



Point Beach Nuclear Plant
6610 Nuclear Rd., Two Rivers, WI 54241

(920) 755-2321

NPL 99-0644

December 9, 1999

Document Control Desk
U. S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKETS 50-266 AND 50-301
MONTHLY OPERATING REPORTS
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Attached are monthly operating reports for Units 1 and 2 of the Point Beach Nuclear Plant for the calendar month of November, 1999.

Sincerely,

A handwritten signature in black ink, appearing to read "A. J. Cayia", is written over a printed name and title.

A. J. Cayia
Manager,
Regulatory Services & Licensing

MBK/tja

Attachments

cc: J. D. Loock, PSCW
NRC Regional Administrator, Region III
NRC Resident Inspector

JE24

OPERATING DATA REPORT

DOCKET NO. 50-266

DATE: 12/08/99

COMPLETED BY: M. B. Koudelka

TELEPHONE: (920) 755-6480

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 1
2. REPORTING PERIOD: November - 1999
3. LICENSED THERMAL POWER (MWT): 1,518.5
4. NAMEPLATING RATING (GROSS MWE): 537.7
5. DESIGN ELECTRICAL RATING (NET MWE): 515.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 530.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 510.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):
NA

NOTES

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	8,016.0	254,808.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	6,600.2	206,946.7
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	667.3
14. HOURS GENERATOR ONLINE	0.0	6,523.3	203,540.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	846.9
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	9,765,120.0	289,604,423.0
17. GROSS ELECTRICAL ENERGY GENERATED	0.0	3,402,530.0	98,089,630.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.0	3,250,542.5	93,502,267.0
19. UNIT SERVICE FACTOR	0.0%	81.4%	79.9%
20. UNIT AVAILABILITY FACTOR	0.0%	81.4%	80.2%
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0%	79.9%	75.3%
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0%	79.0%	73.8%
23. UNIT FORCED OUTAGE RATE	0.0%	3.9%	4.7%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): None			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 12/09/99			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POINT BEACH NUCLEAR PLANT

AVERAGE DAILY UNIT POWER LEVEL

MONTH NOVEMBER - 1999

DOCKET NO. 50-266
 UNIT NAME: Point Beach, Unit 1
 DATE: 12/02/99
 COMPLETED BY: M. B. Koudelka
 TELEPHONE: (920) 755-6480

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-2</u>	21	<u>-2</u>
2	<u>-2</u>	12	<u>-2</u>	22	<u>-2</u>
3	<u>-2</u>	13	<u>-2</u>	23	<u>-2</u>
4	<u>-2</u>	14	<u>-2</u>	24	<u>-2</u>
5	<u>-2</u>	15	<u>-2</u>	25	<u>-2</u>
6	<u>-2</u>	16	<u>-2</u>	26	<u>-5</u>
7	<u>-2</u>	17	<u>-2</u>	27	<u>-8</u>
8	<u>-2</u>	18	<u>-3</u>	28	<u>-9</u>
9	<u>-1</u>	19	<u>-2</u>	29	<u>-9</u>
10	<u>-2</u>	20	<u>-2</u>	30	<u>-14</u>

DOCKET NO. 50-266
UNIT NAME Point Beach Unit 1
DATE 12/03/99
COMPLETED BY M. B. Koudelka
TELEPHONE 920/755-6480

Unit 1 Refueling 25 began on October 15, 1999 and is to be back online on December 9, 1999.

Three Licensee Event Reports (LERs) were submitted to the NRC during November, 1999:

LER 266/1999-008-00, Postulated Fire Could Lead to Loss of Redundant Trains of Charging Capacity.

LER 266/1999-009-00, Design Basis for Control Room/Cable Spreading Room Fire Barrier not Fully Implemented.

LER 266/1999-010-00, Inadequate Appendix R Emergency Lighting.

