UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 12, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 192 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Claudia M. Craig

Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: December 23, 1999

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 12, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 197 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Claudia M. Craig

Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of issuance: December 23, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 192

TO FACILITY OPERATING LICENSE NO. DPR-24

AND LICENSE AMENDMENT NO. 197

TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE	INSERT
Table 15.3.5-5, p. 1 of 4	Table 15.3.5-5, p. 1 of 4
Table 15.3.5-5, p. 2 of 4	Table 15.3.5-5, p. 2 of 4
Table 15.3.5-5, p. 3 of 4	Table 15.3.5-5, p. 3 of 4
Table 15.3.5-5, p. 4 of 4	Table 15.3.5-5, p. 4 of 4
TS 15.4.6-1	TS 15.4.6-1

TABLE 15.3.5-5

INSTRUMENT OPERATING CONDITIONS FOR POST ACCIDENT MONITORING INSTRUMENTATION

NO.	FUNCTIONAL UNIT	1 TOTAL NO. OF CHANNELS	2 MINIMUM OPERABLE CHANNELS	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 2 CANNOT BE MET
1.	PORV Position Indicator	1/Valve	1/Valve	If the operability of the PORV position indicator cannot be restored within 48 hours, shut the associated PORV Block Valve.
2.	PORV Block Valve Position Indicator	1/Valve	1/Valve	If the operability of the PORV Block Valve Position Indicator cannot be restored within 48 hours, shut and verify the Block Valve shut by direct observation or declare the Block Valve inoperable.
3.	Safety Valve Position Indicator	2/Valve	1/Valve	If the operability of at least one of the Safety Valve Position Indicators cannot be restored within seven days, be in at least hot shutdown within the next 12 hours.
4.	Reactor Coolant System Subcooling	2	1	If operability of at least one subcooling monitor cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
5.	AFW Pump Discharge Flowrate	3	#	If the minimum number of AFW Pump Discharge Flowrate channels required to provide indication of AFW flow to both steam generators cannot be restored to an operable status within 48 hours, be in hot shutdown within the next twelve hours.

The minimum number of operable channels for AFW Pump Discharge Flowrate is the number of AFW Pump Discharge Flowrate channels, in conjunction with the number of operable AFW to Steam Generator Flowrate channels, required to provide indication of AFW flow to both steam generators.

NOTE: The channel requirements in this table refer only to that portion of the instrument channel required for post accident monitoring. The applicable channels are listed in FSAR Table 7.6-1.

TABLE 15.3.5-5 (continued)

NO.	FUNCTIONAL UNIT	1 TOTAL NO. OF CHANNELS	2 MINIMUM OPERABLE CHANNELS	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 2 CANNOT BE MET
6.	AFW to Steam Generator Flowrate	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
7.	Containment High Range Radiation	3	2	If the operability cannot be restored within seven days after failure, prepare a special report to be submitted within thirty days in accordance with 15.6.9.2.D.
8.	Containment Sump Level (Sump A)	2	1	Operation may continue up to thirty days. If operability cannot be restored, be in hot shutdown within the next twelve hours.
9.	Containment Sump Level (Sump B)	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
10.	Containment Hydrogen Concentration	2*	1	If operability cannot be restored within 72 hours, be in hot shutdown within the next six hours.
11.	Reactor Vessel Wide Range Level	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
12.	Reactor Vessel Narrow Range Level	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.

* With only one hydrogen monitor operable, restore an inoperable monitor with an independent power supply to an OPERABLE status within 30 days or be in hot shutdown within 6 hours.

NOTE: The channel requirements in this table refer only to that portion of the instrument channel required for post accident monitoring. The applicable channels are listed in FSAR Table 7.6-1.

TABLE 15.3.5-5 (continued)

NO.	FUNCTIONAL UNIT	1 TOTAL NO. OF CHANNELS	2 MINIMUM OPERABLE CHANNELS	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 2 CANNOT BE MET
13.	In-Core Thermocouples	39 installed per core	2/core quadrant	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
14.	Main Steam Line Radiation	1/steam line	1/steam line	If operability cannot be restored within seven days, prepare a special report to be submitted within thirty days in accordance with 15.6.9.2.E.
15.	Refueling Water Storage Tank Level	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
16.	RCS Wide Range Pressure	3	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
17.	RCS Narrow Range Pressure	4	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
18.	RCS Wide Range Hot Leg Temperature	2/loop	1/loop	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
19.	RCS Wide Range Cold Leg Temperature	2/loop	1/loop	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
20.	Pressurizer Level	4	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.

NOTE: The channel requirements in this table refer only to that portion of the instrument channel required for post accident monitoring. The applicable channels are listed in FSAR Table 7.6-1.

TABLE 15.3.5-5 (Continued)

NO.	FUNCTIONAL UNIT	1 TOTAL NO. OF CHANNELS	2 MINIMUM OPERABLE CHANNELS	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 2 CANNOT BE MET
21.	Containment Wide Range Pressure	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
22.	Containment Intermediate Range Pressure	3	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
23.	Containment Low Range Pressure	3	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
24.	Condensate Storage Tank Level	2/tank	1/tank	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
25.	Steam Generator Wide Range Level	2/SG	1/SG	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
26.	Steam Generator Narrow Range Level	3/SG	1/SG	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
27.	Steam Generator Pressure	3/SG	1/SG	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
28.	Containment Isolation Valve Position Indication	1	1	If the operability of the shut position indication of a containment isolation valve cannot be restored within seven days, close the valve or be in hot shutdown within the next twelve hours.

NOTE: The channel requirements in this table refer only to that portion of the instrument channel required for post accident monitoring. The applicable channels are listed in FSAR Table 7.6-1.

15.4.6 EMERGENCY POWER SYSTEM PERIODIC TESTS

Applicability

Applies to periodic testing and surveillance requirements of the emergency power system.

Objective

To verify that the emergency power system will respond promptly and properly when required.

Specification

The following tests and surveillance shall be performed as stated:

A. Diesel Generators

1. Manually-initiated start of the diesel generator, followed by manual synchronization with other power sources and assumption of load by the diesel generator shall not exceed 2850KW. This test will be conducted monthly with a minimum running time of 30 minutes on each diesel generator. Normal plant operation will not be affected.

2. Automatic start of each diesel generator, load shedding, and restoration to operation of particular vital equipment, initiated by an actual interruption of normal AC station service power supplies to associated engineered safety systems busses together with a simulated safety injection signal. In addition, after the diesel generator has carried its load for a minimum of 5 minutes, automatic load shedding and restoration of vital loads are tested again by manually tripping the diesel generator output breaker. This test will be conducted during reactor shutdown for major fuel reloading of each reactor to assure that the diesel generator will start and assume required load in accordance with the timing sequence listed in FSAR Section 8.8 after the initial starting signal.