUS CHECKLIST
(PAGE 2 OF 2)
RADIOLOGICAL INFORMATION

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SALEM GENERATING STATION UNIT NUMBER: _____ CALCULATION TIME: _____ DATE: _____

1. GASEOUS RELEASE>TECH SPEC (T/S) LIMITS:

(T/S LIMITS: 2.42 E+05 μ Ci/sec NG or 2.1E+01 μ Ci/sec IODINE)

YES: [] RELEASE START TIME: _____ DATE: _____

NO: []

A. RELEASE TERMINATED: YES [] NO [] N/A []

B. ANTICIPATED OR UNKNOWN DURATION OF RELEASE: _____ HOURS

C. TYPE OF RELEASE: GROUND [] ELEVATED: [] N/A []

D. ADJUSTED WIND SPEED: _____ (mph) _____ (m/sec) WIND DIR (deg from) _____

E. STABILITY CLASS: _____ (A-G) DELTA T: _____ (deg C)

F. VENT PATH OF RELEASE: R41 [] R45B/C [] R44 [] R46 []

G. NG RELEASE RATE: R41 _____ R45B/C _____ R44 _____

R46 _____ (μ Ci/sec)

H. I-131 RELEASE RATE: R41 _____ R45B/C _____ R44 _____

R46 _____ DEFAULT (μ Ci/sec) (circle if default)

I. TOTAL RELEASE RATE NOBLE GAS: _____ (μ Ci/sec)

J. TOTAL RELEASE RATE IODINE-131: _____ (μ Ci/sec)

2. PROJECTED OFFSITE DOSE RATE CALCULATIONS:

DISTANCE FROM VENT (IN MILES)	XU/Q (1/M2)	TEDE RATE (MREM/HR)	TEDE DOSE (4 DAY) (MREM)	THYROID- CDE RATE (MREM/HR)	THYROID- CDE DOSE (MREM)
MEA 0.79	_____	_____	_____	_____	_____
2.00	_____	_____	_____	_____	_____
LPZ 5.00	_____	_____	_____	_____	_____
EPZ 10.00	_____	_____	_____	_____	_____

3. OTHER PERTINENT INFORMATION: _____

4. UPDATE TO STATES (IF VERBALLY TRASMITTED):

NAME TIME INITIALS

STATE OF NEW JERSEY: _____

STATE OF DELAWARE: _____

AGENCY: _____

APPROVED: _____
EC or RAC or RSM

Common Site Unusual Event STATION STATUS CHECKLIST

Operational Information

Message Date _____ Time _____

Transmitted by: Name _____ Position _____

1. Date and Time Event Declared: Date _____ Time: _____

2. Cause of event: Primary Initiating Condition used for declaration

EAL# _____

Description of the event:

33FT. LEVEL WIND DIRECTION (From): _____ WIND SPEED _____
(From MET Computer) (DEGREES) (MPH)

3. Status of the Reactors	Mode: (Power, Startup, Hot Standby, Hot S/D, Cold S/D, Refuel)	Rx Pressure	Hottest Core Exit TC / Rx Temp	Rx Water Level
Salem 1		psig	°F	covered
Salem 2		psig	°F	covered
Hope Creek		psig	°F	in.

	Salem 1		Salem 2		Hope Creek	
	YES	NO	YES	NO	YES	NO
4. Is offsite power available?						
5. Are two or more diesel generators operable?						
6. Did any Emergency Core Cooling Systems actuate?						
7. Is any Containment Barrier failed? (Loss per EAL section 3.3)						
8. Radiological release (> Tech Spec Limit) in progress		X		X		X

9. Other pertinent information _____

EC Initials
(Approval to Transmit ICMF)

PSE&G
CONTROL
4
SECG0101

SALEM GENERATING STATION
EVENT CLASSIFICATION GUIDE TECHNICAL BASIS
October 10, 2000

CHANGE PAGES FOR
REVISION #07

The Table of Contents forms a general guide to the current revision of each section and attachment of the Salem ECG Technical Basis. The changes that are made in this TOC Revision #07 are shown below.