Safety-related maintenance included:

1. 1A-05 4.16 kV bus switchgear (safeguards) cleaned, inspected and differential overcurrent relays replaced.
2. 1A-06 4.16 kV bus switchgear (safeguards) deenergized bus inspection performed.
3. 1A52-16A feed to 1B-03 from 1X-13 station service transformer starwheel inspection performed.
4. 1A52-57 tie to bus 1A-03 (1A00-38) breaker position indication plunger and links removed.
5. 1A52-58 power to 1X-13 station service transformer starwheel inspection performed.
6. 1A52-59 power to 1P-15A safety injection pump breaker position indication plunger and links removed.
7. 1A52-60 normal power to 1A-05 from G-01 emergency diesel generator breaker position indication plunger and links removed.
8. 1A52-66 alternate power to 1A-05 from G-02 emergency diesel generator breaker position indication plunger and links removed.
9. 1A52-77 bus 1A-04 to 1A-06 bus tie breaker replaced.
10. 1A52-78 spare breaker inspected and tested.
11. 1A52-79 spare breaker inspected and tested.

12. 1A52-81 bus 1A-06 feed to motor control center 1B-40 breaker replaced.
13. 1A52-83 spare breaker inspected.
14. 1A52-84 bus 1A-06 feed to 1X-14 station service transformer breaker replaced.
15. 1A52-85 power to 1P-15B safety injection pump relay calibration performed.
16. 1AF-00100 and 1AF-00101 HX-1A&B steam generator auxiliary feedwater first-off valves replaced.
17. AF-00140 AF-4012 discharge control AF-4053 nitrogen backup regulator in PI-4054 isolation valve replaced.
18. AF-00141 AF-4012 discharge control AF-4053 nitrogen backup regulator in PI-4054 vent valve replaced.
19. AF-00143 P-38A auxiliary feedwater pump AF-4012 nitrogen backup north bottle outlet isolation valve replaced.
20. AF-00144 P-38A auxiliary feedwater pump F-4012 nitrogen backup north bottle outlet vent valve replaced.
21. AF-00146 P-38A auxiliary feedwater pump AF-4012 nitrogen backup south bottle outlet isolation valve replaced.
22. AF-00147 P-38A auxiliary feedwater pump AF-4012 nitrogen backup south bottle outlet vent valve replaced.
23. 1AF-04000 and 1AF-04001 P-29 auxiliary feedwater pump discharge HX-1A&B steam generator inlet isolation motor-operated valve limit switches and wiring reconfigured.
24. AF-04019 P-38A auxiliary feedwater pump discharge control valve flow scan performed.
25. AF-04058 AF-4019 auxiliary feedwater pump discharge control nitrogen backup regulator replaced.
26. 1B-04 480 V safeguards load center inspected and meter checks performed.
27. 1B-32 480 V motor control center primary auxiliary building safeguards as-built inspection performed.
28. 1B-42 480 V motor control center primary auxiliary building safeguards temporary power installed for 1B-04 outage (removed following that outage).
29. 1B-42 480 V motor control center primary auxiliary building safeguards as-built walkdown performed on safety injection-related buckets in 1342 MCC.

