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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

December 7, 1999

Mr. Otto L. Maynard  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, KS 66839

**SUBJECT: WOLF CREEK GENERATING STATION - AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NO. NPF-42 (TAC NO. MA4572)**

Dear Mr. Maynard:

The Commission has issued the enclosed Amendment No. 130 to Facility Operating License No. NPF-42 for the Wolf Creek Generating Station (WCGS). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated December 29, 1998 (WO 98-0104), as supplemented by letters dated July 29 (ET 99-0031) and October 21, 1999 (ET 99-0044).

The amendment revised (1) the reactor coolant system (RCS) heatup and cooldown limit curves in Figures 3.4-2 and 3.4-3, and cold overpressure mitigation system power-operated relief valve setpoint limit curve in Figure 3.4-4 of the current TSs, and (2) the list of references in Section 5.6.6 on the RCS pressure temperature limits report (PTLR) in the improved TSs. The improved TSs were issued in Amendment No. 123, dated March 31, 1999, to replace the current TSs, but have not yet been implemented.

The revision to Section 5.6.6 of the improved TSs replaced the previous references to NRC documents giving criteria for the above limit curves in the current TSs by the references to (1) the NRC letter of December 2, 1999, that approved the use of the PTLR of Generic Letter 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits," dated January 31, 1996, for WCGS, and (2) WCAP-14040-NP-A, "Methodology Used to Develop Cold Overpressure Mitigation System Setpoints and RCS Heatup and Cooldown Limit Curves."

The PTLR will provide the methodology for your staff to revise the heatup and cooldown and setpoint limit curves for WCGS in the future without prior staff approval, after the improved TSs are implemented and have replaced the current TSs. The improved TSs are to be implemented by December 31, 1999.

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In your application, you also requested an exemption to certain requirements in 10 CFR 50.60, "Acceptance Criteria for Fracture Prevention Measures for Lightwater Nuclear Power Reactors for Normal Operation," that requires all light water nuclear power reactors to meet the fracture toughness and material surveillance program requirements for the reactor coolant pressure boundary set forth in Appendices G and H to 10 CFR Part 50. You requested an exemption to use American Society of Mechanical Engineers (ASME) Code Case N-514 as an alternative to the ASME Code requirements for reactor vessel pressure limits at low temperatures. The exemption would allow you to apply the code case to determine the cold overpressure mitigation system (COMS) pressure setpoint. The exemption was granted in the staff's letter of April 30, 1999, under TAC No. MA4955.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,



Jack N. Donohew, Senior Project Manager, Section 2  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-482

- Enclosures: 1. Amendment No. 130 to NPF-42
- 2. Safety Evaluation

cc w/encls: See next page

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Mr. Otto L. Maynard

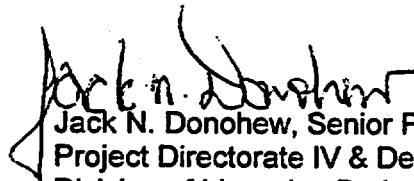
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Docket No. 50-482

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2. Safety Evaluation

cc w/encls: See next page

**Wolf Creek Generating Station**

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