1. Check that your revision packet is complete.
2. Add the revised documents.
3. Remove and recycle the outdated material listed below.

ADD			REMOVE		
<u>Pages</u>	<u>Description</u>	<u>Rev.</u>	<u>Pages</u>	<u>Description</u>	<u>Rev.</u>
ALL	TOC	7	ALL	TOC	6
All	EAL Section 9.1	1	All	EAL Section 9.1	0

Summary of significant changes:

1. Added clarification to the Site Area Emergency unarmed intruder security event.
2. Changed SNSS to OS.
3. Deleted reference to PSE&G.

SALEM ECG TECHNICAL BASIS
TABLE OF CONTENTS/SIGNATURE PAGE

OPY # SEC60101

<u>SECTION</u>	<u>TITLE</u>	<u>REV #</u>	<u>PAGES</u>	<u>DATE</u>
T.O.C.	Table of Contents/Signature Page	07	3	10/10/00
i	Introduction and Usage	00	3	01/21/97
ii	Glossary of Acronyms & Abbreviations	00	6	01/21/97
1.0	Fuel Clad Challenge	01	4	12/29/99
2.0	RCS Challenge	02	2	07/24/00
3.0	Fission Product Barriers (Table)			
	3.1 Fuel Clad Barrier	01	20	12/29/99
	3.2 RCS Barrier	01	16	12/29/99
	3.3 Containment Barrier	03	25	07/24/00
4.0	EC Discretion	00	8	01/21/97
5.0	Failure to Trip	01	8	12/29/99
6.0	Radiological Releases/Occurrences			
	6.1 Gaseous Effluent Release	00	42	01/21/97
	6.2 Liquid Effluent Release	00	4	01/21/97
	6.3 In - Plant Radiation Occurrences	00	6	01/21/97
	6.4 Irradiated Fuel Event	00	10	01/21/97
7.0	Electrical Power			
	7.1 Loss of AC Power Capabilities	01	11	12/29/99
	7.2 Loss of DC Power Capabilities	00	8	01/21/97
8.0	System Malfunctions			
	8.1 Loss of Heat Removal Capability	01	10	12/29/99
	8.2 Loss of Overhead Annunciators	02	6	07/24/00
	8.3 Loss of Communications Capability	00	4	01/21/97
	8.4 Control Room Evacuation	00	4	01/21/97
	8.5 Technical Specifications	00	2	01/21/97
9.0	Hazards - Internal/External			
	9.1 Security Threats	01	8	10/10/00
	9.2 Fire	01	6	12/29/99
	9.3 Explosion	01	5	12/29/99
	9.4 Toxic/Flammable Gases	01	11	12/29/99
	9.5 Seismic Event	01	4	12/29/99
	9.6 High Winds	00	5	01/21/97
	9.7 Flooding	01	4	12/29/99
	9.8 Turbine Failure/Vehicle Crash/ Missile Impact	01	7	12/29/99
	9.9 River Level	00	4	01/21/97

**SALEM ECG TECHNICAL BASIS
TABLE OF CONTENTS/SIGNATURE PAGE**

<u>SECTION</u>	<u>TITLE</u>	<u>REV #</u>	<u>PAGES</u>	<u>DATE</u>
10.0	Reserved for future use			
Licensing is responsible for the Reportable Action Level (Section 11)				
11.0	Reportable Action Levels (RALs)			
11.1	Technical Specifications	00	8	01/21/97
11.2	Design Basis/ Unanalyzed Condition	01	7	03/13/97
11.3	Engineered Safety Features (ESF)	01	6	08/28/97
11.4	Personnel Safety/Overexposure	00	8	01/21/97
11.5	Environmental	00	3	01/21/97
11.6	After-the-Fact	00	1	01/21/97
11.7	Security/Emergency Response Capabilities	01	5	03/13/97
11.8	Public Interest	00	3	01/21/97
11.9	Accidental Criticality/ Special Nuclear Material / Rad Material Shipments - Releases	01	9	08/15/00
11.10	Voluntary Notifications	00	2	01/21/97