30. 1B52-12A normal power to 1B312A-B855B 1P-10A residual heat removal pump transformer switch breaker inspected and reduced voltage testing performed.
31. 1B52-16B feed to 1B-03 from 1X-13 station service transformer reduced voltage test performed.
32. 1B52-17B feed to 1B-04 from 1X-14 station service transformer reduced voltage test and starwheel inspections performed.
33. 1B52-21A normal power to 1B421A-B855B 1P-10B residual heat removal pump transformer switch reduced voltage test performed.
34. Emergency close switches were installed on the contactors of the following components: 1B42-391 D-07 station battery charger Train A contactor; 1B42-491 D-09 swing station battery charger Train B contactor; 1B42-3212H D-109 swing station battery charger Train A contactor; and 1B42-494 D-108 station battery charger Train B contactor.
35. C-001 Unit 1 and Unit 2 safeguards control board status indication window replaced.
36. 1C-1-D1V C-1 outer bulkhead door equalizing vent valve packing and ball seal replaced.
37. 1C-2-D1L C-2 outer bulkhead door latching mechanism ball spring plunger replaced.
38. 1C-3 containment fuel transfer tube flange alignment pins repaired.
39. 1C-110 pressurizer pressure and level control panel computer point lugs replaced.
40. 1C-128 reactor coolant system panel computer point lugs replaced.
41. C-177 white computer input mux power supply replaced.
42. 1CC-00738B HX-11B residual heat removal heat exchanger shell side inlet valve repacked.
43. 1CC-00759A P-1A reactor coolant pump component cooling outlet motor-operated valve cleaned and inspected.
44. 1CC-00759A-O P-1A reactor coolant pump component cooling outlet motor-operated valve operator replaced.
45. 1CC-00764A P-1A reactor coolant pump upper oil cooler component cooling outlet valve repacked.
46. 1CC-00815 T-12 component cooling surge tank emergency makeup water inlet valve repacked.
47. 1CRDM-G3 and 1CRDM-G11 control rod drive mechanism at positions G3 and G11 jack shafts replaced.
48. 1CS-00466 and 1CS-00476 HX-1A&B steam generator regulator control valves inspected and maintained.

49. 1CV-00206B P-1B reactor coolant pump No. 1 seal water return bypass first-off isolation valve cleaned.
50. 1CV-00270A P-1A reactor coolant pump No. 1 seal water return valve cleaned.
51. 1CV-00369A residual heat removal to letdown crossconnect valve gasket replaced.
52. 1CV-01401 HX-2 regenerative heat exchanger outlet to letdown orifices test connection repacked.
53. 1CV-D-11 HX-4 electric heat exchanger outlet drain valve pipe to valve root pass replaced.
54. Circuit breakers were replaced in the following areas: 1A52-57 tie to bus 1A-03 (1A00-38); 1A52-58 power to 1X-13 station service transformer; 1A52-59 power to 1P-15A safety injection pump; 1A52-60 normal power to 1A-05 from G-01 emergency diesel generator; and 1A52-66 alternate power to 1A-05 from G-02 emergency diesel generator.
55. Containment penetrations were inspected and paint recoated as necessary in the following areas: 1CPP-36 HX-15C containment recirculation heat exchanger supply; 1CPP-37 HX-30A cavity cooling coil supply; 1CPP-40 HX-15D containment recirculation heat exchanger supply; 1CPP-45 HX-30A cavity cooling coil return; 1CPP-48 HX-15D containment recirculation heat exchanger return; 1CPP-52 spare; 1CPP-51 HX-1A steam generator blowdown; 1CPP-50 HX-1B steam generator blowdown; 1CPP-47 HD-15B containment recirculation heat exchanger return; 1CPP-46 HX-30B cavity cooling coil return; 1CPP-44 HX-15C containment recirculation heat exchanger return; 1CPP-43 HX-15A containment recirculation heat exchanger return; 1CPP-39 HX-15B containment recirculation heat exchanger supply; 1CPP-38 HX-30B cavity cooling coil supply; 1CPP-35 HX-15A containment recirculation heat exchanger supply; 1CPP-04 main feedwater; 1CPP-03 main feedwater; 1CPP-02 main steam Loop B; and, 1CPP-01 main steam Loop A.
56. 1DI-00011 U1C header demineralized water supply valve replaced.
57. 1HS-7 HX-1A steam generator south side snubber rebuilt.
58. 1HS-11 HX-1A steam generator feedwater line snubber rebuilt.
59. 1HS-2501R-15 pressurizer power-operated relief valve header snubber rebuilt.
60. 1HS-2501R-22A pressurizer power-operated relief valve header snubber rebuilt.
61. 1HX-001A&B steam generator tubesheets cleaned and steam drum inspected.
62. 1HX-015A1 containment accident recirculation heat exchanger coil repaired and tested.
63. 1HX-015A8 containment accident recirculation heat exchanger header nipple expansion plugs installed.

