

50-220



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 14, 1999

Mr. John H. Mueller
Chief Nuclear Officer
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Operations Building, Second Floor
P.O. Box 63
Lycoming, NY 13093

SUBJECT: REQUEST FOR RELIEF FOR THE THIRD 10-YEAR INTERVAL OF THE PUMP AND VALVE INSERVICE TESTING PROGRAM PLAN, NINE MILE POINT NUCLEAR STATION, UNIT NO. 1 (TAC NO. MA5957)

Dear Mr. Mueller:

By letter dated June 23, 1999, Niagara Mohawk Power Corporation (NMPC and the licensee) submitted the third 10-year interval inservice testing program plan for pumps and valves at Nine Mile Point Nuclear Station, Unit 1 (NMP1). The third interval will begin on December 26, 1999. NMPC developed the program plan to the requirements of the 1989 Edition of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) by implementing the 1987 ASME/American National Standards Institute's Operations and Maintenance (OM) Standards, 1988a Addenda, Part 6, "Inservice Testing of Pumps in Light-Water Reactor Power Plants," and Part 10, "Inservice Testing of Valves in Light-Water Reactor Power Plants." The program plan includes one pump relief request (PMP-RR-1), two general valve relief requests (GEN-VR-1 and GEN-VR-2), and two system-specific valve relief requests (CRS-VR-1 and CTN-SP-VR-1). NMPC requested that the U.S. Nuclear Regulatory Commission (NRC or Commission) approve these relief requests.

The NRC staff has reviewed the proposed alternative testing methods contained within the program plan against the requirements of the 1989 Edition of the ASME Code pursuant to Section 50.55a of Part 50 to Title 10 of the Code of Federal Regulations. The results are presented in the enclosed safety evaluation, in which the NRC staff:

- (1) grants PMP-RR-1 for the third 10-year interval pursuant to 10 CFR 50.55a(f)(6)(i) after having considered the impracticality of performing the required testing and the burden on the licensee if the requirements were to be imposed;
- (2) authorizes, for the third 10-year interval, the proposed alternative to the ASME Code requirements described in GEN-VR-1, pursuant to 10 CFR 50.55a(a)(3)(i), based upon the alternative providing an acceptable level of quality and safety; and
- (3) approves, pursuant to 10 CFR 50.55a(f)(4)(iv), the proposed alternatives to the Code requirements described in GEN-VR-2, CRS-VR-1 and CTN-SP-VR-1.

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This completes the NRC staff's review under TAC No. MA5957. If you have any questions regarding this matter, please contact Darl Hood by phone on 301-415-3049 or by electronic mail at dsh@nrc.gov.

Sincerely,

Original signed by A.Dromerick for

Sheri Peterson, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosure: Safety Evaluation

cc w/encl: See next page

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J. Mueller

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Sincerely,

Sheri Peterson, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

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Enclosure: Safety Evaluation

cc w/encl: See next page

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