SIGNATURE PAGE

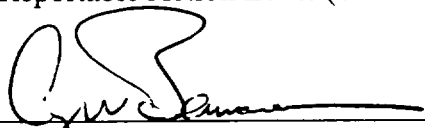
Prepared By: William Detwiler, Rev. 0 09/26/00
(If Editorial Revisions Only, Last Approved Revision) Date

Section/Attachments Revised: N/A 09/26/00
(List Non Editorial Only - Section/Attachments) Date

Reviewed By: N/A _____
Station Qualified Reviewer Date

Reviewed By: NA _____
Department Manager Date

Reviewed By: N/A _____
Manager - Licensing
(Reportable Action Level (Section 11)) Date

Reviewed By:  10/2/00
Manager - EP, & IT Date

Reviewed By: NA _____
Manager - Quality Assurance
(If Applicable) Date

SORC Review and Station Approvals

NA NA
Mtg. No. Salem Chairman Vice President - Nuclear Operations

NA NA
Date Date

Effective Date of this Revision: 10/10/00
Date

9.0 Hazards - Internal/External

9.1 Security Threats

UNUSUAL EVENT - 9.1.1

IC Confirmed Security Event Which Indicates a Potential Degradation in the Level of Safety of the Plant

EAL

Confirmed security threat directed towards the station as evidenced by ANY one of the following:

- Credible threat of malicious acts or destructive device within the Protected Area resulting in **SCP-5** implementation
- Credible intrusion or assault threat to the Protected Area resulting in **SCP-5** implementation
- Attempted intrusion or assault to the Protected Area resulting in **SCP-7 OR SCP-11** implementation
- Malicious acts attempted or discovered within the Protected Area resulting in **SCP-10** implementation
- Hostage/Extortion situation that threatens normal plant operations resulting in **SCP-8** implementation
- Destructive Device discovered within the Protected Area resulting in **SCP-10** implementation

MODE - All

BASIS

A security threat that is identified as being directed towards the station represents a potential degradation in the level of safety of the plant. The intent of this EAL is to classify security events, which threaten the Protected Area, but have not been determined to threaten plant vital areas.

A security threat is confirmed if physical evidence supporting the threat exists, if information independent from the actual threat exists, or if a specific group claims responsibility for the threat. The OS/EC will declare an Unusual Event upon consulting with Security to determine the validity of the entry conditions. Security Contingency Procedure (SCP) numbers are referenced following each EAL threshold. Since some SCP numbers appear in more than one

EAL - 9.1.1

Rev. 01

EAL, the on-duty Security Supervisor will provide information concerning the specific event to aid in classification.

Barrier Analysis

N/A

ESCALATION CRITERIA

This event will be escalated to an Alert based upon an actual Protected Area intrusion, malicious acts, or destructive devices discovered within a Vital Area.

DISCUSSION

Security events, which do not represent a potential degradation in the level of safety of the plant, are reported under RAL 11.7.1.a, an One Hour Non-Emergency Safeguards event. The following is an index of Security Contingency Procedures referenced by this event:

- SCP-5, "Security Threat"
- SCP-7, "Internal Disturbance"
- SCP-8, "Hostage Situation"
- SCP-10, "Discovery of Destructive Devices or Evidence of Malicious Acts"
- SCP-11, "Civil Disturbance"

DEVIATION

None

REFERENCES

NUMARC NESP-007, HU4.1, HU4.2
Safeguards Contingency Plan

9.0 Hazards - Internal/External

9.1 Security Threats

ALERT - 9.1.2

IC Security Event in a Plant Protected Area

EAL

Confirmed hostile intrusion or malicious acts as evidenced by ANY one of the following:

- Discovery of an intruder(s), armed and violent, within the Protected Area resulting in SCP-6 implementation
- Hostage held on-site in a non-vital area resulting in SCP-8 implementation

MODE - All

BASIS

This class of security event represents an escalated threat to the level of safety of the plant. This event is confirmed if physical evidence supporting the hostile intrusion or assault exists. The intent of this EAL is to classify security events that represent an actual intrusion into the plant Protected Area. The OS/EC will declare an Alert upon consulting with the Security to determine the validity of the entry conditions.