64. 1HX-015B4 containment accident recirculation heat exchanger tube removed, evaluated, repaired and tested.
65. 1HX-015C1 containment accident recirculation heat exchanger coil tube repaired.
66. 1HX-015C1-C8 containment accident recirculation heat exchanger insulation installed.
67. 1HX-015D W-1D1 accident recirculation fan motor cooling coil repaired and cooler installed.
68. 1HX-030A1-A4 and 1HX-030B1-B4 containment cavity cooling coil drip pans cleaned.
69. 1IA-03047 U1C instrument air header inlet control valve limit switches adjusted.
70. 1IA-03048 U1C instrument air header inlet control valve limit switches adjusted.
71. 1MS-2005, 1MS-2007, 1MS-2010 and 1MS-2011 HX-1A&B steam generator header safety valves tested.
72. 1MS-02016 HX-1A steam generator header atmospheric steam dump control valve ultrasonic test performed.
73. 1MS-02020 HX-1A steam generator header P-29 auxiliary feedwater pump steam supply motor-operated valve cleaned and torqued bolts.
74. 1MS-05958 HX-1A steam generator blowdown isolation valve flow scanner diagnostic test performed.
75. 1N-00031 nuclear instrumentation source range channel preamp soldered and crimped.
76. 1N-00043 nuclear instrumentation power range channel walkdown and calibration performed.
77. 1P-001B reactor coolant pump inspected, repaired and seal package replaced.
78. 1P-001B-M reactor coolant pump motor replaced.
79. 1P-010A-M residual heat removal pump motor bearings replaced and oil changed..
80. 1P-010B-M residual heat removal pump motor cleaned and bearings replaced.
81. 1P-011A component cooling water pump mechanical seals and bearings replaced.
82. 1P-011B component cooling water pump repaired and oil changed.
83. 1P-014B-M containment spray pump motor bearings replaced.
84. 1P-015A safety injection pump control switch replacement tested and inboard gland plate repaired.

85. 1P-015A-M and 1P-015B-M safety injection pump rotating assemblies replaced and oil changed in motors and pump bearings.
86. 1PT-00922 P-15B safety injection pump discharge pressure transmitter calibrated.
87. 1R-1 reactor vessel and assembly inspected and Swagelok fittings tightened.
88. 1RC-00431A T-1 pressurizer spray line pressure control valve gasket replaced.
89. 1RC-00434 T-1 pressurizer safety to T-2 pressure relief tank tested and repaired.
90. 1RC-00515 and 1RC-00516 T-1 pressurizer RC-430 and RC-431C power-operated relief isolation valves replaced.
91. 1RC-00539 T-2 pressurizer relief tank gas analyzer system isolation valve snap lock switch adjusted.
92. 1RH-00700-M reactor coolant system to P-10A&B residual heat removal pump suction header motor inspected.
93. 1RH-00714A RH-626 residual heat removal heat exchanger bypass flow control outlet valve inspected and repacked.
94. 1RH-00718A P-10A residual heat removal pump suction check valve inspected and maintained
95. Relays were replaced in the following areas: 1RCP-A-XA Loop A Train A reactor coolant pump loss of power trip matrix; 1PC-00484A-XA Train A high condenser pressure matrix; 1RCP-B-XA Loop B Train A reactor coolant pump loss of power matrix; 1SI-11X-AXA Train A safety injection to reactor trip matrix relay; and 1SI-21X-BXA Train A safety injection to reactor trip matrix.
96. 1SC-00959 residual heat removal loop sample isolation valve actuator spring replaced and valve repacked.
97. 1SC-00966C reactor coolant hot leg sample valve actuator spring replaced and valve repacked.
98. 1SI-00842A&B T-34A&B safety injection accumulator outlet check valve Section XI tested.
99. 1SI-00851A-O and 1SI-00851B-O P-10A&B residual heat removal pump suction from containment Sump B motor-operated valve torque switches replaced.
100. 1SI-00856A-O and 1SI-00856B-O T-13 refueling water storage tank outlet to P-010A&B residual heat removal pump suction header operator motor-operated valve torque switches replaced.
101. 1SI-00857A HX-11A residual heat removal heat exchanger outlet to P-015A safety injection pump suction valve stem drive nut replaced.

