

50-269/270/287



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 10, 1999

LICENSEE: Duke Energy Corporation (Duke)
FACILITY: Oconee Nuclear Station, Units 1, 2, and 3
SUBJECT: SUMMARY OF DISCUSSIONS BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC) STAFF AND DUKE REPRESENTATIVES TO DISCUSS MATERIALS RELATED ISSUES REGARDING THE OCONEE LICENSE RENEWAL APPLICATION (LRA)

The purpose of this summary is to document several discussions the staff has had with Duke regarding materials related issues for the Oconee LRA. Specifically, this summary documents discussions regarding the reactor vessel monitoring pipe, BAW-2248, "Demonstration of the Management of Aging Effects for the Reactor Vessel Internals," and an issue related to cast austenitic stainless steel (CASS).

By letter dated October 27, 1999, the staff issued questions to the Babcock and Wilcox Owner's Group (B&WOG) regarding BAW-2251, "Demonstration of the Management of Aging Effects for the Reactor Vessel." The questions related to the reactor coolant pressure boundary relative to the O-rings on the reactor vessel flange and the monitoring pipes for the reactor vessel flange. Because Duke references BAW-2251 in its LRA for Oconee, a safety evaluation report open item (SER OI 3.4.3.3-9) was added to track the resolution of the item for Oconee. Enclosure 1 contains Duke's initial reaction to the October 27, 1999, letter to the B&WOG. Based on Duke's initial reaction the staff asked the questions contained in Enclosure 2. A phone call was held with Duke on November 10, 1999, to discuss the questions contained in Enclosure 2. The staff participants in the call were Steve Koenick, Barry Elliot, Keith Wichman, Stephanie Coffin, and Joe Sebrosky. The Duke participant was Bob Gill.

Based on the results of the November 10, 1999, phone call the staff amended its questions regarding the reactor vessel monitoring pipe. The amended questions can be found in Enclosure 3. With the exception of minor editorial changes, the questions contained in Enclosure 3 were formally sent to Duke in a November 18, 1999, letter as part of the issues that need to be resolved prior to issuance of the final safety evaluation report for the Oconee LRA.

Regarding BAW-2248, the staff sent Duke a letter dated October 27, 1999, that contained the proposed license renewal applicant action items for BAW-2248. The staff requested that Duke provide a cross-reference table for how the license renewal applicant action items were resolved for the Oconee LRA. Enclosure 4 provides Duke's cross-reference table. Based on Duke's response contained in Enclosure 4, the staff determined that it needed more information regarding renewal applicant action items number 4 and number 8. Therefore, renewal applicant action items number 4 and 8 from BAW-2248 are included as issues that need to be resolved in the November 18, 1999, letter to Duke.

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Regarding the CASS issue, there was a phone call held on November 17, 1999. The staff participants in the call were Steve Koenick, Barry Elliot, Allen Hiser and Joe Sebrosky. The Duke participants were Bob Gill, Greg Robison, and Mark Rinckel (Framatome Technologies Incorporated). The purpose of the phone call was to discuss a statement that Duke made in a February 17, 1999, letter to the staff. Specifically, on page 70 of the February 17, 1999, letter in response to request for additional information (RAI) 5.4.1-1 Duke stated the following:

The main coolant piping material that is susceptible to thermal aging is the cast austenitic stainless steel (CASS) portion of the welded joint that connects the CASS reactor coolant pump suction and discharge nozzles to the 28-inch wrought stainless steel transition pieces.

The staff requested clarification of what material was CASS relative to the reactor coolant pump casings. Duke agreed with the staff that the above statement contained in the response to RAI 5.4.1-1 was misleading. To clarify what material was CASS relative to the reactor coolant pump casing Duke provided the drawing contained in Enclosure 5. Duke also referred the staff to figure 2-7 of BAW-2243, "Demonstration of the Management of Aging Effects for the Reactor Coolant System Piping." The staff agreed that the drawings contained in Enclosure 5 and Figure 2-7 of BAW-2243 clarifies what materials are CASS. Therefore, the staff considers this issue resolved.

A draft of this summary was provided to Duke to allow them the opportunity to comment on the summary prior to issuance.

Original Signed By

Joseph M. Sebrosky, Project Manager
License Renewal and Standardization Branch
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosures: As stated (5)

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NAME	EHylton	AHiser	BElliot	JSebrosky <i>JM</i>	CIGrimes
DATE	12/06/99*	12/10/99*	12/10/99*	12/13/99	12/10/99*

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OFFICE	LA	EMCB/DE	EMCB/DE	RLSB/DRIP:PM	RLSB:D
NAME	EKylton	AHiser	BElliot	JSebrosky	CIGrimes
DATE	12/16/99	12/10/99	12/10/99	12/11/99	12/10/99

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Oconee Nuclear Station (License Renewal)

cc:

Ms. Lisa F. Vaughn
Duke Energy Corporation
422 South Church Street
Mail Stop PB-05E
Charlotte, North Carolina 28201-1006

Anne W. Cottingham, Esquire
Winston and Strawn
1400 L Street, NW
Washington, DC 20005

Mr. Rick N. Edwards
Framatome Technologies
Suite 525
1700 Rockville Pike
Rockville, Maryland 20852-1631

Manager, LIS
NUS Corporation
2650 McCormick Drive, 3rd Floor
Clearwater, Florida 34619-1035

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
7812B Rochester Highway
Seneca, South Carolina 29672

Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, Georgia 30303

Virgil R. Autry, Director
Division of Radioactive Waste Management
Bureau of Land and Waste Management
Department of Health and
Environmental Control
2600 Bull Street
Columbia, South Carolina 29201-1708

W. R. McCollum, Jr., Vice President
Oconee Site
Duke Energy Corporation
P. O. Box 1439
Seneca, SC 29679

Mr. Larry E. Nicholson
Compliance Manager
Duke Energy Corporation
Oconee Nuclear Site
P. O. Box 1439
Seneca, South Carolina 29679

Ms. Karen E. Long
Assistant Attorney General
North Carolina Department of Justice
P. O. Box 629
Raleigh, North Carolina 27602

L. A. Keller
Manager - Nuclear Regulatory Licensing
Duke Energy Corporation
526 South Church Street
Charlotte, North Carolina 28201-1006

Mr. Richard M. Fry, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health, and
Natural Resources
3825 Barrett Drive
Raleigh, North Carolina 27609-7721

Gregory D. Robison
Duke Energy Corporation
Mail Stop EC-12R
P. O. Box 1006
Charlotte, North Carolina 28201-1006

Robert L. Gill, Jr.
Duke Energy Corporation
Mail Stop EC-12R
P. O. Box 1006
Charlotte, North Carolina 28201-1006
RLGILL@DUKE-ENERGY.COM

Douglas J. Walters
Nuclear Energy Institute
1776 I Street, NW
Suite 400
Washington, DC 20006-3708
DJW@NEI.ORG

Chattooga River Watershed Coalition
P. O. Box 2006
Clayton, GA 30525

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RLSB RF

S. Duraiswamy, ACRS - T2E26

E. Hylton

E-mail:

R. Zimmerman

D. Matthews

S. Newberry

C. Grimes

C. Carpenter

B. Zalcman

J. Strosnider

R. Wessman

E. Imbro

G. Bagchi

W. Bateman

J. Calvo

M. Tschiltz

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M. Mayfield

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M. Modes

RLSB Staff

R. Emch

D. LaBarge

L. Plisco

C. Ogle

R. Trojanowski

D. Billings

M. Shannon

C. Julian

J. Wilson

C. Sochor

J. Vora