Security Contingency Procedure (SCP) numbers are referenced following each EAL threshold. Since some SCP numbers appear in more than one EAL, the on-duty Security Supervisor will provide information concerning the specific event to aid in classification.

Barrier Analysis

N/A

ESCALATION CRITERIA

This event will be escalated to a Site Area Emergency based upon a hostile intrusion in plant Vital Areas.

EAL - 9.1.2

Rev. 01

DISCUSSION

The following is an index of Security Contingency Procedures referenced by this event:

- **SCP-6**, "Discovery of Intruders or Attack"
- **SCP-8**, "Hostage Situation"

DEVIATION

None

REFERENCES

NUMARC NESP-007, HA4.1, HA4.2
Safeguards Contingency Plan

9.0 Hazards - Internal/External

9.1 Security Threats

SITE AREA EMERGENCY - 9.1.3

IC Security Event in a Plant Vital Area

EAL

Confirmed hostile intrusion or malicious acts in Plant Vital Areas as evidenced by:

- Discovery of an intruder(s), armed and violent, within the Vital Area, resulting in **SCP-6** implementation
- Malicious acts or destructive device discovered in a Vital Area, resulting in **SCP-10** implementation

MODE - All

BASIS

The first condition covered in this EAL covers a class of security events that represents an escalated threat to plant safety above that contained in an Alert. This EAL concerns a hostile intrusion or assault that has progressed from the Protected Area to a Vital Area. The Vital Areas are within the Protected Area and are generally controlled by key card readers. These areas contain vital equipment, which includes any equipment, system, device or material required for safe shutdown and for protection of the health and safety of the public and plant personnel.

The second condition covered in this EAL is intended to cover the discovery of malicious acts or destructive devices that could cause significant damage to a plant vital structure or more than one safety system. For malicious acts or destructive devices discovered in a plant vital structure that cannot cause significant damage to a plant vital structure or more than one safety system, evaluate EAL 9.1.1.

The Security Contingency Procedure (SCP) number is referenced following the EAL threshold. Since some SCP numbers appear in more than one EAL, the on-duty Security Supervisor will provide information concerning the specific event to aid in classification.

EAL - 9.1.3
Rev. 01

Barrier Analysis

N/A

ESCALATION CRITERIA

This event will be escalated to a General Emergency based upon the loss of physical control of the Control Room or Remote Shutdown Capability.

DISCUSSION

The following is an index of the Security Contingency Procedure referenced by this event:

- **SCP-6**, "Discovery of Intruders or Attack"
- **SCP-10**, "Discovery of Destructive Devices or Evidence of Malicious Acts"

DEVIATION

None

REFERENCES

NUMARC NESP-007, HS1.1, HS1.2
Safeguards Contingency Plan

9.0 Hazards - Internal/External

9.1 Security Threats

GENERAL EMERGENCY - 9.1.4

IC Security Event Resulting in Loss of Ability to Reach and Maintain Cold Shutdown

EAL

Security event resulting in the **actual loss of physical control** of EITHER one of the following:

- Control Room
- Remote Shutdown Panel 213

MODE - All

BASIS

Security events classified under this EAL represent conditions under which a hostile force has taken physical control of areas required to reach and maintain Cold Shutdown. Both the Control Room and Remote Shutdown Panel are included, since control of either could hamper the operating crew's ability to perform a safe plant shutdown. **Actual loss of physical control** is defined as the condition where licensed Control Room Operators can no longer take required action to operate the plant, including unauthorized transfer of plant equipment controlled from the Control Room.

Barrier Analysis

None

ESCALATION CRITERIA

N/A

DISCUSSION

The Remote Shutdown Panel 213 was the only panel included in this EAL due to its central location and ability to allow for **physical control** of multiple Safety Related components without detailed knowledge of plant operations. Security threats, which meet the threshold for declaration of a General Emergency, are an actual loss of physical control of the Control

EAL - 9.1.4

Rev. 01

Room or remote shutdown locations. This situation places the plant in a potentially unstable condition with high potential of multiple fission product barrier failures.

DEVIATION

None

REFERENCES

NUMARC NESP-007, HG1
Safeguards Contingency Plan