102. 1SI-00860A P-14A containment spray pump discharge redundant isolation valve repacked.
103. 1SI-00860D-M P-14B containment spray pump discharge redundant isolation valve motor replaced.
104. 1SI-00866A-O and 1SI-00866B-O cold leg and core deluge injection line isolation valve motor-operated valve torque switches replaced.
105. 1SI-00866B core deluge injection line isolation valve flow and differential pressure tests performed.
106. 1SI-00867B safety injection system to reactor coolant Loop B cold leg check valve Section XI tested.
107. 1SI-00878A&C P-15B safety injection pump R-1 reactor vessel injection valve handwheel housing bolts replaced.
108. 1SI-00878C-O P-15B safety injection pump R-1 reactor vessel injection valve operator torque switch replaced.
109. 1SI-00885A&B SI-852A&B core deluge injection thermal release check valve replaced.
110. 1SI-00897A safety injection test line return valve Swagelok fittings tightened.
111. SW-00652 and SW-00653 HX-13A&B spent fuel pool heat exchanger inlet valve motor operators installed.
112. SW-02911-BS north service water header Zurn strainer coupling spider replaced.
113. 1SW-02959 HX-15A containment recirculation heat exchanger outlet relief valve replaced.
114. 1SW-02963 HX-15B containment recirculation heat exchanger outlet relief valve setpoint tested.
115. TS-04139 W-12C G-02 emergency diesel generator room exhaust fan temperature switch replaced.
116. 1W-001A1 containment accident recirculation fan control switch replaced.
117. 1W-001A2-A containment cooling fan discharge damper bearing repaired.
118. 1W-001B1 containment accident recirculation fan bearings inspected.
119. 1W-001B1-M containment accident recirculation fan motor bearings replaced.
120. 1W-001B1 1W-1B1 containment accident recirculation fan control switch replaced.
121. 1W-001C1 containment accident recirculation fan inspected.

122. 1W-001D1-M containment accident recirculation fan motor flow switch linkage replaced and motor tested.
123. 1-49/A52-85 1P-15B safety injection pump thermal overload relay calibrator and resistor inspected.
124. 1-273/A01 1A-01 bus undervoltage relay internal output relay replaced.
125. 1-274/A04 1A-01 bus undervoltage relay internal output relay replaced.

POINT BEACH NUCLEAR PLANT
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER - 1999

Docket No. 50-266
 Unit Name Point Beach, Unit 1
 Date 12/8/1999
 Completed By M. B. Koudelka
 Telephone No. 920/755-6480

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Reactor Shut Down ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
5	991015	S	720	C	1	NA	NA	NA	The unit began its 25 th refueling outage on October 15, 1999 and is scheduled back on-line December 9, 1999.

¹F: Forced
 S: Scheduled

²Reason:
 A - Equipment Failure (explain)
 B - Maintenance or Testing
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Exam
 F - Administrative
 G - Operational Error (explain)
 H - Other (explain)

³Method:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Continuation of Previous Shutdown
 5 - Reduced Load
 6 - Other (explain)

⁴Exhibit G - Instructions for preparation of data entry sheets LER file (NUREG-0161)

⁵Exhibit I - Same Source

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE: 12/02/99

COMPLETED BY: M. B. Koudelka

TELEPHONE: (920) 755-6480

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 2
2. REPORTING PERIOD: November - 1999
3. LICENSED THERMAL POWER (MWT): 1,518.5
4. NAMEPLATING RATING (GROSS MWE): 537.7
5. DESIGN ELECTRICAL RATING (NET MWE): 515.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 532.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 512.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):
NA

NOTES

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	8,016.0	239,593.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	6,649.4	201,190.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	233.9
14. HOURS GENERATOR ONLINE	720.0	6,452.2	198,358.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	302.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,091,483.0	9,538,781.0	285,330,837.0
17. GROSS ELECTRICAL ENERGY GENERATED	386,190.0	3,342,410.0	97,100,750.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	369,952.0	3,192,720.5	92,535,139.0
19. UNIT SERVICE FACTOR	100.0%	80.5%	82.8%
20. UNIT AVAILABILITY FACTOR	100.0%	80.5%	82.9%
21. UNIT CAPACITY FACTOR (USING MDC NET)	100.4%	78.2%	79.0%
22. UNIT CAPACITY FACTOR (USING DER NET)	99.8%	77.6%	77.6%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	2.3%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): None			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA			

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE 12/03/99
COMPLETED BY M. B. Koudelka
TELEPHONE 920/755-6480

The daily power average for Unit 2 during November, 1999, was 513.8 MWe.

No Licensee Event Reports (LERs) were submitted to the NRC during November, 1999.

Safety-related maintenance included:

1. 2MS-00249 HX-1A steam generator header sample first-off isolation valve prefabrication work performed for MS-00249 and MS-00249A valves and piping assembly.
2. 2P-002A charging pump cleaned, inspected, and packing, O-rings, gasket and plunger replaced.
3. 2SI-00857A&B HX-11A&B residual heat removal heat exchanger outlet to P-15A&B safety injection pump suction valve operators replaced.
4. 2SI-00897A&B safety injection test line return valve and second-off isolation valve actuators and valve/pipe supports replaced.

POINT BEACH NUCLEAR PLANT

AVERAGE DAILY UNIT POWER LEVEL

MONTH NOVEMBER - 1999

DOCKET NO. 50-301
 UNIT NAME: Point Beach, Unit 2
 DATE: 12/02/99
 COMPLETED BY: M. B. Koudelka
 TELEPHONE: (920) 755-6480

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>513</u>	11	<u>518</u>	21	<u>515</u>
2	<u>513</u>	12	<u>509</u>	22	<u>516</u>
3	<u>514</u>	13	<u>514</u>	23	<u>514</u>
4	<u>515</u>	14	<u>515</u>	24	<u>515</u>
5	<u>515</u>	15	<u>510</u>	25	<u>513</u>
6	<u>515</u>	16	<u>513</u>	26	<u>516</u>
7	<u>513</u>	17	<u>513</u>	27	<u>515</u>
8	<u>510</u>	18	<u>514</u>	28	<u>514</u>
9	<u>514</u>	19	<u>514</u>	29	<u>514</u>
10	<u>513</u>	20	<u>514</u>	30	<u>514</u>

POINT BEACH NUCLEAR PLANT
UNIT SHUTDOWNS AND POWER REDUCTIONS
 REPORT MONTH NOVEMBER - 1999

Docket No. 50-301
 Unit Name Point Beach, Unit 2
 Date 12/6/1999
 Completed By M. B. Koudelka
 Telephone No. 920/755-6480

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Reactor Shut Down ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
--	NA	--	NA	NA	NA	NA	NA	NA	NA

¹F: Forced
 S: Scheduled

²Reason:
 A - Equipment Failure (explain)
 B - Maintenance or Testing
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Exam
 F - Administrative
 G - Operational Error (explain)
 H - Other (explain)

³Method:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Continuation of Previous Shutdown
 5 - Reduced Load
 6 - Other (explain)

⁴Exhibit G - Instructions for preparation of data entry sheets LER file (NUREG-0161)

⁵Exhibit I - Same Source